

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285  
 UNIT Fort Calhoun Station  
 DATE September 9, 1988  
 COMPLETED BY W. J. Blessie  
 TELEPHONE 402-536-4595

MONTH August 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	408	17	365
2	406	18	362
3	407	19	364
4	410	20	365
5	414	21	367
6	416	22	368
7	416	23	369
8	416	24	369
9	418	25	368
10	418	26	370
11	419	27	372
12	418	28	374
13	415	29	375
14	411	30	375
15	406	31	374
16	377		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

8809200237 880831  
 PDR ADOCK 05000285  
 R PDC

IE24  
 1/1

# OPERATING DATA REPORT

DOCKET NO.	50-285
UNIT	Fort Calhoun Station
DATE	September 9, 1988
COMPLETED BY	W. J. Blessie
TELEPHONE	402-536-4595

## OPERATING STATUS

- |   | Notes |
|---|-------|
| 1. Unit Name: <u>Fort Calhoun Station</u>   |       |
| 2. Reporting Period: <u>August 1988</u>   |       |
| 3. Licensed Thermal Power (Mwt): <u>1500</u>  |       |
| 4. Nameplate Rating (Gross MWe): <u>502</u>   |       |
| 5. Design Electrical Rating (Net MWe): <u>478</u>   |       |
| 6. Maximum Dependable Capacity (Gross MWe): <u>50%</u>  |       |
| 7. Maximum Dependable Capacity (Net MWe): <u>478</u>  |       |
| 8. If changes occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:<br><u>N/A</u> |       |
| <hr/>   |       |
| 9. Power Level to Which Restricted, If Any (Net MWe): <u>N/A</u>  |       |
| 10. Reasons for Restrictions, If Any:<br><u></u>  |       |
| <hr/>   |       |

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	<u>744.0</u>	<u>5855.0</u>	<u>130,927.0</u>
12. Number of Hours Reactor Was Critical	<u>744.0</u>	<u>5855.0</u>	<u>102,694.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,309.5</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>5855.0</u>	<u>101,773.8</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>951,709.3</u>	<u>7,861,092.8</u>	<u>132,408,718.0</u>
17. Gross Electrical Energy Generated (MWH)	<u>307,298.0</u>	<u>2,619,322.0</u>	<u>43,592,253.2</u>
18. Net Electrical Energy Generated (MWH)	<u>290,624.0</u>	<u>2,488,273.8</u>	<u>41,632,356.6</u>
19. Unit Service Factor	<u>100.0</u>	<u>100.0</u>	<u>77.7</u>
20. Unit Availability Factor	<u>100.0</u>	<u>100.0</u>	<u>77.7</u>
21. Factor (Using MDC Net)	<u>81.7</u>	<u>88.9</u>	<u>69.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>81.7</u>	<u>88.9</u>	<u>67.7</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>2.9</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
The 1988 Refueling Shutdown is tentatively scheduled for September 24, 1988 with startup tentatively scheduled for December 10, 1988.

25. If Shut Down at End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):    Forcast                      Achieved

INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A	<u>      </u>	<u>      </u>
COMMERCIAL OPERATION		<u>      </u>	<u>      </u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285  
 UNIT NAME Fort Calhoun Station  
 DATE September 9, 1988  
 COMPLETED BY W. J. Blessie  
 TELEPHONE (402) 536-4595

REPORT MONTH AUGUST 1988

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason 2	Method of Shutting Down Reactor 3	Licensee Event Report#	System Code 4	Component Code 5	Cause & Corrective Action to Prevent Recurrence
88-03	880816	S	0	H	4	N/A	SD	COND	On August 16, 1988, power was further reduced (from 90% to 80%) due to an increase in river water temperature coupled with a poor performing condenser. This caused a further decrease in the cooling capabilities of the condenser and is limiting the electrical output of the generator. The unit is remaining at 80%.

1  
 F-Forced  
 S-Scheduled

2  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error  
 H-Other (Explain)

3  
 Method:  
 1-Manual  
 2-Manual Scram  
 3-Automatic Scram  
 4-Other (Explain)

4  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5  
 Exhibit 1 - Same Source

Refueling Information  
Fort Calhoun - Unit No. 1

Report for the month ending August 1988

1. Scheduled date for next refueling shutdown. September 1988
2. Scheduled date for restart following refueling. December 1988
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
  - a. If answer is yes, what, in general, will these be?
    - Change total planar radial peaking factor limit from 1.85 to 1.80
    - Change core inlet temperature from 545°F to 543°F
    - Clarification of LHR LCO
    - Miscellaneous changes
  - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. \_\_\_\_\_
  - c. If no such review has taken place, when is it scheduled? \_\_\_\_\_
4. Scheduled date(s) for submitting proposed licensing action and support information. August 1988
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:
 

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>393</u>	"
c) spent fuel pool storage capacity	<u>729</u>	"
d) planned spent fuel pool storage capacity	May be increased via fuel pin consolidation "	
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1994\*

\*Full core offload of 133 assemblies lost.

