

July 19, 1971

Pacific Gas and Electric Company
245 Market Street
San Francisco, California 94106

Docket No. 050-0275
Docket No. 050-0323

Attention: Mr. Richard N. Peterson
Senior Vice President

Gentlemen:

This refers to the inspection conducted by Messrs. A. D. Johnson and R. T. Dodds of this office on June 23-25, 1971, of construction activities authorized by AEC Construction Permit Nos. CFFR-39 and CFFR-69, and to the discussion of our findings held by Mr. Johnson with Mr. J. D. Worthington, Mr. F. F. Mauts, and members of their staffs on June 30, 1971.

Areas examined during this inspection included the QA-QC programs as related to manufacture, testing, placement and curing of concrete to be used in the construction of Unit No. 2, and the QA-QC program specifically relating to Class I components to be installed in Unit No. 1. Additional areas examined included (1) the methods and procedures used for assuring that vendor certifications are consistent with descriptions to be provided in the Final Safety Analysis Report for both units, (2) the need for, and availability of, documented dimensional checks of wall thicknesses of significant sized valves to be installed within the pressure boundary of the primary coolant system, (3) the use of your Safety Report Analysis change procedure, and (4) the maintenance and control of informal records relating to construction activities which may reflect the quality characteristics of the structures, systems and components incorporated in the units. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with reactor facility personnel, and observations by the inspectors.

No items of nonconformance were identified within the scope of this inspection.

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It is our understanding, with respect to the following items, that you intend to take the actions indicated:

1. Review your QA system to assure that all changes made either by you or your nuclear steam supplier, which would significantly alter the description provided in the PSAR, will be reflected in the FSAR.
2. Collect all calibration procedures related to inspection and test equipment and incorporate these procedures into an appropriate manual form as required by your recently approved QA Procedure No. PRQ-5.
3. Conduct a survey of both official and unofficial logs and records at the site to determine the scope and content of all records being maintained.
4. Determine whether dimensional checks have been made of significant sized valves within the pressure boundary of the primary coolant system and, where documented results are unavailable, initiate a program to determine the adequacy of valve wall thicknesses.

These items will be examined during the next inspection.

No reply to this letter is necessary, but should you have any questions concerning this inspection, we will be glad to discuss them with you.

Sincerely,

R. W. Smith
Director

cc: J. D. Worthington
F. P. Nautz
G. V. Richards

bcc: J. B. Henderson, CO (5)
A. Giambusso, CO
R. H. Engelken, CO
L. Kornblith, Jr., CO