

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Surry Power Station, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 8 0	PAGE (3) 1 OF 0 3
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TITLE (4)
Auto Start of Auxiliary Feedwater Pumps

EVENT DATE (5)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																								
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<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9) N</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="6">POWER LEVEL (10) 0 0 2</td> <td>20.402(b)</td> <td>20.408(a)</td> <td><input checked="" type="checkbox"/></td> <td>90.73(a)(2)(iv)</td> <td>73.71(b)</td> </tr> <tr> <td>20.405(a)(1)(i)</td> <td>90.38(a)(1)</td> <td></td> <td>90.73(a)(2)(v)</td> <td>73.71(c)</td> </tr> <tr> <td>20.405(a)(1)(ii)</td> <td>90.38(a)(2)</td> <td></td> <td>90.73(a)(2)(vii)</td> <td rowspan="4">OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td>20.405(a)(1)(iii)</td> <td>90.73(a)(2)(i)</td> <td></td> <td>90.73(a)(2)(viii)(A)</td> </tr> <tr> <td>20.405(a)(1)(iv)</td> <td>90.73(a)(2)(ii)</td> <td></td> <td>90.73(a)(2)(viii)(B)</td> </tr> <tr> <td>20.405(a)(1)(v)</td> <td>90.73(a)(2)(iii)</td> <td></td> <td>90.73(a)(2)(ix)</td> </tr> </table>												OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)											POWER LEVEL (10) 0 0 2	20.402(b)	20.408(a)	<input checked="" type="checkbox"/>	90.73(a)(2)(iv)	73.71(b)	20.405(a)(1)(i)	90.38(a)(1)		90.73(a)(2)(v)	73.71(c)	20.405(a)(1)(ii)	90.38(a)(2)		90.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.405(a)(1)(iii)	90.73(a)(2)(i)		90.73(a)(2)(viii)(A)	20.405(a)(1)(iv)	90.73(a)(2)(ii)		90.73(a)(2)(viii)(B)	20.405(a)(1)(v)	90.73(a)(2)(iii)		90.73(a)(2)(ix)
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LICENSEE CONTACT FOR THIS LER (12)

NAME R. F. Saunders, Station Manager	TELEPHONE NUMBER AREA CODE: 8 0 4 3 5 7 1 3 1 8 4
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	S	J H C V	F 1 1 3 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH: DAY: YEAR:
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On February 8, 1986, at 2% reactor power, during a Unit 1 startup, the operating main feedwater pump tripped due to a high level in "C" steam generator. This caused the auxiliary feedwater pumps to auto start.

The high steam generator level occurred when the "C" main feedwater bypass valve (E11S HCV) failed to close on demand. This valve was found to have a dust accumulation in the air pilot relay which blocked air to the valve operator. The blockage was removed and the valve closed. Steam generator levels were returned to normal.

Engineering will evaluate methods for controlling contamination in the instrument air system.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (if more space is required, use additional NRC Form 366A's) (17)

Auto Start of Auxiliary Feedwater Pumps

1. Description of the Event

On February 8, 1986, at 2% reactor power, a Unit 1 startup was in progress following a reactor trip on 2/7/86 (See LER 86-008). 1-FW-P-1A ("A" main feedwater pump) was in service, and operators were controlling steam generator levels with the main feedwater bypass valves. Following a level increase in "C" steam generator, the operator closed 1-FW-HCV-155C ("C" main feedwater bypass valve). However, level continued to increase and at 0553 hours, the operating main feedwater pump tripped due to high level in "C" steam generator. This caused auxiliary feedwater pumps 1-FW-P-3A and 3B to auto start.

2. Safety Consequences and Implications

The auxiliary feedwater pumps operated as designed to provide residual heat removal to the steam generators when the normal feedwater supply was lost. The main feedwater pump tripped on high steam generator level to prevent water from entering the main steam line and turbine. All safety related systems remained operable and plant parameters remained within the bounds of the safety analysis. Therefore, this event did not constitute an unreviewed safety question and the health and safety of the public were not affected.

3. Cause

The auto start of the auxiliary feedwater pumps was due to the loss of the "A" main feedwater pump which tripped as a result of a high level in "C" steam generator. The high steam generator level occurred when 1-FW-HCV-155C failed to close on demand. This valve was found to have a dust accumulation in the air pilot relay, which blocked air to the valve operator.

4. Immediate Corrective Action

After verifying adequate levels in the steam generators, the operator stopped the auxiliary feedwater pumps and restarted the "A" main feedwater pump.

5. Additional Corrective Action

Instrument Technicians removed the blockage in the air pilot relay for 1-FW-HCV-155C and the valve closed. Steam generator blowdown of 70 gpm was established and levels were returned to normal.

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TEXT (if more space is required, use additional NRC Form 388A's) (17)

6. Action Taken to Prevent Recurrence

An engineering review will be conducted to evaluate the most effective method for controlling contaminants in the instrument air system.

7. Generic Implications

None.

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

Surry Power Station
P. O. Box 315
Surry, Virginia 23883

February 28, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No: 86-009
Docket No: 50-280
License No: DPR-32

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

REPORT NUMBER

86-009-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

R. F. Saunders

R. F. Saunders
Station Manager

Enclosure

cc: Dr. J. Nelson Grace
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

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