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100PBP0 301-1/0\//.	At 0505 on Ap was increasin The Operator investigate. full of water Trip" alarm w realizing tha the "Low Powe to conserve s "C" circulato steam was rou valves. Read was returned The expansion pump pit fail	ril 7, 1988, wit g power when a " began to reduce The Auxiliary T and smoke. At as received. Th t all circulatin r" position and team. The React r on condensate. ted through "D" tor depressuriza to service at ap joint on circul ed due to degrad	th the real Circulating load and do lender report the same time placed one tor Operator The decay circulator ation was in oproximately lating water dation and h	Wate spatc ted t me, a manua ild be circu manu heat and o itiat 0600 pump as be or wi	71% r Pum hed a "Cir lly s lost lator ally exch ut th hour "1A" en re	power, th p [1A] Tr n Auxilia rculating culating crammed t . The Op per loop shutdown anger was e reheat d the dec s. in the cc placed.	e Reactor ip" alarr ry Tender water pu Water Pur he reactor erator th on self Loop I ar cleared power re ay heat f irculatin All simi pected an	r Operator n came up. r to ump pit was mp [1B] or hen went to turbining nd put out so lief exchanger

NRC Form 366A . LICENSEE EVENT RE	PORT (LER) TEXT CONTINU	JATION	APPROVED OF EXPIRES: 8/31/	ULATORY COMMISSION MB NO. 3150-0104 88
FACILITY NAME (1)	DOCKET NUMBER (2)	LEP NUMBER	(6)	PAGE (3)
		YEAR SEQUENTI	AL REVISION NUMBER	
Fort St. Vrain, Unit No. 1	0 15 10 10 10 12 16 17	818 - 01016	5 -010	012 05 016
TEXT (# more appear is required, use additional NRC Form 305A'z) (17)		12121-12121		
DESCRIPTION OF EVENT:				
Initial Conditions				
Reactor				
Reactor Power: 71% Average Core Inlet Temperature Average Core Outlet Temperatur Reactor Pressure: 635 PSIA	: 676 degrees Fahrenhei e: 1445 degrees Fahrenh	t eit		
Primary Coolant:				
"A" Circulator Speed: 7041 RP "B" Circulator Speed: 6876 RP "C" Circulator Speed: 6900 RP "D" Circulator Speed: 6876 RP	M M M			
Total Percent Helium Flow: 74	4.9% (2612 KPPH)			
Secondary Coolant:				
Loop I Feedwater Flow: 760.9 Loop II Feedwater Flow: 774.0	Э КРРН Э КРРН			
Circulating Water Pump Pit Des	scription:			
The circulating water pump pit tower. The pit dimensions are with a total volume of approxi- pump pit sump located in the deep with a volume of 125 cub- sump is 38,740 cubic feet. The (1A and 1B) with a pumping cap circulating water pumps (1C & and two circulating water pump 150 GPM each.	t is located south of the e 29.5 feet wide, 77 feet imately 38,615 cubic feet southeast corner is 5 fee ic feet. Total volume of he pit contains two large pacity of 67,000 GPM each 1 D) with a pumping cap p pit sump pumps with a p	e circulating t long, and 17 t. The circul et by 5 feet b f both the pit e circulating h, two smaller acity of 11,00 pumping capaci	water feet deep ating wate y 5 feet and the water pump 0 GPM each ty of), er),

EVENT CHRONOLOGY :

At 1207 on April 6, 1988, a Station Service Request (SSR# 88502258) was written against the level switch LS-7510-2 for the circulating water pump pit sump because it allowed the sump pumps to run continuously. The pumps were placed in manual and the Auxiliary Tender was responsible for checking the sump and pumping it out on a periodic basis. This switch discrepancy caused the alarm to remain permanently lit in the control room, therefore defeating the only level alarm for the pit.

