NORTHEAST UTILITIES

THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Selden Street, Berlin, Connecticut

P.O. BOX 270
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May 16, 1988
MP-11817

Re: 10CFR50.73(a)(2)(i)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49
Docket No. 50-423
Licensee Event Report 88-015-00

Gentlemen:

This letter forwards Licensee Event Report 88-015-00 required to be submitted within thirty days pursuant to 10CFR50.73(a)(2)(i), any operation or condition prohibited by the Plant's Technical Specifications.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Stephen E. Scace
Station Superintendent
Millstone Nuclear Power Station

SES/NDH:mo

Attachment: LER 88-015-00

cc: W. T. Russell, Region I
W. J. Raymond, Senior Resident Inspector

IE22
1/1

NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	1 OF 0 3

TITLE (4)

Reactor Coolant System Unidentified Leakage Action Statement Improperly Terminated

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 4	1 5	8 8	8 8	— 0 1 5	— 0 0	0 5	1 6	8 8		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Nelson D. Hulme, Senior Engineer X5398	2 0 3 4 4 7 - 1 7 9 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 15, 1988 at 1500 hours in Mode 3 (Hot Standby) 557 degrees and 2250 psia, it was determined that the action taken to terminate a previously documented Unidentified Leakage Event had been improperly performed.

At 1715 hours on April 14, the action statement for an unidentified Reactor Coolant System (RCS) leakrate of greater than 1 gallon per minute (gpm) was entered based on the increased leakrate observed by Volume Control Tank make up. At 2107 hours on April 14, after a leak source was identified and isolated, the action statement was exited based on qualitative assessment of plant parameters being steady. Following the resolution of computer problems and the completion of RCS sampling, a leakage surveillance program was initiated at 0900 hours on April 15. At 1332 hours, the leakrate was determined to be 1.15 gpm. Root cause of this event was personnel error. The decision was made to terminate the Unidentified Leakage Action Statement without a detailed quantitative analysis to determine the unidentified leakrate.

All Operations Department personnel have been sent a memorandum stipulating that approved surveillances or other quantitative methods, must be used to determine whenever RCS unidentified leakage has been corrected. The applicable operating procedure has been changed to provide better guidance on measuring unidentified RCS leakage.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. Description of Event

On April 15, 1988 at 1500 hours with the plant in Mode 3 (Hot Standby) at 0% power, 557 degrees Fahrenheit and 2250 psia, it was determined that the action taken to terminate a previously documented Unidentified Leakage Event had been improperly performed.

At 1715 hours on April 14, 1988, the Action Statement for an Unidentified Reactor Coolant System (RCS) leakrate of greater than 1 gallon per minute (gpm) was entered based on the increased leakrate observed by Volume Control Tank make up. At 2006 hours on the same day, stem leakoff was discovered at a pressurizer relief block valve. After the leak was isolated, Volume Control Tank (VCT) and Pressurizer levels were monitored for further changes. At 2107 hours, based only on qualitative assessment of steady VCT and Pressurizer levels, the Action Statement for RCS Unidentified Leakage was exited. A leakrate surveillance program was initiated at 2330 hours, but due to computer problems unreliable results were obtained. Sampling of the RCS by the Chemistry Department also introduced errors in the calculations. At approximately 0900 hours on April 15, 1988, a leakrate surveillance program was reinitiated. At 1332 hours it was evident that an unidentified leakrate of greater than 1 gpm still existed. Therefore, an Unidentified Leakage Event was again declared.

II. Cause of Event

The root cause of this event was personnel error. The decision was made to terminate the Unidentified Leakage Action Statement without performing the leakrate surveillance program, or without performing some detailed quantitative analysis to precisely determine the unidentified RCS leakrate.

III. Analysis of Event

This Licensee Event Report (LER) is being submitted in compliance with 10CFR50.73(a)(2)(i) due to operating in a condition prohibited by the plant's Technical Specifications.

There were no safety consequences as a result of this event. The plant was in Hot Standby at 1715 hours on April 14 when the leakage was first discovered and the plant was eventually placed in Cold Shutdown, as required by the Technical Specifications, after an unidentified leakrate of 1.15 gpm was determined at 1332 hours on April 15.

IV. Corrective Action

All Operations Department personnel have been sent an internal department memorandum describing this event. The memo stresses that approved surveillances or other quantitative methods must be used whenever RCS unidentified leakage has been corrected. The applicable Operating procedure has been changed to provide better guidance on measuring unidentified RCS leakage.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

V. Additional Information

There have been no previous events where an Action Statement was improperly terminated.

EIIS Codes

System
Reactor Coolant System - AB