



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 11, 1988

Docket Nos. 50-327/326

Mr. S. A. White
Manager of Nuclear Power
Tennessee Valley Authority
6N 38A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Mr. White:

SUBJECT: REQUEST FOR RELIEF FROM ASME BOILER AND PRESSURE VESSEL CODE,
SECTION XI, INSERVICE INSPECTION PROGRAM

The staff has reviewed Tennessee Valley Authority's (TVA) request for relief from certain American Society for Mechanical Engineers (ASME) Boiler and Pressure Code, Section XI inservice inspection requirements for Essential Raw Cooling Water (ERCW) piping submitted by two letters both dated April 4, 1988 and by a clarification letter dated May 4, 1988. This relief was initiated because leaking occurred in the ERCW piping in Sequoyah Units 1 and 2, caused by microbiologically induced corrosion (MIC).

The staff finds that the relief request is acceptable provided the program includes the following additional items:

(1) The frequency of direct visual inspection of weld specific leaks is increased to monthly, and performed as scheduled, (2) at the same time leakage is detected, an assessment is made to ensure the acceptance criterion is not exceeded prior to the scheduled outage or repair, (3) the leakage from a leaking weld does not exceed 0.5 gpm and the total leakage from all welds does not exceed 2 gpm. The licensee will establish a threshold below which the leakage is considered immeasurable for the purpose of calculating the combined leakage.

During a May 6, 1988 telephone conference call between R. Hermann of NRC and TVA management, TVA committed to incorporate the three items specified above into their program by June 1, 1988.

Acceptance of this relief request is conditional upon these three additional items being included in the MIC Program. The basis for our conclusions is provided in the enclosed staff's Safety Evaluation.

The staff concludes that the Code repair requirements are impractical to perform at Sequoyah and that pursuant to 10 CFR 50.55a(a)(3)(i), the proposed alternatives to the described requirements will provide an acceptable level of quality and safety. The staff also concludes that granting relief where the Code requirements are impractical is authorized by law and will not endanger life or property, or the common defense and security. The relief is in the public interest giving due consideration to the burden that could result

upon TVA if the specific Code requirements were imposed upon Sequoyah Nuclear Plant, Units 1 and 2. Accordingly, pursuant to 10 CFR 50.55a(g)(6)(i), TVA is granted the requested relief, on the conditions set forth above.

Sincerely,

Robert A. Hermann, Acting Assistant Director
for Projects
TVA Projects Division
Office of Special Projects

Enclosure:
Safety Evaluation

cc w/enclosure:
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