



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119  
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated March 17, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.2 of Facility Operating License No. DPR-32 is hereby amended to read as follows:

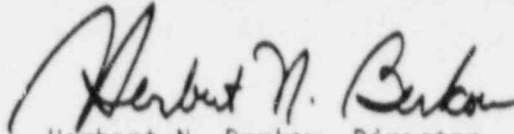
8805180054 880428  
PDR ADOCK 05000280  
P PDR

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 28, 1988



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119  
License No. DPR-37

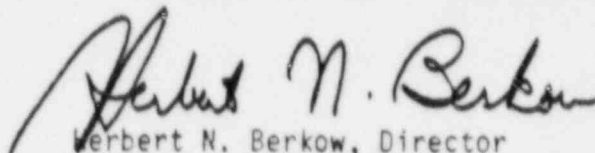
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated March 17, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 28, 1988

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 119 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 119 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

TS 6.1-1  
TS 6.1-7  
TS 6.1-8  
TS 6.1-8a  
TS 6.1-9  
TS 6.1-10  
TS 6.1-12  
TS 6.1-14  
TS 6.1-14a  
TS 6.1-15  
TS Figure 6.1-1  
TS Figure 6.1-2  
TS 6.2-1  
TS 6.3-1

Insert Pages

TS 6.1-1  
TS 6.1-7  
TS 6.1-8  
TS 6.1-8a  
TS 6.1-9  
TS 6.1-10  
TS 6.1-12  
TS 6.1-14  
TS 6.1-14a  
TS 6.1-15  
TS Figure 6.1-1  
TS Figure 6.1-2  
TS 6.2-1  
TS 6.3-1

6.0 ADMINISTRATIVE CONTROLS6.1 Organization, Safety and Operation ReviewSpecification

- A. The Station Manager shall be responsible for the overall operation of the facility. In his absence, the Assistant Station Manager (Operations and Maintenance) shall be responsible for the safe operation of the facility. During the absence of both, the Station Manager will delegate in writing the succession to this responsibility.

An onsite and an offsite organization shall be established for facility operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

1. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the UFSAR.
  2. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
  3. The Vice President-Nuclear shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
  4. The management position responsible for training of the operating staff and the position responsible for the quality assurance functions shall have sufficient organizational freedom including sufficient independence from cost and schedule when opposed to safety considerations.
  5. The management position responsible for health physics shall have direct access to that onsite individual having responsibility for overall facility management. Health physics personnel shall have the authority to cease any work activity when worker safety is jeopardized or in the event of unnecessary personnel radiation exposures.
- B. The facility organization shall conform to the following requirements:
1. Each member of the facility staff shall meet or exceed the minimum qualifications of ANS 3.1 (12/79 Draft)\* for comparable positions, and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for the Superintendent-Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

\*Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operations Phase."

f. Responsibilities

The SNSOC shall be responsible for:

1. Review of a) all proposed normal, abnormal, and emergency operating procedures and all proposed maintenance procedures and changes thereto, b) any other proposed procedures or changes thereto as determined by the Station Manager which affect nuclear safety.
2. Review of all proposed test and experiment procedures that affect nuclear safety.
3. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
4. Review of proposed changes to Technical Specifications and shall submit recommended changes to the Station Manager.
5. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President-Nuclear and to the offsite management position responsible for independent/operational event review.
6. Review of all Reportable Events and special reports submitted to the NRC.
7. Review of facility operations to detect potential nuclear safety hazards.
8. Performance of special reviews, investigations or analyses and report thereon as requested by the Chairman of the SNSOC or Station Manager.

9. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Station Manager.
10. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Station Manager.
11. Review of every unplanned onsite release of radioactive material to the environs exceeding the limits of Specification 3.11, including the preparation or reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear and to the offsite management position responsible for independent/operational event review.
12. Review of changes to the Process Control Program and the Offsite Dose Calculation Manual.

g. Authority

The SNSOC shall:

1. Provide written approval or disapproval of items considered under (1) through (3) above. SNSOC approval shall be certified in writing by an Assistant Station Manager.
2. Render determinations in writing with regard to whether or not each item considered under "(1)" through "(5)" above constitutes an unreviewed safety question.
3. Provide written notification within 24 hours to the Vice President-Nuclear and to the offsite management position responsible for independent/operational event review of disagreement between SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.A above.



h. Records

The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Vice President-Nuclear and to the offsite management position responsible for independent/operational event review.

