

LICENSEE: System Energy Resources, Inc.

FACILITY: Grand Gulf Nuclear Station, Unit 1

SUBJECT: SUMMARY OF AUGUST 15, 1988 MEETING REGARDING POTENTIAL CHANGES TO
THE TECHNICAL SPECIFICATIONS

The purpose of the meeting was to discuss potential changes to the Technical Specifications (TS). Other licensing actions were also discussed including the status of completion of conditions in the operating license and actions taken regarding problems identified in information notices and commitments made in previous licensing actions. Enclosure 1 is a list of participants. Enclosure 2 is an agenda proposed by the licensee.

The following comments were made regarding the agenda items in Enclosure 2.

- I.A.2 The present specification (4.9.6.1.b.1) requires the downtravel cutoff for the refueling main hoist to be 3.5 ± 0.5 inches below the top of fuel assembly handles in the core. With the lengthened fuel assemblies due to irradiation, the downtravel cutoff point may be too high to grapple control rod guide handles. Difficulty was experienced in this regard during the second refueling; and it is expected to be more difficult, if not impossible, to meet the TS during the next refueling outage. The proposed specification would be for downtravel cutoff to be greater than or equal to one inch above the reactor vessel top guide, a fixed referenced point. The basis for the specification limiting downtravel is to prevent the grapple from engaging the top guide instead of a fuel assembly handle. The licensee believes this change would only be applicable to BWR-6's.
- I.A.3 The addition of a footnote taking exception to local leak rate tests (LLRTs) for containment isolation valves in test, vent, and drain lines in TS Table 3.6.4-1 would reduce the potential for LLRTs to be on the outage critical path. Administrative Controls would be used to assure that valves are left in the closed position after maintenance. The BWR Owners Group is recommending this change.
- I.A.4 The licensee commented that some modes of spent fuel cooling during outages may be affected by having the spent fuel pool gate in place. However, having this gate and the upper containment pool gate in place would allow work to be done on the horizontal fuel transfer mechanism, if needed. The licensee will consider effects on spent fuel cooling in its safety analysis of the proposed TS change.

- I.A.5 The licensee raised the question again whether the model Bases for TS 4.0.3 were correct in Generic Letter 87-09. In the June 14 and 15, 1988 meeting (Summary dated 8/8/88), the licensee had questioned whether a missed surveillance test must be reported as a TS violation. Staff in the Technical Specifications Branch reaffirmed that a missed surveillance test should be reported as a TS violation, whether or not the allowed outage time of the action statement has been exceeded at the time of discovery of the missed surveillance test.
- I.B.2 Regarding the new alternate decay heat removal system to be installed in the third refueling outage, two new TS will be proposed. R. Radiation monitors will be added to the plant service water (PSW) system because PSW will be used in the new heat exchanger. Thermal overload devices for the motors operating isolation valves in the new system will be added to TS. The licensee will perform a 10 CFR 50.59 review of the system to determine whether to submit the system design criteria to NRC for approval prior to installation. The staff requested that a description of the system and safety analysis be provided with the proposed TS changes.
- III.A The staff indicated that the licensee's 8/3/88 proposed implementation of the 10 CFR Part 55 rule was adequate and that the revised application for TS change would be renoticed.
- III.F The licensee stated that a TS change would be submitted in August to remove unnecessary organization position titles from TS Section 6.0.

The staff requested that information be provided in the next meeting regarding the following:

- Disposition of a July 6, 1987 proposed TS change to delete the requirement for isolation valve instrumentation to be operable during shutdown. The staff had indicated in previous meetings that the proposed change is not acceptable.
- An update of J. G. Cesare's memo dated 10/22/86 which gives the status of completion of Operating License conditions.
- A description of procedures implementing the use of MSA GMR-1 canisters. In its Exemption to 10 CFR Part 20 permitting their use, the staff required procedures to be in place within a certain time.
- A description of how assurance has been obtained that gaps in neutron absorber material in high density fuel racks are not a problem at Grand Gulf Nuclear Station, Unit 1, as they were at another facility that had installed Joseph Oats fuel racks. (IN 87-43)

PARTICIPANTS AT AUGUST 15, 1988 MEETING

REGARDING TS CANGES

| | |
|------------------|--------------------------------------|
| J.G. Cesare | SERI, Director of Licensing |
| J. Fowler | SERI, Licensing |
| J.K. Fortenberry | SERI, Licensing |
| L. Kintner | NRC, Project Directorate, II-1 |
| R. Emch* | NRC, Section Head, TSB |
| T. Dunning* | NRC, Technical Specifications Branch |

