

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

APRIL 1988

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

IE24
1/1

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Steven J. Samolinski, telephone number (815)357-6761, extension 705.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

April 1-30

April 1, 0000 hours

Unit One entered April with the Reactor subcritical and the generator off line. Unit One is in second refuel outage.

April 30, 2400 hours

Unit One in second refuel outage. Reactor vessel is defueled.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility License or Technical Specification.

There were no amendments to the Facility License or Technical Specifications during this reporting period.

2. Changes to procedures which are described in the Safety Analysis Report.

There were no changes made to procedures which are described in the Safety Analysis Report during this reporting period.

3. Tests and Experiments not described in the Safety Analysis Report.

There were no tests or experiments conducted which are not described in the Safety Analysis Report during this reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a Summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

5. Completed Safety Related Modifications.

The following Table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L72508	LPCI water leg pump min. flow orifice 1E12-D002	Corrosion of orifice	Low pressure in B/C RHR loop	Replaced orifice
L72931	Rx pressure vessel safety relief valve	Malfunction of fitting on sensing line	Sensing line leaking	Replaced fitting and performed leak test
L75866	HPCS cooling water strainer 1E22-F319	Packing malfunction	Valve leaking	Repacked valve
L76682	Valve 1E32-F001J	Packing malfunction	Valve leaking	Replaced packing
L76833	Rx low level relay 1E22A-K7	Contacts not properly adjusted	Contacts not making connection	Adjusted, cleaned and tested contacts
L79142	1B D/G overcurrent relay 1E22B-K7	Weak tension in contact arm for contacts 7 & 8	Prevents overcurrent alarm from being received	Readjusted contact arms
L79403	1A D/G 1DG01K	Malfunctioning fuel injectors	Low exhaust temp. on 2 cylinders	Replaced fuel injectors on cylinder 20 and cylinder 10
L79728	APRM Channel "A" 1C51-K605GM	Shorted output circuits on Quad trip card	ESF actuation	Replaced card
L79755	Under voltage relays 1327-AP108A/B	Faulty operation of undervoltage relays	Improper load shedding for bus 136Y	Cleaned under voltage relays

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER: A brief synopsis of incorporated modification objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

UNIT ONE

- M-1-0-85-013: Installation of a third radio frequency. The new frequency will allow the security force a second usable frequency and another frequency for the new clip on pager system.
- M-1-1-82-160: Revise the refrigerated air dryer control for D/G 0, 1A, 1B, and 2A to allow continuous operation of the dryers.
- M-1-1-84-116: Installation of Potter & Brumfield 5138 relays in the 1A D/G control circuitry. These relays replace the K6 and K36 relays.
- M-1-1-87-022: Remove two existing pipe supports and a section of line 1VQ02AB to improve Drywell Cooling system ventilation.
- M-1-1-87-063: Snubber reduction on the primary containment combustible gas control system (HG) per the snubber reduction program.
- M-1-1-87-071: Snubber reduction on the Diesel Generator piping subsystem per the snubber reduction program.
- M-1-1-87-075: Snubber reduction on the MSIV leakage control system piping subsystem per the snubber reduction program.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, April 1, 1988 through April 30, 1988. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
88-004-00	4/22/88	Loss of RPS Bus A while Replacing Relay.
88-005-00	4/22/88	1A Diesel Generator Failure to Meet Load Acceptance.
88-006-00	4/17/88	Inoperative conductivity monitor due to Reactor Vessel drain down for chemical cleaning. (voluntary)

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE May 10, 1988
 COMPLETED BY S. J. Samolinski
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: April, 1988 GROSS HOURS IN REPORTING PERIOD: 719
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY
 (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 0
 4. REASONS FOR RESTRICTION (IF ANY): Refuel Outage
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|--------------|----------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>0.0</u> | <u>1729.8</u> | <u>21774.3</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>1642.0</u> |
| 7. HOURS GENERATOR ON LINE | <u>0.0</u> | <u>1729.8</u> | <u>21159.8</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>0.0</u> | <u>5252088</u> | <u>57086386</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>0.0</u> | <u>2227148</u> | <u>19194407</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>-2920</u> | <u>2149510</u> | <u>18248151</u> |
| 12. REACTOR SERVICE FACTOR | <u>0.0%</u> | <u>59.6%</u> | <u>57.3%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>0.0%</u> | <u>59.6%</u> | <u>61.6%</u> |
| 14. UNIT SERVICE FACTOR | <u>0.0%</u> | <u>59.6%</u> | <u>55.7%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>0.0%</u> | <u>59.6%</u> | <u>55.7%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>-0.4%</u> | <u>71.5%</u> | <u>46.4%</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>-0.4%</u> | <u>68.7%</u> | <u>44.6%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>0.0%</u> | <u>0.0%</u> | <u>13.2%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 None
20. IF SHUT DOWN IS AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
 Unit One is in the second refuel outage and is scheduled to startup July 3, 1988.

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: May 10, 1988
COMPLETED BY: S. J. Samolinski
TELEPHONE: (815) 357-6761
MONTH: APRIL, 1988

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____ - 1 _____
2. _____ - 1 _____
3. _____ - 1 _____
4. _____ - 1 _____
5. _____ - 1 _____
6. _____ - 11 _____
7. _____ - 13 _____
8. _____ - 12 _____
9. _____ - 13 _____
10. _____ - 13 _____
11. _____ - 13 _____
12. _____ - 13 _____
13. _____ - 5 _____
14. _____ - 3 _____
15. _____ - 6 _____
16. _____ - 1 _____

17. _____ - 1 _____
18. _____ - 2 _____
19. _____ - 2 _____
20. _____ - 1 _____
21. _____ - 1 _____
22. _____ - 1 _____
23. _____ - 1 _____
24. _____ - 1 _____
25. _____ - 1 _____
26. _____ - 1 _____
27. _____ - 1 _____
28. _____ - 1 _____
29. _____ - 1 _____
30. _____ - 2 _____
31. _____

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE May 10, 1988
COMPLETED BY S. Samolinski
TELEPHONE (815)357-6761

REPORT MONTH APRIL, 1988

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
4	4/1/88		S	719.0	C	4	Continuing Unit One second refuel outage

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no Safety Relief Valves actuated for Unit One during the reporting period April 1 through April 30, 1988.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
0-86-88	0 D/G Cooling Water Pump	Breaker inspection
0-87-88	0 D/G Cooling Water Pump	LMS-DG-01 surveillance, replace oil filter
0-89-88	0 D/G	Rebuild stab solenoids.
0-90-88	0 D/G	Check Cyl. temp. indication and speed indication
0-91-88	0 D/G Space Heater	Replace control and 74 relays
1-617-88	1A D/G	LMS-DG-01
0-618-88	1A D/G	Lubrication
1-619-88	1A D/G	LES-GM-129
1-620-88	1A D/G	BOP Calibrations
1-621-88	1A D/G	1DG034 Cooler relief
1-622-88	1A D/G	LES-DG-101, LES-GM-109
1-627-88	Bus 143, 1B D/G Feed	Modification M-1-1-84-019
1-628-88	1B D/G	Lubrication
1-652-88	1B D/G	Replace relay
1-712-88	1A D/G	Replace governor
1-713-88	1A D/G	Replace pyrometer probes
1-714-88	1B D/G	Repair relay terminations
1-719-88	1A D/G	Valve adjustments
1-740-88	1A D/G Heat exchanger	Clean heat exchanger
1-743-88	1A D/G Fuel transfer pump	Remove manway
1-744-88	1DO004	Upgrade valve internals
1-746-88	1A D/G	Replace 1DG-K39 relay

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-747-88	1A D/G	Relay modification
1-762-88	1B D/G	Repair oil leaks
1-890-88	1A D/G	Change air intake filters
1-893-88	1A D/G	Repair governor
1-913-88	1DG035	Repack valve
1-986-88	1B D/G	Prevent autostart

3. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculation Manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to Radioactive Waste treatment systems during this reporting period.

5. Indications of Failed Fuel Elements

There were no indications of failed fuel elements during this reporting period. The reactor is defueled.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

APRIL, 1988

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Steven J. Samolinski, telephone number (815)357-6761 extension 705.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

APRIL 1-30

April 1, 0000 hours	Unit Two entered April with the Reactor critical and the generator on-line at 1075 MWe.
April 6, 0130 hours	Load drop to 615 MWe for maintenance on A TDRFP.
April 12, 2030 hours	Begin ramping to 1067 MWe.
April 15, 0001 hours	Load drop to 627 MWe for control rod adjustment.
April 15, 0300 hours	Ramping to 1085 MWe.
April 30, 2200 hours	Begin load drop to 632 MWe for control rod adjustment.
April 30, 2400 hours	Unit Two completed April with the Reactor Critical and the Generator on line at 774 MWe. Continuing load drop to 632 MWe.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility license or Technical Specification.

There were no amendments to the Facility License or Technical Specifications during this reporting period.

2. Changes to procedures which are described in the Safety Analysis Report.

There were no changes to procedures described in the Safety Analysis Report during this reporting period.