Prepared by *Jim Holt* Date August 25, 1988

OMAHA PUBLIC POWER DISTRICT  
Fort Calhoun Station Unit No. 1

August 1988  
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 90% power until August 15, 1988, when a power reduction to 80% was initiated due to continued problems with condenser efficiency and high river temperatures. Power remained at 80% to conserve reactivity due to the extended shutdown date for the refueling outage. During August two operators and two engineers received Senior Reactor Operator licenses, and one operator received a Reactor Operator license. NRC inspections were performed in health physics addressing outage preparation and dosimetry, Appendix R triennial inspection on fire protection and a five-man region closeout team for open items. The chemical control program was implemented.

No safety valves or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

All tests and inspections were done on schedule with acceptable results.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure

Description

SP-STROKE-1

In Service Testing of Air Operated, CQE Valves.

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it only allowed stroke testing to be conducted on 38 air operated CQE valves. The objective of the testing is to determine if valve operability was degraded (or is degrading) due to the intrusion of water into the instrument air system. This testing did not in any way compromise plant safety, but enhanced it by ensuring operability of safety related valves.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (continued)

System Acceptance Committee Packages for August 1988:

<u>Package</u>	<u>Description/Analysis</u>
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EEAR FC-86-010	QSPDS Software Changes.
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This modification provided for new EPROMS for the QSPDS. This modification does not affect any safety analyses under Technical Specifications.

E. RESULTS OF LEAK RATE TESTS

The leak rate test on the containment purge valves (penetrations M-87 and M-88) was completed on July 19, 1988, per ST-CONT-3, F.4. The "as found" and "as left" leak rates for penetrations M-87 and M-88 were all 0 sccm. The test frequency of ST-CONT-3, F.4, has been increased as a result of the unsatisfactory "as found" leak rate of penetration M-87 during the December 1987 test of the containment purge valves.

The "as left" "B" and "C" leak rate was 374.2 sccm following the June 3, 1988, leak rate test. Since both sets of containment purge valves tested zero leakage during the July 19, 1988, test, the total leakage remains unchanged. This leak rate is well below the allowed leakage of  $0.6 L_a$  as specified in 10 CFR 50 Appendix J.

F. CHANGES IN PLANT OPERATING STAFF

Mr. Eugene Riggs received a Reactor Operators license. Mr. Carl Jensen and Mr. Robert Luikens each received a Senior Reactor Operators license. Mr. John Adams was promoted from Operations Engineer to Reactor Engineer. Mr. Jerry Shuck was promoted from Nuclear Production Engineer to Operations Engineer and received a Senior Reactor Operators license. Mr. Jens Friedrichsen received a Senior Reactor Operators license.

G. TRAINING

During the month of August, notification was received that all five individuals sitting for the NRC license examinations passed (4 SRO's and 1 RO). Work continued on preparation of Job Performance Measures (JPM's) and questions for the licensed requalification exam bank.

G. TRAINING (continued)

Non-licensed and licensed operator requalification, STA requalification, and EONT training continued. Orientation training was conducted for system engineers. The first half of technical staff training was completed. The Supervisor-Chemistry and Radiation Protection Training has worked with the Metropolitan Technical Community College to establish a degree program for Radiation Protection Technicians. Effective August 1988, the Board of Governors of Metropolitan Technical Community College approved an associate degree program in Radiological Health Technology. Radiation Protection Senior Technician contractors for the upcoming refueling outage were screened by an entrance examination prior to being accepted to work at the Fort Calhoun Station. Senior Technicians who failed to pass the entrance examination will not be allowed to work at Fort Calhoun Station. Two Chemistry instructors are rotating shifts with the Shift Chemistry Technicians to qualify and to improve the OJT process. Maintenance training completed precutoutage administrative training for Central Maintenance personnel. Observation Training has been taught continually through the month of August to qualify appropriate personnel prior to the outage. General Employee Training is providing additional classes to handle outage personnel.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

II. MAINTENANCE (Significant Safety Related)

See attached printout.