## 2. Independent/Operational Event Review Group

### a. Independent Review Function

The Independent/Operational Event Review Group shall function to provide independent review of designated activities in the areas of:

1. Nuclear Power Plant Operations
2. Nuclear Engineering
3. Chemistry and Radiochemistry
4. Metallurgy
5. Instrumentation and Control
6. Radiological Safety
7. Mechanical and Electrical Engineering
8. Administrative Controls and Quality Assurance Practices
9. Other appropriate fields associated with the unique characteristics of the Nuclear Power Plant

### b. Composition

The Independent/Operational Event Review (IOER) staff shall be composed of the offsite management position responsible for Independent/Operational Event Review and a minimum of three individuals who qualify as staff specialists. Each Staff specialist shall have an academic degree in an engineering or physical science field and, in addition, shall have a minimum of five years technical experience in one or more areas given in Specification 6.1.C.2.a. These staff specialists shall not be directly involved in the licensing function.

### c. Consultants

Consultants shall be utilized to provide expert advice to the IOER when determined to be appropriate.

## d. Meeting Frequency

The IOER Staff shall meet at least once per calendar month for the purpose of fostering interaction of reviews regarding safety-related operational activities and related licensing activities.

## e. Review

The following subjects shall be reviewed by IOER Staff:

1. Written safety evaluations of changes in the stations as described in the Safety Analysis Report, changes in procedures as described in the Safety Analysis Report, and tests or experiments not described in the Safety Analysis Report which are completed without prior NRC approval under the provisions of 10 CFR 50.59(a)(1). This review is to verify that such changes, tests, or experiments did not involve a change in the Technical Specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2) and is accomplished by review of minutes of the Station Nuclear Safety and Operating Committee and the design change program.
2. Proposed changes in procedures, proposed changes in the station, or proposed tests or experiments, any of which may involve a change in the Technical Specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2). Matters of this kind shall be referred to the offsite management position responsible for IOER by the Station Nuclear Safety and Operating Committee following its review prior to implementation.

6. Any other matter involving safe operation of the nuclear power stations which is referred to the offsite management position responsible for independent/operational event review.
7. Reports and meeting minutes of the Station Nuclear Safety and Operating Committee.

f. Authority

The offsite management position responsible for independent/operational event review shall report through the established organization structure and advise the Vice President-Nuclear on any safety concerns discovered during the independent review of those areas of responsibility specified in Section 6.1.C.2.e.

g. Records

Records of IOER activities required by Specification 6.1.C.2.e shall be prepared and maintained in the IOER files and a summary shall be disseminated each calendar month to the Vice President-Nuclear, corporate management position responsible for the quality assurance audit program, and other appropriate management personnel.

6. The Station Security Plan and implementing procedures at least once per 12 months.
7. Any other area of facility operation considered appropriate by the corporate management position responsible for the quality assurance audit program or the Vice President-Nuclear.
8. The Station Fire Protection Program and implementing procedures at least once per 24 months.
9. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
10. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.
11. The radiological environmental monitoring program at least once per 12 months.
12. The Offsite Dose Calculation Manual and implementing procedures at least once per 24 months.

13. The Process Control Program and implementing procedures for processing and packaging of radioactive waste at least once per 24 months.

b. Authority

The Quality Assurance Department shall advise the Vice President-Nuclear on those areas of responsibility specified in 6.1.C.3.a above.

c. Records

Records of the Quality Assurance Department audits shall be prepared and maintained in the company files. Audit reports shall be disseminated to the Vice President-Nuclear and the management position responsible for the areas audited.

DELETED



DELETED

6.2 GENERAL NOTIFICATION AND REPORTING REQUIREMENTS

Specification

- A. The following actions shall be taken for Reportable Events:
1. A report shall be submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
  2. Each Reportable Event shall be reviewed by the SNSOC. The offsite management position responsible for independent/operational event review and the Vice President-Nuclear shall be notified of the results of this review.
- B. Immediate notifications shall be made in accordance with Section 50.72 to 10 CFR Part 50.

6.3 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDEDSpecification

- A. The following actions shall be taken in the event a Safety Limit is violated:
1. The facility shall be placed in at least hot shutdown within 1 hour.
  2. The Safety Limit violation shall be reported to the Commission, the Vice President-Nuclear, and the offsite management position responsible for independent/operational event review within 24 hours.
  3. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
  4. The Safety Limit Violation Report shall be submitted to the Commission, the offsite management position responsible for independent/operational event review and the Vice President-Nuclear within 14 days of the violation.