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FACILITY NAME (1)	DOCKET NUMBER 121	LE	R NUMBER 6
Fort St. Vrain, Unit No. 1		YEAR	SEGUINTIAL NUMBER
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Form 366A

At approximately 0440 hours on April 7, 1988, the Auxiliary Tender made a routine check of the circulating water pump pit and manually pumped down the sump. Nothing abnormal was observed at that time.

Minutes before the event, the "Circulating Water Pump Pit Sump High Level" alarm cleared momentarily, but came right back on so no action was taken at that time.

At 0505 hours on April 7, 1988, the Reactor Ope r received a "Circulating Water Pump Tripped" alarm on Circulating Water sump "1A". The Reactor Operator began to reduce load and dispatched an Auxiliary Tender to investigate the problem.

At 0508 hours, the Auxiliary Tender reported the circulating water pump pit was full of water and smoke. At the same time, another "Circulating Water Pump Tripped" alarm was received on Circulating Water Pump "1B". A "Circulating Water Pressure (low pressure)" alarm was also received.

At 0509 hours, the Reactor Operator manually scrammed the reactor due to the loss of the condenser as a heat sink (see FSAR Section 10.3.4). One circulator per loop was placed on self-turbining to conserve steam.

At 0535 hours, the Reactor Operator manually shutdown Loop 1 and put "C" helium circulator on condensate to maintain it at self-turbining speed. The Operator maintained "D" helium circulator on steam from the bypass flash tank and continued venting to atmosphere out of the hot reheat power operated relief valves. Only the one circulator was used on steam to conserve the condensate inventory.

At 0605 hours, depressurization of the reactor (primary coolant) was started and circulators "C" and "D" were placed on pelton wheel drive with emergency feedwater, again to conserve the condensate inventory. The decay heat exchanger was placed in service.

At 0610 hours, it was noted that reactor plant exhaust stack radiation monitor RT-7324-1 was showing increased activity readings. The release was calculated to be 5.3% of the Technical Specification ELCO 8.1.1(a) limits resulting in a total dose of 4.45E-5 REM at the Exclusion Area Boundary. The cause and corrective actions associated with this release were identified in LER 88-004.

At 0650 hours, the Loop II Economizer-Evaporator-Superheat section (EES) was put on condensate and circulators "C" and "D" were put on condensate. At the same time. a Notice of Unusual Event (NOUE) was declared because of the unplanned release indicated in the reactor plant exhaust stack. LICENSEE EVENT REPORT 'LERI TEXT CONTINUATION

U.S NUCLEAR REGULATORY COMMISSION

APPROVED OM8 NO 3150-0104 EXPIRES 8-31 88

FARILITY NAME (1)	DOCKET NUMBER (2) LER NUMBER (6)			
Fort St. Vrain, Unit No. 1		VEAR SEQUENTIAL REVISION		
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CAUSE OF EVENT:

The event was caused by a catastrophic failure of an expansion joint on the discharge side of circulating water pump "1A". The expansion joint is rated at 55 psi (the pump is rated at 32.6 psi). Circulating water pumps "1A" and "1B" were running at the time of the event. Assuming 20% of pump flow passed through the 24 inch tear in the expansion joint, it would have taken 21.7 minutes to completely fill the circulating water pump pit, and only 14.4 minutes to fill the pit to the point at which it was discovered. The expansion joint which failed can be isolated but the pump suction valve is manually operated, and the valve has a high gear ratio due to its size. It would have taken six to ten minutes just to shut the valve. Also, access into the pit would have required the Auxiliary Tender to pass across the leak and become exposed to an electrocution hazard from the 480V and 4160V sources. Therefore, isolation of the leak required all power to be shut off in the pit, which would shut off all circulating water pumps, and result in a complete loss of condenser vacuum. The subsequent reactor scram was inevitable.

