

MAY 12 1988

Docket No. 50-334

Duquesne Light Company
ATTN: Mr. J. D. Sieber
Vice President
Nuclear Group
Post Office Box 4
Shippingport, Pennsylvania 15077

Gentlemen:

Subject: Inspection No. 50-334/88-08

This refers to the special operational safety inspection conducted by a team of NRC personnel led by Dr. P. K. Eapen of this office on March 21-April 1, 1988 at the Beaver Valley Power Station Unit No. 1, Shippingport, Pennsylvania of activities authorized by NRC License No. DPR-66 and to the discussion of our findings held by Dr. Eapen with you and members of your staff at the conclusion of the inspection.

Areas examined during this inspection included systems and plant personnel actions required to mitigate the consequences of several accident sequences which have the potential to result in core melt. The systems and accident sequences were selected from generic probabilistic risk studies for pressurized water reactors and were modified to reflect specific features at Beaver Valley Power Station Unit No. 1. The systems selected for this inspection were the on site and off site electrical power system, emergency core cooling system, and the auxiliary feedwater system. The inspection focused on the availability of critical components in the above systems as evidenced by plant conditions, operating experience, and the implementation of surveillance and maintenance programs. In addition, this inspection assessed the ability of the licensee staff to use procedures and equipment to respond to and recover from certain postulated accidents. The assessments were made by conducting procedure walkthroughs with licensee personnel in the facility and by observing the performance of qualified licensee personnel during selected scenarios on the licensee plant specific simulator.

Within these areas, our inspectors examined selected procedures and representatives records, conducted interviews with personnel, inspected plant equipment and observed personnel actions during simulated accident scenarios. These areas are described in the NRC Region I inspection Report which is enclosed with this letter.

Overall, the results of this inspection indicate that the plant equipment and licensee staff are capable of reliable response to loss of onsite and offsite power events and to loss of coolant accidents. This conclusion assumes that the existing management initiatives and maintenance efforts to reduce equipment down time will continue in the future. The inspection report summarizes other

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cc w/encls:

- J. J. Carey, Executive Vice President, Operations
- J. O. Crockett, General Manager, Corporate Nuclear Services
- W. S. Lacey, General Manager, Nuclear Operations
- N. R. Tonet, Manager, Nuclear Engineering
- T. P. Noonan, Plant Manager
- C. E. Ewing, QA Manager
- K. D. Grada, Manager, Nuclear Safety
- Public Document Room (PDR)
- Local Public Document Room (LPDR)
- Nuclear Safety Information Center (NSIC)
- NRC Resident Inspector
- Commonwealth of Pennsylvania

bcc w/encls:

- Region I Docket Room (with concurrences)
- Management Assistant, DRMA (w/o encl)
- L. Tripp, DRP
- D. Limroth, DRP
- P. Tam, LPM, NRR
- R. Bores, DRSS
- PAO (10) SALP Reports Only

DRS:RI *asc*
LCheung/gcb/pj/ms

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NRR:HQ
RKarsch

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NDudley

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KMurphy

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WJohnston

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positive attributes such as senior management initiatives to maintain high reliability for plant equipment, and highly capable operations, maintenance and engineering personnel.

Based on the results of this inspection, it appears that certain of your activities were not conducted in full compliance with the NRC requirements, as set forth in the notice of violation, enclosed herewith as Appendix A. This violation has been categorized by Severity Level in accordance with the General Statement of Policy and Procedures for NRC Enforcement Actions, 10 CFR Part 2, Appendix C (Enforcement Policy 1987). You are required to respond to this letter and in preparing your response, you should follow the instructions in Appendix A.

In addition, the team identified three unresolved items requiring action. Two of the unresolved items discuss an apparent lack of documented basis for 1) maintaining the containment pressure between 8.9 psia and 14 psia during LOCA accidents, without assuring that the low head safety injection pumps will have sufficient net positive suction head (NPSH) at 8.9 psia, and 2) not incorporating several of the vendor recommendations in the plant procedures for the turbine driven auxiliary feedwater pump. The third unresolved item discusses the inability of the existing vibration monitoring program for pumps to provide consistent readings and to detect possible degradation between tests. Please provide details of the actions you plan to take to resolve these items with completion dates in your response to this letter.

The responses directed by this letter and the accompanying notice are not subject to the clearance procedures of the office of management and budget as required by the Paper Work Reduction Act of 1980, PL 96-511.

Your cooperation with us in this matter is appreciated.

Sincerely,

Original Signed By:

William V. Johnston
Acting Director
Division of Reactor Safety

Enclosures:

1. Appendix A, Notice of Violation
2. Inspection Report No. 50-334/88-08

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