

UNITED STATES  
ATOMIC ENERGY COMMISSION  
Washington, D.C. 20545

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

FACILITY OPERATING LICENSE

License No. DPR-42  
Amendment No. 2

1. The Atomic Energy Commission (the Commission) having found that:
  - A. The application for license filed by Northern States Power Company (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and that all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the Prairie Island Nuclear Generating Plant, Unit 1 (the facility), has been substantially completed in conformity with Provisional Construction Permit No. CPPR-45, as amended, the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
  - E. The licensee is technically and financially qualified to engage in the activities authorized by this operating license in accordance with the rules and regulations of the Commission;
  - F. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements", of the Commission's regulations;

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- G. The issuance of this amended license will not be inimical to the common defense and security or to the health and safety of the public;
  - H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Amendment No. 2 to Facility Operating License No. DPR-42 is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
  - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30 and 70, including 10 CFR Section 30.33, 70.23 and 70.31.
2. Pursuant to an Initial Decision of the Atomic Safety and Licensing Board (ASLB) dated April 2, 1974, Facility Operating License DPR-42 (issued to Northern States Power Company on August 9, 1973, and amended on December 14, 1973) is hereby amended in its entirety to read as follows:
- A. This amended license applies to the Prairie Island Nuclear Generating Plant, Unit 1, a pressurized water nuclear reactor and associated equipment (the facility), owned by the Northern States Power Company. The facility is located in Goodhue County, Minnesota, and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 36) and the Environmental Report as supplemented and amended (Supplements 1 and 2).
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Northern States Power Company:
    - (1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities, to possess, use, and operate the facility at the designated location in Goodhue County, Minnesota, in accordance with the procedures and limitations set forth in this license;
    - (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended as of May 11, 1976.
    - (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

UNIT 1 LICENSE

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility."

Amendment 12

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The license is authorized to operate the facility at steady state reactor core power levels not in excess of 1650 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 76, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

Amendment 76

(3) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the following Commission approved documents, including amendments and changes made pursuant to the authority of 10 CFR 50.54 (p). These approved documents consist of information withheld from public disclosure pursuant to 10 CFR 2.790(d):

- a. "Prairie Island Nuclear Generating Plant Security Plan"-- Revision 4 filed on March 3, 1978 and Revision 5 filed September 25, 1978.
- b. "Prairie Island Nuclear Station Safeguards Contingency Plan", dated March 23, 1979, as revised by submittal dated August 20, 1980 which contained revised pages dated July 1, 1980, submitted pursuant to 10 CFR 73.40. The Contingency Plan shall be fully implemented, in accordance with 10 CFR 73.40(b), within 30 days of this approval by the Commission (February 25, 1981).
- c. "Prairie Island Nuclear Generating Plant Security Guard Force Training and Qualification Plan", submitted by letter dated August 17, 1979 as amended by Revision 1 submitted May 16, 1980. This Plan shall be followed in accordance with 10 CFR 73.55(b)(4), 60 days after approval by the Commission (February 25, 1981). All security personnel, as required in the above plans, shall be qualified within two years of this approval (February 25, 1981). The licensee may make changes to this plan without prior Commission approval if the changes do not decrease the safeguards effectiveness of the plan. The licensee shall maintain records of and submit reports concerning such changes in the same manner as required for changes made to the Safeguards Contingency pursuant to 10 CFR 50.54(p).

(4) Fire Protection

The licensee may proceed with and is required to complete the modifications identified in Paragraphs 3.1.1 through 3.1.21 of the NRC's Fire Protection Safety Evaluation, dated September 6, 1979 for the facility according to the schedule in Table 3.1. If any modifications cannot be completed on schedule the licensee shall submit a report explaining the circumstances and propose, for staff approval, a revised schedule.

In addition, the licensee shall submit the additional information identified in Sections 3.1 and 3.2 of the related Safety Evaluation in accordance with the schedule contained therein. In the event these dates for submittal cannot be met, the licensee shall submit a report, explaining the circumstances, together with a revised schedule.