*W. Gary Gates*

W. Gary Gates  
Manager-Fort Calhoun Station

FILE: EQUIPMENT M0 MASTER

COMPLETED CQE MAINTENANCE ORDERS FOR AUGUST

SORT: 30090

MO NUMBER *****	ITEM TAG *****	PROBLEM GRPT *****
883287	AC-3C	OUTBOARD BEARING TEMP. EXCEEDED THE 170 DEGREE UPPER LIMIT.
883301	SI-8D	PI-286B IS READING LOW
883330	FW-6	LEAK OFF DRAINS ARE PLUGGED. ALSO PUMP SEALS HAVE EXCESSIVE LEAK
883043	VA-297	TIGHTENED LOOSE BOLTS IN ROOM 59 IN THE AUX BLDG.
883293	LI-3B	PUMP PIPING NOTED TO HAVE TWO 2" U-BOLTS MISSING. SEE TOM BARTLEY
883886	AC-7	REMOVE MANWAY FROM AC-7 AND INSPECT CONDITION AND LEVEL OF EXISTING RESI
883193	PS-6026	MONITOR PRESSURE SWITCH CONTACTS IN ORDER TO DETERMINE THE TIME TAKEN FO
883484	VIT-6288B	VIT-6288B FLOWRATER HAS MOISTURE IN IT. NEEDS TO BE CLEANED.
883483	VIT-6288A	VIT-6288A FLOWRATER HAS MOISTURE IN IT. NEEDS TO BE CLEANED.
883410	AI-31C-EW11-M1	OPERATORS ARE RECEIVING A 'NUCLEAR DELTA T POWER DEVIATION' ALARM.
881964	AC-1C	AC-1C - REMOVE TEMP. BRICK WALL AT END OF AC-1C ROOM 1B (SUPPORT ECT).
883497	EE-1G	EXHAUST LINE SUPPORTS MAY NOT SEISMICALLY ADEQUATE, (BASED ON PRELIMINAR
883496	EE-1F	EXHAUST LINE SUPPORTS MAY NOT SEISMICALLY ADEQUATE, (BASED ON PRELIMINAR
883150	PCV-742F	A CONTRACT PERSON FELL FROM THE OVERHEAD WHILE PAINTING AND BUMPED THE I
883170	FT-2891	TESTING PER MO-883048 FOUND ZERO TO BE OFF ON THE TRANSMITTER. CALIB-
883619	SI-370	BONNET NUT NOT INSTALLED. VALVE IN OVERHEAD
883647	CW-3A	FAULTY "A" PHASE TRIP DEVICE
881864	HCV-291B	THE PIN THAT IS HOLDING ON THE HAND WHEEL NEEDS A COTTER PIN INSTALLED.
883091	HCV-292B	CYCLE HCV-292B TO REVERIFY OPERABILITY. WHILE PERFORMING ST-SI/CS-1, CO
883114	CG-1B	SECTION STABILIZER IS LEAKING WATER FROM THE TOP FLANGE.
883542	QSPDS	EVEN STRING OF "B" QSPDS HJTC PROBE APPEARS TO BE FAILING.
873844	AI-RPS	WHILE PERFORMING ST-RPS-11, SECTION F, 1, STEP 1.B, 15 ON THE B/D MATRIX
882987	CH-14	NEED TO CHECK DRAIN VALVE ON LOW PRESSER ACCUMULATOR, SEEMS TO BE STUCK C
883643	VIT-6288A	ROTAMETER CONTAINS MOISTURE MAKING IT DIFFICULT TO ADJUST FLOW.
883644	VIT-6288B	ROTAMETER CONTAINS MOISTURE MAKING IT DIFFICULT TO ADJUST FLOW.
883344	CH-1B	PACKING COOLANT PUMP GAGE LEAKS.
840279	AC-134	AC-134 LEAKS VERY BADLY. WHEN SHUT WILL NOT HOLD WATER.
887058	AI-RPS	BAD CONNECTION ON RPSCP DRAWER. NUCPOWER/BT DEVIATION ANNUNCIATOR ACTUA



EMATA PUBLIC POWER DISTRICT  
FORT CALHOUN PLANT

\*\*\*\*\* C H A M P S \*\*\*\*\*

DATE 09/14/88  
TIME 10:11  
PAGE 2  
RVT 051-01

FILE: EQUIPMENT MO MASTER

COMPLETED COE MAINTENANCE ORDERS FOR AUGUST

MO  
NUMBER  
\*\*\*\*\*  
873694

ITEM  
TAG  
\*\*\*\*\*

PROBLEM  
GRP1  
\*\*\*\*\*

SPARE 4160V BREAKER S/N 20448837-017NEEDS TO BE INSPECTED AND REPAIRED.

ITEMS SELECTED = 000029

**Omaha Public Power District**  
1623 Harney Omaha, Nebraska 68102-2247  
402/536-4000

September 15, 1988  
LIC-88-718

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Mail Station P1-137  
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: August Monthly Operating Report

Pursuant to Technical Specification Section 5.9.1, and 10 CFR Part 50.4(b)(1), please find enclosed one copy of the August 1988 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,



K. J. Morris  
Division Manager  
Nuclear Operations

KJM/wjb

Enclosures

- c: P. H. Harrell - NRC Senior Resident Inspector  
NRC Regional Office  
Office of Management & Program Analysis (2)  
R. M. Caruso - Combustion Engineering  
R. J. Simon - Westinghouse  
Nuclear Safety Analysis Center  
INPO Records Center  
American Nuclear Insurers  
NRC File (FCS)

JE24  
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