*Part-time

TECH SPEC MEETING
August 15, 1988

I. RFO3 LICENSING ACTIONS FOR DISCUSSION

REFERENCE

A. TECH SPEC CHANGES

- ~~1. Fuel Handling Area/Fuel Pool Sweep
Ventilation Exhaust Radiation Monitors~~ T.S. Table 3.3.7.1-1
2. Refueling Platform Main Hoist Down
Travel Cutoff T.S. 4.9.6.1 b.1
3. LLRT Requirements for Containment
Penetration Isolation Test, Vent &
Drain Valves T.S. Table 3.6.4-1
4. Upper Containment Pool Fuel Transfer
Canal Gate T.S. 3.5.2/3.5.3
Footnote *
5. Generic Letter 87-09

B. OTHER RFO3 LICENSING ITEMS

1. Exception to R.G. 1.108 to Allow Only
One LOP/LOCA Test for D/Gs I, II & III R.G. 1.108 Section C
2. Alternate Decay Heat Removal System

II. NON OUTAGE TECH SPEC CHANGES

- A. DELETION OF T.S. CHAPTER 6 REFERENCES
TO ORGANIZATION POSITIONS GL 88-06
- B. COMBINED SRO/STA POSITION GL 86-04
- C. RWCU DELTA-FLOW TIMER BYPASS

III. OTHER ITEMS FOR STATUS

REFERENCE

| | | |
|----|---|-------------------------|
| A. | 10CFR55 RULE | AECM-88/0146 8/3/88 |
| B. | DELETION OF FIRE ZONE OC506 FROM T.S. Table 3.3.7.9-1 | AECM-88/0114 7/12/88 |
| C. | D/G TEST SCHEDULE (Amendment issued) | AECM-88/0049 4/8/88 |
| D. | DELETION OF LPSP/HPSP DAILY FUNCTIONALS | AECM-88/0105 6/21/88 |
| E. | SCRAM ACCUMULATOR ACTION STATEMENT | AECM-88/0071 4/22/88 |
| F. | DELETION OF ORGANIZATION CHARTS (Amendment issued) | AECM-88/0106 6/3/88 |
| G. | RPS INSTRUMENTATION SURVEILLANCE INTERVALS/ ALLOWED OUT-OF-SERVICE TIMES | AECM-88/0118 6/30/88 |
| H. | DELETION OF THE SNUBBER TEST REJECT REGION | AECM-88/0149 7/25/88 |
| I. | SEPARATION OF THE 24 HR D/G TEST FROM THE LCP/LOCA TEST | AECM-88/0150 7/26/88 |
| J. | WIDE RANGE NEUTRON MONITORING | AECM-88/0110 7/1/88 |

DISTRIBUTION FOR MEETING SUMMARY DATED: September 14, 1988

Facility: GRAND GULF UNIT 1

Docket File

NRC PDR

Local PDR

PDI-1 Reading

E. Adensam

P. Anderson

Project Manager

OGC

E. Jordan (MNBB 3302)

B. Grimes (9A2)

NRC Participants

ACRS (10)

B. Trokoski (17D19)

cc: Licensee/Applicant Service List

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- Whether responses to Generic Letter 83-28 dated 3/30/88 and 10/16/87 should be applicable for Unit 2, as well as Unit 1, since all other responses and SER's for GL 83-28 are for both units.
- A description of how assurance has been obtained that springs in main steam isolation valves are not broken, as they were in another facility using the same design valves (IN 86-81).
- A description of plans for providing additional heat removal capability, or justifying present capability, for the high density spent fuel pool when full capacity is authorized. Present TS limit the number of fuel assemblies which can be stored in the spent fuel. A commitment was made to provide this analysis by the third refueling outage.

The next meeting was tentatively scheduled for the last week in August. The agenda will include discussions of previously submitted applications on which reviewers have questions.

Lester L. Kintner, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects I/II

cc: See next page

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|------|--------------|------------------------|---|---|---|---|---|---|---|
| OFC | : PM: PV | : DRPR: D: PO: I: DRPR | : | : | : | : | : | : | : |
| NAME | : L. Kintner | : jw: E. Adensam | : | : | : | : | : | : | : |
| DATE | : 8/13/88 | : 8/14/88 | : | : | : | : | : | : | : |

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