3. Tests and Experiments not described in the Safety Analysis Report.

There were no Tests or Experiments conducted which are not described in the Safety Analysis Report.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of Safety-Related Maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

5. Completed Safety Related Modifications.

The following table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT*	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L79120	LPCS pump cooler upstream stop valve 2DG035	Faulty packing	Valve would not move	Repacked valve
L79397	IRM "G" 2C51-K601G	Faulty connection and dirty range switch contacts	No noise reading on channel 1	Cleaned contacts and replaced range switch
L79434	"C" MSL outboard isolation 2B21-F028C	Actuating bar out of adjustment on limit switches	Limit switch would not close	Adjusted actuating bars and replaced cover gaskets

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER: A brief Synopsis of Incorporated Modification Objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

UNIT TWO

There were no safety related modifications completed on Unit Two during the reporting period of April 1 through April 30, 1988.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, April 1, through April 30, 1988. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
88-005-00	4/12/88	HPCS Minimum Flow Bypass Pressure Switch Out of Tolerance.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE May 10, 1988
 COMPLETED BY Steven J. Samolinski
 TELEPHONE (815)357-6761

OPERATING STATUS

- 1. REPORTING PERIOD: April, 1988 GROSS HOURS IN REPORTING PERIOD: 719
- 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
- 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None
- 4. REASONS FOR RESTRICTION (IF ANY): None

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBLR OF HOURS REACTOR WAS CRITICAL	<u>719.0</u>	<u>2701.5</u>	<u>19486.3</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>29.83</u>
7. HOURS GENERATOR ON LINE	<u>719.0</u>	<u>2684.9</u>	<u>19156.0</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>2073168</u>	<u>7913808</u>	<u>55708864</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>694986</u>	<u>2648921</u>	<u>18457132</u>
11. NET ELEC. ENJRGY GENERATED (MWH)	<u>668945</u>	<u>2552307</u>	<u>17634920</u>
12. REACTOR SERVICE FACTOR	<u>100.0%</u>	<u>93.1%</u>	<u>62.9%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100.0%</u>	<u>93.1%</u>	<u>63.0%</u>
14. UNIT SERVICE FACTOR	<u>100.0%</u>	<u>92.5%</u>	<u>61.8%</u>
15. UNIT AVAILABILITY FACTOR	<u>100.0%</u>	<u>92.5%</u>	<u>61.8%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>89.8%</u>	<u>84.9%</u>	<u>54.9%</u>
17. UNIT CAPACITY FACTOR(USING DESIGN MWe)	<u>86.3%</u>	<u>81.6%</u>	<u>52.8%</u>
18. UNIT FORCED OUTAGE RATE	<u>0.0%</u>	<u>7.5%</u>	<u>19.0%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	Unit Two is scheduled to shutdown for second refuel outage October 12, 1988.		
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP.	N/A		

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: May 10, 1988
COMPLETED BY: Steven J. Samolinski
TELEPHONE: (815) 357-6761
MONTH: APRIL, 1988

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	1048	17.	1066
2.	1042	18.	1057
3.	977	19.	1050
4.	1016	20.	1047
5.	1014	21.	1045
6.	634	22.	1041
7.	581	23.	1036
8.	577	24.	1030
9.	667	25.	1021
10.	693	26.	1024
11.	715	27.	1022
12.	570	28.	1017
13.	925	29.	1013
14.	993	30.	998
15.	891	31.	
16.	1053		

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
 UNIT NAME LaSalle Two
 DATE May 10, 1988
 COMPLETED BY S. Samolinski
 TELEPHONE (815)357-6761

REPORT MONTH APRIL, 1988

NO.	DATE	TYPF	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
8	4/6/88	S	0.0	H	5	"A" TDRFP was brought off line to replace coupling and correct alignment.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES</u> <u>ACTUATED</u>	<u>NO & TYPE</u> <u>ACTUATIONS</u>	<u>PLANT</u> <u>CONDITION</u>	<u>DESCRIPTION</u> <u>OF EVENT</u>
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There were no Safety Relief Valves actuated on Unit Two during the reporting period April 1 through April 30, 1988.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-261-88	2E22-F012	Administrative Control; SOR removal.
2-273-88	2A D/G	Alignment check of AC circ. oil pump.
2-277-88	2DG035	Inspect torque switch.

3. Off-Site Dose Calculation Manual

There were no changes to the Off Site Dose Calculation manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to Radioactive Waste Treatment Systems during this reporting period.

5. Indications of Failed Fuel Elements.

Off Gas levels indicate that there is one pinhole fuel element failure in the reactor core. This does not represent a change from the previous reporting period.

There were no further indications of failed fuel elements during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

May 10, 1988

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period April 1, 1988 through April 30, 1988.

Very truly yours,

WRo
for C. J. Diederich
Station Manager
LaSalle County Station

GJD/SJS/jdp

Enclosure

cc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
P. Shemanski, NRR Project Manager
D. P. Galle, CECO
D. L. Farrar, CECO
INFO Records Center
L. J. Anastasia, PIP Coordinator SNED
M. A. Ortin, GE Resident
H. E. Bliss, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle
Dennis Carlson/Tech Staff
Terry Novotney/INPO Coordinator, Tech Staff
Central File

IE24
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