AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE May 13, 1988
COMPLETED BY W. J. Blessie
TELEPHONE 402-536-4595

MONTH April 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	331.0	17	486.7
2	330.4	18	485.7
3	316.3	19	484.9
4	390.6	20	485.1
5	450.0	21	484.8
6	470.4	22	485.2
7	481.7	23	486.8
8	487.9	24	487.8
9	487.5	25	486.2
10	487.2	26	485.8
11	488.0	27	486.1
12	487.6	28	487.2
13	486.7	29	476.4
14	486.2	30	461.5
15	486.5	31	
16	487.0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE May 13, 1988
COMPLETED BY W. J. Blessie
TELEPHONE 402-536-4595

OPERATING STATUS

1. Unit Name: Fort Calhoun Station — Notes
2. Reporting Period: May 1988
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If changes occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   N/A
9. Power Level to Which Restricted, If Any (Net MWe): N/A
10. Reasons for Restrictions, If Any:

		This Month	Yr-to-Date	Cumulative
11.	Hours in Reporting Period	719.0	2903.0	127,969.0
12.	Number of Hours Reactor Was Critical	719.0	2903.0	99,742.8
13.	Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14.	Hours Generator On-Line	719.0	2903.0	98,821.8
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	1,030,843.3	3,750,856.0	128,298,481.2
17.	Gross Electrical Energy Generated (MWH)	350,196.0	1,275,038.0	42,247,959.2
18.	Net Electrical Energy Generated (MWH)	334,202.9	1,210,776.7	40,354,859.5
19.	Unit Service Factor	100.0	100.0	77.2
20.	Unit Availability Factor	100.0	100.0	77.2
21	Factor (Using MDC Net)	97.2	87.3	68.4
22.	Unit Capacity Factor (Using DER Net)	97.2	87.3	66.6
23.	Unit Forced Outage Rate	0.0	0.0	3.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    The 1988 Refueling Shutdown is tentatively scheduled for September 2, 1988 with startup tentatively scheduled for November 18, 1988.
25. If Shut Down at End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation:

		Forcast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April 1988

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE May 13, 1988
COMPLETED BY W. J. Blessie
TELEPHONE (402) 536-4595

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
88-01	880212	S	0	H	4	N/A	BK	FAN	Fort Calhoun Station was at 70% power until starting a power ascension on April 4, 1988. Power was maintained at 100% from April 8 to April 29 when it was reduced to 95% in preparation for moderator temperature coefficient (MTC) testing

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending April 1988.

1. Scheduled date for next refueling shutdown. — September 1988

2. Scheduled date for restart following refueling. — December 1988

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

      Incorporate cycle specific requirements resulting from reload safety analysis.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. —

   c. If no such review has taken place, when is it scheduled? —

4. Scheduled date(s) for submitting proposed licensing action and support information. — July 1988

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:

a) in the core	133	assemblies
b) in the spent fuel pool	393	"
c) spent fuel pool storage capacity	729	"
d) planned spent fuel pool storage capacity	May be increased via fuel pin consolidation	"

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1994*

*Full core offload of 133 assemblies lost.

Prepared by R L Jaworski — Date April 25, 1988

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

April 1988
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station was at 70% power to conserve reactivity for end of cycle until starting a power ascension on April 4, 1988. Power was maintained at 100% from April 8 to April 29 when it was reduced to 95% in preparation for moderator temperature coefficient (MTC) testing.

An NRC inspection on licensed operator initial and requalification training was conducted during April 1988. All NRC reactor operator license annual requalification exams have been completed. Annual simulator training at Combustion Engineering in Windsor, Connecticut, started in April 1988.

An INPO plant evaluation was conducted during April 1988.

Construction continues on the new warehouse, maintenance shop, and training facility.

No safety valves or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-CONT-3	Verification of Containment Penetration Configurations. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59. This one time special procedure was performed to ensure containment integrity. The procedure consisted of a walkdown of all containment penetrations with a follow-up independent verification. The results were acceptable.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

Procedure	Description
SP-STROKE-1	In Service Testing of Air Operated, CQE Valves. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it only allowed stroke testing to be conducted on 39 air operated CQE valves. The objective of the testing is to determine if valve operability was degraded (or is degrading) due to the intrusion of water into the instrument air system. This testing did not in any way compromise plant safety, but enhanced it by ensuring operability of safety related valves.

System Acceptance Committee Packages for April 1988:

Package	Description/Analysis
EEAR FC-86-020A	Warehouse Relocation - Earthwork. This modification provided for earthwork preparation prior to construction of the new warehouse. This modification does not have an adverse effect on the safety analysis.
EEAR FC-86-036	Shielding Above Room 11. This modification provided for radiation protection for the H.P. counting room. This modification does not have an adverse effect on the safety analysis.
EEAR FC-86-100	Auxiliary Building Roof Access Ladder. This modification provided for the addition of a platform in the fan room and modification of ladders on the auxiliary building roof. This modification does not have an adverse effect on the safety analysis.
EEAR FC-87-033B	Security Fence Rearrangement - Maintenance Building. This modification provided for temporary relocation of security barriers to accommodate facility upgrades. This modification does not have an adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

During April, the leak rate test on the containment purge valves (penetrations M-87 and M-88) was completed per ST-CONT-3, F.4. The "as found" and "as left" leak rates for penetrations M-87 and M-88 were 0 sccm.

In January, the "B" and "C" "as left" leak rate was 374.2 sccm. Since both sets of containment purge valves tested zero leakage during this April test, the total leakage remains unchanged. This leak rate is well below the allowed leakage of 0.6 La as specified in 10 CFR 50 Appendix J.

F. CHANGES IN PLANT OPERATING STAFF

None

G. TRAINING

During April, the National Academy for Nuclear Training accredited the radiation protection, instrument and control, electrical, mechanical, chemistry, shift technical advisor, and technical staff training programs. OPPD is now a full member of the National Academy.

Remedial training and successful examination of two senior reactor operators who failed the NRC-administered walkthrough examination was completed. An EONT training class began. Simulator training at CE for the licensed operator requalification program was administered. Initial training continued for the reactor operator and senior reactor operator training programs. Quarterly seminars for technical staff and managers continuing training were conducted. Examination scenarios for the NRC-administered simulator examinations to be administered at the end of May were forwarded to Region IV personnel for their review and approval.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
112	The amendment revises the Technical Specifications to permit an extension to the next due date from April 30, 1988 to the refueling outage scheduled for September 1988 for performing the inspection of Diesel Generator No. 1 required by Surveillance Requirements.

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates
Manager-Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102-2247
402/536-4000

May 13, 1988
LIC-88-369

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: April Monthly Operating Report

Pursuant to Technical Specification Section 5.9.1, and 10 CFR Part 50.4(b)(1), please find enclosed, one copy of the April 1988 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

R L Jaworski for

R. L. Andrews
Division Manager
Nuclear Production

RLA/me

Enclosures

c: NRC Regional Office
Office of Management & Program Analysis (2)
R. M. Caruso - Combustion Engineering
R. J. Simon - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
American Nuclear Insurers
NRC File (FCS)

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