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S. J. KOWALSKI
VICE-PRESIDENT
NUCLEAR ENGINEERING

May 13, 1988

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

SUBJECT: Interim Report for Limerick Generating Station, Unit 2
Aluminum Vertical Bus Failures in Cutler Hammer MCCs
Limerick Generating Station, Unit 2
NRC Construction Permit No. CPPR-107

REFERENCE: Telecon of PECO to NRC dated 4/15/88

FILE: QUAL 2-10-2 (SDR #230-2)

Dear Sir:

In compliance with 10CFR Part 50.55(e), we are hereby submitting an interim report concerning the use of Aluminum Vertical Bus in Cutler-Hammer MCCs at Limerick Generating Station Unit 2. The Philadelphia Electric Company (PECO) discussed this condition by telephone with the NRC Regional Office of Inspection and Enforcement on April 15, 1988, after it was determined that this may be a reportable condition.

On February 23, 1988 the Nuclear Engineering Division of Philadelphia Electric Company (PECO) was advised of a potential problem which may exist in the motor control centers at Limerick Generating Station Unit 2. A failed aluminum vertical bus was discovered in the waste water treatment plant motor control center at our Eddystone Generating Station. After examination of the bus section by PECO and the manufacturer (Cutler-Hammer) the problem is believed to have been caused by heat build-up which, over a period of time, deteriorates the connection point between the copper stab and the aluminum vertical bus. The manufacturer has indicated that the potential for failure only exists for aluminum vertical bus sections feeding NEMA size 4 motor starters. Limerick Generating Station Unit 2 also utilizes motor control centers with aluminum vertical bus sections and size 4 starters, manufactured by Cutler-Hammer, in safety-related applications. Therefore, due to the potential for similar failures to occur at Limerick Generating Station Unit 2 we have concluded that this is a reportable condition.

We are currently evaluating this condition and are in contact with the manufacturer to determine the extent of the condition, the root cause and the corrective action required.

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A detailed report will be filed after our evaluation is completed and corrective action determined. It is expected that the evaluation and detailed report will be completed by July 15, 1988.

Sincerely,

*J. S. Kemp
for W. Kowalski*

DSM/ghr/05118801

Copy to: USNRC Site Resident Inspector
United States Nuclear Regulatory Commission
Region I
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