The expansion joint on the discharge of circulating water pump "1A" apparently failed due to weakening of the internal reinforcing fibers, due to age and exposure to the weather. The root cause of this failure was the lack of an adequate preventive maintenance program for the plant rubber expansion joints. An inspection of the other joints in the circulating water pump pit revealed degradation (in the form of weak spots, cracking, or surface abrasion) on four joints in addition to the one that failed. All five of these joints have now been replaced, as well as four others on the circulating water system at the condenser inlet and outlet waterboxes.

The expansion joint which failed was manufactured by the Uniroyal Company, Model Number CH-4140, and was 54 inches in diameter. It was approximately 15 years old. The design life expectancy on these joints is approximately ten years.

The high activity indication on RT-7324-1 was the result of the core support floor vent line relief valve, V-6389, lifting due to excess pressure and discharging unpurified helium into the reactor plant exhaust stack. The cause and corrective action were addressed in LER 88-004.

ANALYSIS OF EVENT:

This event resulted in a manual scram and cooldown of the reactor, and is reported herein per 10CFR50.73(a)(2)(iv).

As a result of the failure of the one expansion joint, all circulating water pumps became inoperable. In response to this, the plant operations personnel manually scrammed the reactor and stabilized the plant in accordance with standard operating procedures. No unusual problems were encountered during the plant shutdown. There would have been no difference in consequences if this failure had occurred at a higher power level. Therefore, it is concluded that this incident posed no threat to the health and safety of the public.

U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMU NO 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)
Fort St. Vrain, Unit No. 1		VEAR SEQUENTIAL REVISION NUMBER	
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CORRECTIVE ACTION:

NRC Form 366A

All eight expansion joints in the circulating water pit will be replaced. Five were replaced prior to plant restart on April 21, 1988, and the remainder will be replaced during the helium circulator outage presently scheduled for July 5, 1988. In addition, all similar expansion joints in the plant in critical service functions have been inspected for acceptability with the remainder to be inspected as soon as possible. These expansion joints will be replaced as necessary.

The level switches on the circulating water pump pit sump pumps have been repaired and restored to operation.

A Preventive Maintenance program for the inspection and periodic replacement of the expansion joints will be developed. Estimated implementation date for this program is the end of the helium circulator outage, scheduled for October 1988.

U.S. NUCLEAR REGULATORY COMMISSION NRC Form 364A LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OM" NO 3150-0104 EXPIRES: 8/31/88 DOCKET NUMBER (2) PAGE (3) FACILITY NAME (1) LER NUMBER (6) SEQUENTIAL REVISION YEAR Fort St. Vrain, Unit No. 1 0 |5 |0 |0 |2 |6 |7 8 |8 - 0 |0 | 6 - 0 |0 0 |6 OF 0 | 6 TEXT (If more space is required, use additional NRC Form 386A's) (17) T- D. The Terry D. Staley Results Engineer Mark Jorgh Mark Joseph Technical Services Supervisor .M. Gramling Licensing C. H. Fuller by Allt Manager, Nuclear Production

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Public Service Company of Colorado

16805 WCR 19 1/2, Platteville, Colorado 80651

May 9, 1988 Fort St. Vrain Unit No. 1 P-88160

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Docket No. 50-267

SUBJECT: Licensee Event Report 88-006-00, Final Report

REFERENCE: Facility Operating License No. DPR-34

Gentlemen:

Enclosed please find a copy of Licensee Event Report No. 50-267/88-006-00, Final, submitted per the requirements of 10 CFR 50.73(a)(2)(iv).

If you have any questions, please contact Mr. M. H. Holmes at (303) 480-6960.

Sincerely,

C.H. Fuller by

C. H. Fuller Manager, Nuclear Production

Enclosure

cc: Regional Administrator, Region IV ATTN: Mr. T. F. Westerman, Chief Projects Section B

> Director Nuclear Reactor Regulation ATTN: Mr. J. A. Calvo, Director Project Directorate IV

Mr. R. E. Farrell Senior Resident Inspector, FSV

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