



# Duquesne Light

Nuclear Group  
P.O. Box 1  
Shippingport, PA 15077-0004

Telephone (412) 393-6000

May 4, 1988

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

Reference: Beaver Valley Power Station, Unit No. 1  
Docket No. 50-334, License No. DPR-66  
NRC Bulletin 88-02

Gentlemen:

In our reply to NRC Bulletin 88-02 "Rapidly Propagating Fatigue Cracks in Steam Generator Tubes" dated March 28, 1988, we stated that we would submit an evaluation to address Items C.2.a and C.2.b of the Bulletin for Beaver Valley Unit 1 by May 31, 1988. This evaluation has been completed and enclosed are:

1. (5) copies of WCAP-11799 entitled, "Beaver Valley Unit 1 Evaluation for Tube Vibration Induced Fatigue (Proprietary).
2. (5) copies of WCAP-11800 entitled, "Beaver Valley Unit 1 Evaluation for Tube Vibration Induced Fatigue (Non-Proprietary).

Also enclosed is a Westinghouse authorization letter, CAW-88-033, Proprietary Information Notice, and accompanying Affidavit.

As item 1 contains information proprietary to Westinghouse Electric Corporation, it is supported by an affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the bases on which the information may be withheld from public disclosure by the Commission and addresses with specificity the considerations listed in paragraph (b)(4) of Section 2.790 of the Commission's regulations.

Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR Section 2.790 of the Commission's regulations.

Correspondence with respect to the proprietary aspects of the Application for Withholding or the supporting Westinghouse Affidavit should reference CAW-88-03 and should be addressed to R. A. Wiesemann, Manager of Regulatory & Legislative Affairs, Westinghouse Electric Corporation, P.O. Box 355, Pittsburgh, PA 15230-0355.

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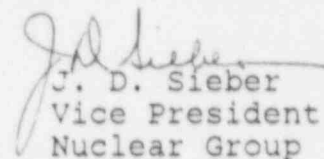
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The above evaluation provides the necessary information for NRC review and approval of our long term corrective actions to minimize the probability of a rapidly propagating fatigue failure of a steam generator tube. However, our Enhanced Primary to Secondary Leakrate Monitoring Program will remain in effect as an interim compensatory measure until NRC approval has been granted.

In our initial response to Item C.2 of the Bulletin, we stated that five tubes in the "C" steam generator and one tube in the "A" steam generator had been preventively plugged with leak limiting plugs. The final analysis in the above evaluation concludes that preventive plugging of the one tube (R10C43) in the "A" steam generator was not necessary. Therefore, this tube (R10C43) in the "A" steam generator may be returned to service during a future outage.

If there are any questions concerning this response, please contact my office.

Very truly yours,

  
J. D. Sieber  
Vice President  
Nuclear Group

Enclosures

cc: w/o enclosures

Mr. J. Beall, Sr. Resident Inspector  
Mr. W. T. Russell, NRC Region I Administrator  
Mr. P. Tam, Project Manager  
Director, Safety Evaluation & Control (VEPCO)