

October 5, 1998

Mr. Michael B. Roche  
Vice President and Director  
GPU Nuclear, Inc.  
Oyster Creek Nuclear Generating Station  
P.O. Box 388  
Forked River, New Jersey 08731

SUBJECT: NRC INSPECTION REPORT NO. 50-219/98-07 - REPLY

Dear Mr. Roche:

This letter refers to your September 3, 1998 correspondence, in response to our July 2, 1998 letter.

Thank you for informing us of the clarifications and commitments documented in your letter. These actions pertain to your program implemented in response to Generic Letter 89-10, "Safety-related Motor-Operated Valve Testing and Surveillance." Your commitments will be examined during a future inspection of your licensed program.

Your cooperation with us is appreciated.

Sincerely,

Eugene M. Kelly, Chief  
Systems Engineering Branch  
Division of Reactor Safety

Docket No. 50-219

cc:  
M. Laggart, Manager, Licensing and Vendor Audits  
G. Busch, Manager, Nuclear Safety and Licensing  
State of New Jersey

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Mr. Michael B. Roche

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September 3, 1998  
1940-98-20411

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Gentlemen:

Subject: Oyster Creek Nuclear Generating Station (OCNGS)  
Docket No. 50-219  
Facility Operating License No. DPR-16  
Response to NRC Motor Operated Valve Inspection  
Report No. 50-219/98-07

GPU Nuclear's review of NRC Inspection Report (IR) No. 50-219/98-07, Motor Operated Valve Program Review, identified the following areas requiring clarification or update:

1. IR 98-07, Section E1.1.b, page 3, states that GPU Nuclear plans to increase the design capability of valve V-14-30 by replacing the motor power cable during the 17R refueling outage. This statement is correct based on previously docketed GPU Nuclear commitments and accurately reflected the 17R outage plan at the time of the inspection. However, recent assessment indicates that the valve design margin based on a preliminary Electric Power Research Institute (EPRI) Performance Prediction Methodology (PPM) thrust determination accounting for Limitorque Bulletin 98-01 Guidance is considered sufficient for Cycle 17 operation. Therefore, GPU Nuclear is deferring this modification to V-14-30 to the 18R refueling outage. Replacement of the power cable to valve V-14-36 is still planned for the 17R refueling outage.
2. IR 98-07, Section E1.1.b, page 4, states that GPU Nuclear plans to dynamically test both shutdown cooling system valves, V-17-19 and V-17-54, in the 17R refueling outage. GPU Nuclear is planning to dynamically test only the shutdown cooling system pump discharge valve (V-17-54) during the 17R outage. A dynamic test of the pump suction valve (V-17-19) is not practical.

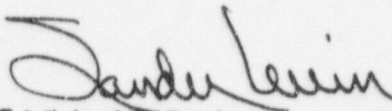
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3. IR 98-07, Section E1.1.b, page 4, last paragraph, states GPU Nuclear committed to dynamically test four valves in the drywell spray and shutdown cooling systems. Based on the clarification described in Item 2 above, it should be noted that this dynamic testing includes the shutdown cooling system valve V-17-54 (Group 4), the containment spray system valves V-21-15 and V-21-18 (Group 6), and the containment spray valve V-21-13 (Group 5).

As discussed during the subject inspection, provided in the attachment to this letter is a summary of the remaining OCNGS Generic Letter 89-10 Program docketed commitments. It is noted that EPRI PPM calculations, previously committed to be completed by 18R (Fall 2000), have been recently completed as shown on the attachment.

If any additional information is needed, please contact Mr. David J. Distel, GPU Nuclear Licensing and Regulatory Affairs at (973) 316-7955.

Sincerely,

  
for Michael B. Roche  
Vice President and Director  
Oyster Creek

/DJD

Attachment: OCNGS Generic Letter 89-10 Program Commitments

cc: Administrator, Region I - H. J. Miller  
NRC Region I - D. Dempsey  
OCNGS Senior Project Manager - R. Eaton  
OCNGS Senior Resident Inspector - J. Schoppy

## ATTACHMENT

### OCNGS Generic Letter 89-10 Program Commitments

#### A. Valve Modifications

1. Containment Spray Valves V-21-5 and V-21-11 to be modified (torque switch adjustment) in 17R (Fall 1998) to provide improved thrust margins to accommodate the EPRI PPM maximum predicted thrust.
2. Isolation Condenser Valves V-14-36 to be modified in 17R (Fall 1998) and V-14-30 to be modified in 18R to improve available voltage and design margin. Modification consists of installing larger power cables to the valve motors.
3. Isolation Condenser Valves V-14-36 and V-14-37 to be modified in 18R (Fall 2000) to install new guides or discs, or to install new replacement valves.
4. Reactor Water Cleanup Valves V-16-1, V-16-2, V-16-14, and V-16-61 to be modified in 17R (Fall 1998) to install replacement valve discs with hard-faced disc guide areas.

#### B. EPRI PPM Calculations

1. Complete EPRI PPM calculation for Isolation Condenser Valves V-14-36 and V-14-37 in 18R (Fall 2000). This calculation was completed on July 24, 1998.
2. Complete EPRI PPM calculation for Reactor Water Cleanup Valves V-16-1, V-16-2, V-16-14, and V-16-61 in 17R (Fall 1998). This calculation was completed on July 24, 1998.
3. Complete EPRI PPM calculation for Core Spray Valves V-20-15, V-20-21, V-20-40, and V-20-41 in 18R (Fall 2000). This calculation was completed on July 24, 1998.

#### C. Valve Testing

1. Containment Spray Valves V-21-15 and V-21-18 will be dynamically tested in 17R (Fall 1998) to confirm the established valve factor.
2. Containment Spray Valve V-21-13 and Shutdown Cooling System Valve V-17-54 to be dynamically tested in 17R (Fall 1998) in accordance with Generic Letter 96-005 Joint Owner, Group Program Periodic Verification Testing commitment.

#### D. Program Documentation Updates

1. Convert rate-of-loading methodology to the EPRI method for valve stroke time calculation in 17R (Fall 1998).
2. Upgrade weak link calculation and complete design verification in 17R (Fall 1998).
3. Evaluate EPRI/NMAC testing of stem lubricant Synco Super Lube EP-2 to determine impact on rate-of-loading assumptions by 18R (Fall 2000).
4. Revise hand calculations for Anchor-Darling double-disc valves to reference NRC SER limitations and conditions on the EPRI PPM and include Revision 2 of EPRI TR-103237 by December 31, 1998.