TABLE 15.4.1-1 (continued)

		17	ADLE 15.4.1-1 (Continued)			
NO.	CHANNEL DESCRIPTION	CHECK	CALIBRATE	TEST	PLANT CONDITIONS WHEN REQUIRED	
20.	Auxiliary Feedwater Flowrate	(13)	R		ALL	
21.	Boric Acid Control System		R		ALL	
22.	Boric Acid Tank Level	D	R	-	ALL	
23.	Charging Flow	-	R		ALL	
24.	Condensate Storage Tank Level	S(1)	R		ALL	
25.	Containment High Range Radiation	M(1)	R(14)		ALL	
26.	Containment Hydrogen Monitor -Gas Calibration -Electronic Calibration	D -	Q(15) R	:	ALL ALL ALL	
27.	Containment Pressure	S	R	Q(1,3,9)	ALL	
28.	Containment Water Level	M	R	-	ALL	
29.	Emergency Plan Radiation Survey Instruments	Q	A	Q	ALL	
30.	Environmental Monitors	M			ALL	
31.	In-Core Thermocouples	M	R(14)		ALL	
32.	Low Temperature Overpressure Protection System	S(12)	R	(10)	ALL	
33.	PORV Block Valve Position Indicator	Q	R		ALL	
34.	PORV Operability	-	R	Q(11)	ALL	
35.	PORV Position Indicator	S(21)	R	R	ALL	

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TABLE 15.4.1-1 (continued	TABI	E	15.4	1-1	(continued)
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			1ABLE 13.4.	1-1 (continued)		
	NO.	CHANNEL DESCRIPTION	CHECK	CALIBRATE	TEST	PLANT CONDITIONS WHEN REQUIRED
	36.	Radiation Monitoring System - RE-218 WDS Liquid Monitor - RE-223 Waste Distillate Overboard Monitor - RE-231 A Steam Line Release Monitor - RE-232 B Steam Line Release Monitor - RE-101 Control Room Monitor - RE-235 Control Room Noble Gas Monitor - RE-215 Air Ejector Monitor	(7) (7) M(1) M(1) S S D(1)	R(14) R(14) R(14) R(14) R(14) R(14) R(14)	Q Q - - Q Q	ALL ALL ALL ALL ALL ALL ALL ALL
	37.	Reactor Vessel Fluid Level System	M	R	-	ALL
	38.	Refueling Water Storage Tank Level	-	R		ALL
	39.	Residual Heat Removal Pump Flow	-	R	-	ALL
	40.	Safety Valve Position Indicator	M	R	-	ALL
	41.	Subcooling Margin Monitor	M	R	-	ALL
	42.	Deleted				
-	43.	Volume Control Tank Level	-	A	-	ALL
	44.	Reactor Protection System and Emergency Safety Feature Actuation System Logic	-	-	M(1,23)	ALL
	45.	Reactor Trip System Interlocks -Intermediate Range Neutron Flux, P-6 -Power Range Neutron Flux, P-8 -Power Range Neutron Flux, P-9 -Power Range Neutron Flux, P-10 -1st Stage Turbine Impulse Pressure		R(24) R(24) R(24) R(24) R(24)	R R R R	ALL ALL ALL ALL ALL
		-1st Stage Turbine Impulse Fressure		14(24)	K	TUL

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NOTATION USED IN TABLE 15.4.1-1

A-Annually (12 months)

S- Each shift

D- Daily

W- Weekly

Q- Quarterly

M- Monthly

P- Prior to reactor criticality if not performed during the previous week.

R-Each refueling interval (18 months)

PWR-Power and Low Power Operation, as defined in Specifications 15.1.h. and 15.1.ra.

HOT S/D- Hot Shutdown, as defined in Specification 15.1.g.1.

COLD S/D- Cold Shutdown, as defined in Specification 15.1.g.2.

REF S/D- Refueling Shutdown, as defined in Specification 15.1.g.3.

ALL- All conditions of operation, as defined in Specifications 15.1.g, h, and m.

NOTES USED IN TABLE 15.4.1-1

- (1) Not required during periods of refueling shutdown, but must be performed prior to reactor criticality if it has not been performed during the previous surveillance period.
- (2) Tests of the low power trip bistable setpoints which cannot be done during power operations shall be conducted prior to reactor criticality if not done in the previous surveillance interval.
- (3) Perform test of the isolation valve signal
- (4) Perform by means of the moveable incore detector system.
- (5) Recalibrate if the absolute difference is ≥3 percent.
- (6) Verification of proper breaker alignment and that the 120 Vac instrument buses are energized.
- (7) Source check is required prior to initiation of a release. Source check is an assessment of channel response by exposing the detector to a source of increased radiation. Channel check is required shiftly during a release. If monitor or isolation function is discovered inoperable, discontinue release immediately.
- (8) Verify that the associated rod insertion limit is not being violated at least once per 4 hours whenever the rod insertion limit alarm for a control bank is inoperable.
- (9) Test of Narrow Range Pressure, 3.0 psig, -3.0 psig excluded.

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