



Callaway Plant

May 9, 1988

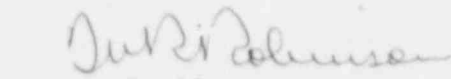
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC- 1770

Gentlemen:

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
SPECIAL REPORT 88-01
EMERGENCY CORE COOLING SYSTEM ACTUATION

The enclosed Special Report is submitted pursuant to
Technical Specification 3.5.2 and 6.9.2 concerning an inadvertent
Emergency Core Cooling System actuation.


J. D. Blosser
Manager, Callaway Plant

PSP:jlh

Enclosure

cc: Distribution attached

8805170177 880509
PDR ADOCK 05000483
S DCD

JE22
11

cc distribution for ULNRC-1770

Mr. A. Bert Davis
Regional Administrator
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Mr. Thomas Alexion (2 copies)
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Mail Stop 13-E-21
Washington, D.C. 20555

American Nuclear Insurers
c/o Dottie Sherman, Library
The Exchange Suite 245
270 Farmington Avenue
Farmington, CT 06032

Mr. O. Maynard
Wolf Creek Nuclear Operating Corp.
P. O. Box 411
Burlington, KS 66839

Manager, Electric Department
Missouri Public Service Commission
P. O. Box 360
Jefferson City, MO 65102

Mr. Merlin Williams
Supt. of Regulatory Quality &
Administrative Services
Wolf Creek Nuclear Operating Corp.
P. O. Box 411
Burlington, KS 66839

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

Mr. R. W. DeFayette
Chief, Project Section 3A
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

NRC Resident Inspector

D. F. Schnell (400)
R. J. Schukai (470)
G. L. Randolph
J. F. McLaughlin
W. R. Campbell
R. P. Wendling (470)
F. D. Field (480)
A. P. Neuhalfen
A. C. Passwater/D. E. Shafer/D. J. Walker (470)
G. A. Hughes
Z170.05 (QA Record)
Z40SR (Z170.09 Commercial Record)
M. E. Evans
M. E. Taylor
H. Wuertenbaecher, Jr. (100)
S. L. Auston (470)(NSRB)
S. J. Bellers/J. D. Schnack
JDB Chrono
3456-0021.6
3456-0260
Z40ULNRC
A160.761
N. Date (Sandra Auston) (470)

SPECIAL REPORT 88-01
EMERGENCY CORE COOLING SYSTEM ACTUATION

At 0522 CST on 2/13/88, a Low Steamline Pressure Signal (LSPS) was received resulting in a Safety Injection (SI). At the time of the event, the plant was in Mode 3, Hot Standby, 0% reactor power. This report is submitted pursuant to Technical Specifications 3.5.2 and 6.9.2.

During restoration from a turbine/reactor trip, utility licensed operators were not continuously cognizant of RCS temperature and consequently did not take effective action to stop the decreasing RCS temperature caused by excessive steam loads. The excessive steam loads were the result of open main steam line drain valves and a malfunctioned main steam seal automatic pressure regulator valve. As part of the restoration, Emergency procedure ES-0.1, "Reactor Trip Response", was entered. The procedure required the temperature of the RCS to be verified at 557°F, but did not provide guidance if the temperature fell below this value. RCS temperature decreased to 495°F resulting in correspondingly lower S/G temperatures and lower steam line pressures. The steam line pressure reached its low setpoint (615 psig) and an SI resulted.

In response to the SI, the main steam isolation valves shut. The RCS temperature immediately began to return to the normal average temperature of 557°F due to the isolation of the steam loads.

Licensed utility operators performed the appropriate emergency procedures and declared an Unusual Event at 0532 as required by the emergency plan. The plant was restored to a normal condition and the Unusual Event was terminated at 0645. This event was also reported in Licensee Event Report 88-004-00 transmitted via ULNRC-1742 dated 3/14/88.

The total accumulated actuation cycles at operating temperature for the Callaway Plant Emergency Core Cooling System is three. The previous actuations were reported by Special Report 84-03, via ULNRC-970, dated 11/8/84 and Special Report 85-03, via ULNRC-1105, dated 5/31/85. In all three cases, the usage factor for each Safety Injection nozzle was below the 0.70 limit specified in Technical Specification 3.5.2 action (b).