

8809160005 880831 PDR ADOCK 05000346 PNU

#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### TOLEDO EDISON COMPANY

AND

### THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

#### DOCKET NO. 50-346

#### DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendmen' No. 118 License No. NPF-3

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated December 7, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

(a) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 118, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

 This license amendment is effective as of its date of issuance and shall be implemented not later than October 17, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

utter Perk

Kenneth E. Perkins, Director Project Directorate III-3 Division of Reactor Projects - III, IV, V, & Special Projects

Attachment: Changes to the Technical Specifications

Date of Issuance: August 31, 1988

# ATTACHMENT TO LICENSE AMENDMENT NO. 118

## FACILITY OPERATING LICENSE NO. NPF-3

# DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

#### Remove

#### Insert

3/4 3-28

3/4 3-28

## TABLE 3.3-11 (Continued)

# TABLE NOTATION

- \* May be bypassed when steam pressure is below 650 psig. Bypass shall be automatically removed when the steam pressure exceeds 650 psig.
- # The provisions of Specification 3.0.4 are not applicable.

### ACTION STATEMENTS

- ACTION 13 With the number of OPERABLE Channels one less than the Total Number of Channels, startup and/or power operation may proceed until performance of the next required CHANNEL FUNCTIONAL TEST provided the inoperable section of the channel is placed in the tripped condition within 1 hour.
- ACTION 14 With the number of OPERABLE Channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### TABLE 3.3-12

# STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNITS		TAIP SETPOINTS	ALLOWABLE VALUES
1.	Steam Line Pressure - Low	≥ 591.6 psig	<pre>&gt; 591.6 psig* &gt; 586.6 psig**</pre>
2.	Steam Generator Level - Low <sup>(1)</sup>	≥ 16.4"	≥ 15.6"* ≥ 12.9"**
3.	Steam Generator Feedwater Differential Pressure - High (2)	≤ 197.6 psid	<pre>≤ 197.6 psid* ≤ 199.6 psid**</pre>
4.	Reactor Coolant Pumps - Loss of	High $\leq$ 1384.6 amps Low $\geq$ 106.5 amps	<pre>≤ 1384.6 amps# ≥ 106.5 amps#</pre>
(1)			

 (1) Actual water level above the lower steam generator tubesheet.
 (2) Where differential pressure is steam generator minus feedwater pressure. \*Allowable Value for CHANNEL FUNCTIONAL TEST \*\*Allowable Value for CHANNEL CALIBRATION

#Allowable Value for CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION