

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

AUGUST 1988

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Steven J. Samolinski, telephone number (815)357-6761, extension 705.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

August 1-31

August 1, 0000 hours	Unit One entered August with the Reactor critical and the Generator on-line at 900 MWe.
August 5, 2200 hours	Load drop to 638 MWe to collect RR data.
August 6, 0100 hours	Ramping to 943 MWe.
August 24, 2300 hours	Load drop to 644 MWe for rod set.
August 25, 0200 hours	Ramping to 1096 MWe.
August 29, 1800 hours	Load drop to 803 MWe for heater drain problems.
August 31, 2400 hours	Unit One is on line and holding at 1100 MWe.

B. PLANT OF PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility License or Technical Specification.

There were no amendments to the Facility License or Technical Specifications during the month of August.

2. Changes to procedures which are described in the Safety Analysis Report.

There were no changes to procedures which are described in the Safety Analysis Report.

3. Tests and Experiments not described in the Safety Analysis Report.

There were no tests or experiments conducted during August which are not described in the Safety Analysis Report.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a Summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

5. Completed Safety Related Modifications.

The following Table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L83334	ACB 1433 GE Magna- blast breaker	Breaker failure due to wear and improper maintenance	Breaker would not latch closed	Replaced breaker
L83216	1VR05YB Isolation damper	Asco solenoid failure	Damper would not close	Replaced solenoid
L75842	1VR04YA Isolation damper	Ascc solenoid failure	Damper would not close	Replaced solenoid
L83168	ODG01P O D/G cooling water pump	Breaker failure resulted in faulted motor windings	Unable to supply cooling water to OD/G, Div. I switchgear rooms, and LPCS pumps.	Breaker and motor replaced

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER: A brief synopsis of incorporated modification objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

UNIT ONE

- M-1-0-83-019 Installation and revision of floor drains and curbing in the electrode boiler area. The new curbing will contain the water which has collected on the floor in the boiler area.
- M-1-1-85-018 Removal of the existing dampers 1VP02CA/B and their associated operators and replace with standard galvanized, non safety-related dampers.
- M-1-1-87-084 Snubber reduction on the Main Steam piping subsystem per snubber reduction program.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, August 1 through August 31, 1988. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
88-017-00	8/21/88	"B" VC Ammonia Detector Actuation.
88-018-00	8/22/88	0 D/G Cooling Pump Motor Failure.
88-019-00	8/29/88	Failure of 1B D/G Breaker to Auto Close onto Bus within 13 seconds.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE Sept, 10, 1988
 COMPLETED BY S. J. Samolinski
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: August, 1988 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None
4. REASONS FOR RESTRICTION (IF ANY): None

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>744.0</u>	<u>3072.1</u>	<u>23046.6</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>1642.0</u>
7. HOURS GENERATOR ON LINE	<u>744.0</u>	<u>2890.3</u>	<u>22320.3</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWh)	<u>2277968</u>	<u>8287560</u>	<u>60121858</u>
10. GROSS ELEC. ENERGY GENERATED (MWh)	<u>736521</u>	<u>2252901</u>	<u>19669677</u>
11. NET ELEC. ENERGY GENERATED (MWh)	<u>703253</u>	<u>3067338</u>	<u>19165979</u>
12. REACTOR SERVICE FACTOR	<u>100.0%</u>	<u>51.3%</u>	<u>56.3%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100.0%</u>	<u>51.3%</u>	<u>60.3%</u>
14. UNIT SERVICE FACTOR	<u>100.0%</u>	<u>49.4%</u>	<u>54.5%</u>
15. UNIT AVAILABILITY FACTOR	<u>100.0%</u>	<u>49.4%</u>	<u>54.5%</u>
16. UNIT CAPACITY FACTOR (USING MLC)	<u>91.2%</u>	<u>50.6%</u>	<u>45.2%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>87.7%</u>	<u>48.6%</u>	<u>43.4%</u>
18. UNIT FORCED OUTAGE RATE	<u>0.0%</u>	<u>4.8%</u>	<u>13.1%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			

N/A

20. IF SHUT DOWN IS AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: Sept. 10, 1988
COMPLETED BY: S. J. Samolinski
TELEPHONE: (815) 357-6761
MONTH: AUGUST, 1988

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	950.6	_____	17. _____	939.2	_____
2. _____	974.0	_____	18. _____	964.2	_____
3. _____	947.6	_____	19. _____	989.8	_____
4. _____	963.4	_____	20. _____	910.1	_____
5. _____	982.3	_____	21. _____	898.8	_____
6. _____	825.1	_____	22. _____	952.5	_____
7. _____	819.0	_____	23. _____	925.7	_____
8. _____	913.1	_____	24. _____	1005.0	_____
9. _____	1026.6	_____	25. _____	889.9	_____
10. _____	990.0	_____	26. _____	1039.2	_____
11. _____	974.3	_____	27. _____	991.4	_____
12. _____	949.4	_____	28. _____	886.1	_____
13. _____	977.5	_____	29. _____	895.3	_____
14. _____	936.0	_____	30. _____	908.7	_____
15. _____	999.5	_____	31. _____	939.5	_____
16. _____	951.3	_____			

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME: LaSalle One
DATE September 10, 1988
COMPLETED BY S. Samolinski
TELEPHONE (815)357-6761

REPORT MONTH AUGUST, 1988

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
7	8/5/88		S	0.0	B	5	Load drop to gather RR data
8	8/24/88		S	0.0	H	5	Load drop to 644 MWe for control rod set

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no Safety Relief Valves actuations for Unit One during the reporting period August 1 through August 31, 1988.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
0-208-88	0 D/G cooling water pump 0 DG01P	Replace motor
0-210-88	0 DG01P	Repack coupling
0-211-88	0 D/G	Lubrication
1-1634-88	1A D/G	Repair air pilot valve
1-1635-88	1A RHR heat exchanger	Hydrolaze Service water side.
1-1683-88	LPCS pump	Change oil

3. Off-Site Dose Calculation Manual

Review of past Monthly Operating Reports submitted to the NRC identified that changes to the Offsite Dose Calculation Manual (ODCM) were incompletely reported. Technical Specifications require a determination that changes made will not reduce the accuracy or reliability of dose calculations or setpoint determinations. This information was not included in the submittal of revisions 11A, 12 and 12A to the ODCM.

Technical evaluation of revisions 11A, 12 and 12A to the ODCM has determined that the changes do not reduce the accuracy or reliability of the dose calculations and setpoint determinations have not been affected.

4. Radioactive Waste Treatment Systems.

No changes to Radioactive Waste Treatment systems during the reporting period.

5. Indications of Failed Fuel Elements

No indications of Failed Fuel Elements on Unit One during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

AUGUST, 1988

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Steven J. Samolinski, telephone number (815)357-6761 extension 705.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

August 1-31

August 1, 0000 hours Unit Two entered August with the reactor critical and the generator on-line at 750 MWe.

August 6, 1200 hours Load drop to 446 MWe to maintain condenser backpressure.

August 10, 0000 hours Load drop to 440 MWe per L.D.

August 10, 0530 hours Ramping to 857 MWe.

August 19, 0400 hours Generator off line to repair steam leaks.

August 20, 0430 hours Generator on line and ramping to 178 MWe.

August 22, 0500 hours Ramping to 665 MWe.

August 31, 2323 hours Generator taken off line. Reactor in process of being shutdown for Technical Specification Compliance.

August 31, 2400 hours Generator off line, reactor shutdown proceeding.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility license or Technical Specification.

There were no amendments to the Facility License or Technical Specifications during the month of August.

2. Changes to procedures which are described in the Safety Analysis Report.

There were no changes to procedures which are described in the Safety Analysis Report.

3. Tests and Experiments not described in the Safety Analysis Report.

There were no tests or experiments conducted during the month of August which are not described in the Safety Analysis Report.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of Safety-Related Maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

5. Completed Safety Related Modifications.

The following table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L8304i	PDS-2E31-N007BB Pressure Differential Switch	Ruptured diaphragm	Division II isolation and RCIC trip unavailable	Switch replaced

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER: A brief Synopsis of Incorporated Modification Objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

UNIT TWO

There were no Safety-Related Modifications completed on Unit Two during the month of August.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, July 1, through July 31, 1988. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
88-009-00	8/17/88	RCIC INOP Due to Failed SOR Switch.
88-010-00	8/31/88	Failure of ADS Backup Nitrogen Bottle Pressure Regulator.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE Sept. 10, 1988
 COMPLETED BY Steven J. Samolinski
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: August, 1988 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (Mwe-Net): 1036 DESIGN ELECTRICAL RATING (Mwe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (Mwe-Net): 725
4. REASONS FOR RESTRICTION (IF ANY): Fuel Depletion, Administrative
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|----------------|-----------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>744.0</u> | <u>5635.5</u> | <u>22438.3</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>29.8</u> |
| 7. HOURS GENERATOR ON LINE | <u>718.8</u> | <u>5611.7</u> | <u>22082.8</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>1700136</u> | <u>15973752</u> | <u>63768808</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>509725</u> | <u>5224376</u> | <u>21032587</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>489642</u> | <u>5021821</u> | <u>20104434</u> |
| 12. REACTOR SERVICE FACTOR | <u>100.0%</u> | <u>96.6%</u> | <u>66.1%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>100.0%</u> | <u>96.6%</u> | <u>66.2%</u> |
| 14. UNIT SERVICE FACTOR | <u>96.6%</u> | <u>95.8%</u> | <u>65.1%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>96.6%</u> | <u>95.8%</u> | <u>65.1%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>63.5%</u> | <u>82.8%</u> | <u>57.2%</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN Mwe) | <u>61.1%</u> | <u>79.6%</u> | <u>55.0%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>0.1%</u> | <u>4.0%</u> | <u>17.0%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

Unit 2 is scheduled for Second Refuel Outage on October 15, 1988. The outage is scheduled to last 15 weeks.

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 9/3/88

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
 UNIT: LASALLE TWO
 DATE: Sept. 10, 1988
 COMPLETED BY: Steven J. Samolinski
 TELEPHONE: (815) 357-6751
 MONTH: AUGUST, 1988

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1.	792.4	17.	748.2
2.	803.9	18.	736.5
3.	789.8	19.	17.5
4.	773.5	20.	105.0
5.	789.5	21.	162.0
6.	696.8	22.	465.8
7.	540.3	23.	745.0
8.	607.8	24.	747.7
9.	578.7	25.	717.8
10.	709.3	26.	752.2
11.	744.9	27.	741.5
12.	766.7	28.	736.3
13.	783.5	29.	730.1
14.	703.3	30.	723.7
15.	795.3	31.	634.3
16.	762.8		

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no Safety Relief Valve actuations during the reporting period August 1 through August 31, 1988 for Unit Two.

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE September 10, 1988
COMPLETED BY S. Samolinski
TELEPHONE (815)357-6761

REPORT MONTH AUGUST, 1988

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
11	8/6/88	F		0.0	B	5	Load drop to 446 MWe to maintain Condenser Backpressure.
12	8/19/88	S		24.5	B	5	Off-line to repair steam leaks.
13	8/31/88	F		0.62	A	5	Shutdown for Technical Specification compliance. Div I and II inoperative due to instrument nitrogen problems.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-424-88	2E51-F063 2E51-F076	Technical Specification 3.3.2, Bad SOR.
2-452-88	2E12-F047A	EQ MOV inspection.
2-453-88	2E12-F040A	EQ MOV inspection.

3. Off-Site Dose Calculation Manual

Review of past Monthly Operating Reports submitted to the NRC identified that changes to the Offsite Dose Calculation Manual (ODCM) were incompletely reported. Technical Specifications require a determination that changes made will not reduce the accuracy or reliability of dose calculations or setpoint determinations. This information was not included in the submittal of revisions 11A, 12 and 12A to the ODCM.

Technical evaluation of revisions 11A, 12 and 12A to the ODCM has determined that the changes do not reduce the accuracy or reliability of the dose calculations and setpoint determinations have not been affected.

4. Radioactive Waste Treatment Systems.

No changes to Radioactive waste treatment systems during this reporting period.

5. Indications of Failed Fuel Elements.

Off Gas levels indicate that there is one pinhole Fuel Element failure in the reactor vessel. This does not represent a change from the previous reporting period.

There were no further indications of failed fuel elements during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

September 10, 1988

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period August 1, 1988 through August 31, 1988.

Very truly yours,


G. J. Diederich 9/10/88
Station Manager
LaSalle County Station

GJD/SJS/jdp

Enclosure

xc: A. B. Davis, NRC, Region III
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