

LICENSEE EVENT REPORT (LER)

Facility Name (1) Byron, Unit 2 Docket Number (2) 0 5 0 0 0 4 5 5 Page (3) 1 of 0 2

Title (4) FEEDWATER ISOLATION ACTUATION ON HIGH-2 STEAM GENERATOR LEVEL DUE TO PERSONNEL ERROR

Event Date (5)			LER Number (6)			Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
0 8	2 9	8 7	8 7	0 1 5	0 1	0 9	1 3	8 8	NONE	0 5 0 0 0 1 1
										0 5 0 0 0 1 1

OPERATING MODE (9) 4

POWER LEVEL (10) 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> Other (Specify in Abstract below and in Text)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

Name D. Brindle, Unit 2 Operating Engineer Ext. 2216

TELEPHONE NUMBER AREA CODE 8 1 5 2 3 4 - 5 4 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

Yes (If yes, complete EXPECTED SUBMISSION DATE) NO

Expected Submission Date (15): Month Day Year

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 29, 1987, at 0156, Byron Unit 2 was in Mode 4 at zero percent power. Instrument and control technicians (non-licensed) were calibrating one of the 2D Steam Generator level transmitters. The technician in containment isolated the wrong level transmitter. This action satisfied a 2 of 4 coincidence causing a feedwater isolation on Steam Generator High-2 level, P-14. The feedwater isolation actuated as designed. The level transmitter was unisolated and the Feedwater Isolation Signal was reset.

The cause of the actuation was a cognitive personnel error on the part of the instrument and control technician. The technician was disciplined.

There have been no previous occurrences of this nature.

8809200221 880913
PDR ADOCK 05000455
S PDC

FEED
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			Page (3)		
		Year	Sequential Number	Revision Number			
Byron, Unit 2	0 5 0 0 0 4 5 5	8 7	- 0 1 5	- 0 1	0 2	OF	0 2

TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [xx]

A. PLANT CONDITIONS PRIOR TO EVENT:

Event Date/Time 8/29/87 / 0156 hrs.

Unit 2 MODE 4 - Hot Shutdown Rx Power 0 RCS [AB] Temperature/Pressure 330°F/340 psig

B. DESCRIPTION OF EVENT:

On 8/29/87 Byron Unit 2 was in Mode 4 at zero percent power. At 0156 hours instrument and control technicians (non-licensed) were calibrating the 2D Steam Generator (FW)[JB] level transmitter 2LT-559, which had its bi-stables tripped and in the test mode. The instrument and control technicians in containment isolated the wrong transmitter, 2LT-549. This satisfied a 2 out of 4 coincidence required to cause a Feedwater Isolation on Steam Generator High-2 level, P-14. All the safety actuations associated with the P-14 permissive operated properly. The 2LT-549 transmitter was unisolated and the Feedwater Isolation signal was reset.

C. CAUSE OF EVENT:

The Feedwater Isolation was due to a cognitive personnel error by the instrument technician. The Feedwater Isolation Signal was received due to the isolation of one level transmitter on the 2D Steam Generator while another level transmitter was in test. A P-14 actuation was received after a 2 out of 4 coincidence was satisfied on Steam Generator 2D level.

D. SAFETY ANALYSIS:

The plant or public safety was not affected by the Feedwater Isolation actuation. In Mode 4, most Feedwater Isolation valves were already closed. Plant conditions did not require a Feedwater Isolation. The safety systems operated properly. If this event had occurred at power, the Reactor Protection System would have actuated per design for a loss of feedwater event.

E. CORRECTIVE ACTIONS:

The incident was discussed with the instrument and control technician and a letter was placed in his personnel file and he was placed on a one year probation. The incident was also discussed in a general meeting with all the technicians and foremen.

The Trip Prevention Committee has investigated the use of additional identification on different instruments in an attempt to prevent technician errors. The conclusion was that, in this case, the event was an isolated technician error, that the equipment is readily identifiable, and that no additional identification is needed.

F. PREVIOUS OCCURRENCES:

LER NUMBER	TITLE
NONE	

G. COMPONENT FAILURE DATA:

a)	MANUFACTURER	NOMENCLATURE	MODEL NUMBER	MFG PART NUMBER
	Not Applicable			

b) RESULTS OF NPRDS SEARCH:

Not Applicable



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

September 13, 1988

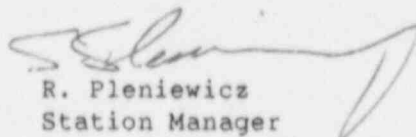
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted as a supplemental report.

This report is number 87-015; Docket No. 50-455.

Sincerely,



R. Pleniewicz
Station Manager
Byron Nuclear Power Station

Enclosure: Licensee Event Report No. 87-015-01

cc: A. Bert Davis, NRC Region III Administrator
P. Brochman, NRC Senior Resident Inspector
INPO Record Center
CECo Distribution List

Ltr: BYRON 88-0904 (1921M/0206M)

(1651M/0196M)

IE22
11