The licensee is required to develop and implement the administrative controls identified in Section 6 of the related Safety Evaluation within four months from the date of this amendment.

D. This amended license is effective as of the date of issuance and shall expire at midnight on August 9, 2013.

FOR THE ATOMIC ENERGY COMMISSION

/s/

Attachment:  
Change No. 3 to  
Appendices A and B

Date of Issuance:

April 5, 1974

UNITED STATES  
ATOMIC ENERGY COMMISSION  
Washington, D.C. 20545

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

FACILITY OPERATING LICENSE

License No. DPR-60

1. The Atomic Energy Commission (the Commission) having found that:
  - A. The application for license filed by Northern States Power Company (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and that all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the Prairie Island Nuclear Generating Plant, Unit 2 (the facility), has been substantially completed in conformity with Provisional Construction Permit No. CPPR-46, as amended, the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
  - E. The licensee is technically and financially qualified to engage in the activities authorized by this operating license in accordance with the rules and regulations of the Commission;
  - F. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements", of the Commission's regulations;
  - G. The issuance of this amended license will not be inimical to the common defense and security or to the health and safety of the public;

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Facility Operating License No. DPR-60 is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
  - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70, including 10 CFR Section 30.33, 40.32, 70.23, and 70.31.
2. Facility Operating License No. DPR-60 is hereby issued to Northern States Power Company to read as follows:
- A. This license applies to the Prairie Island Nuclear Generating Plant, Unit 2, a pressurized water nuclear reactor and associated equipment (the facility), owned by the Northern States Power Company. The facility is located in Goodhue County, Minnesota, and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 38) and the Environmental Report as supplemented and amended (Supplements 1 and 2).
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Northern States Power Company:
    - (1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities, to possess, use, and operate the facility at the designated location in Goodhue County, Minnesota, in accordance with the procedures and limitations set forth in this license;
    - (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended as of May 11, 1976.
    - (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any by-product, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such by-product and special nuclear materials as may be produced by the operation of the facility."

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The license is authorized to operate the facility at steady state reactor core power levels not in excess of 1650 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 69, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

The license shall fully implement and maintain in effect all provisions of the following Commission approved documents, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p). These approved documents consist of information withheld from public disclosure pursuant to 10 CFR 2.790(d):

- a. "Prairie Island Nuclear Generating Plant Security Plan" -- Revision 4 filed on March 3, 1978 and Revision 5 filed September 25, 1978.



- b. "Prairie Island Nuclear Station Safeguards Contingency Plan", dated Mach 23, 1979, as revised by submittal dated August 20, 1980 which contained revised pages dated July 1, 1980, submitted pursuant to 10 CFR 73.40. The Contingency Plan shall be fully implemented, in accordance with 10 CFR 73.40(b), within 30 days of this approval by the Commission. (February 25, 1981)
  
- c. "Prairie Island Nuclear Generating Plant Security Guard Force Training and Qualification Plan", submitted by letter dated August 17, 1979 as amended by Revision 1 submitted May 16, 1980. This Plan shall be followed in accordance with 10 CFR 73.55(b)(4), 60 days after approval by the Commission (February 25, 1981). All security personnel, as required in the above plans, shall be qualified within two years of this approval (February 25, 1981). The licensee may make changes to this plan without prior Commission approval if the changes do not decrease the safeguards effectiveness to the plan. The licensee shall maintain records of and submit report concerning such changes in the same manner as required for changes made to the Safeguards Contingency Plan pursuant to 10 CFR 50.54(p).

(4) Fire Protection

The licensee may proceed with and is required to complete the modifications identified in Paragraphs 3.1.1 through 3.1.21 of the NRC's Fire Protection Safety Evaluation, dated September 6, 1979 for the facility according to the schedule in Table 3.1. If any modifications cannot be completed on schedule the licensee shall submit a report explaining the circumstances and propose, for staff approval, a revised schedule.

In addition, the licensee shall submit the additional information identified in Sections 3.1 and 3.2 of the related Safety Evaluation in accordance with the schedule contained therein. In the event these dates for submittal cannot be met, the licensee shall submit a report, explaining the circumstances, together with a revised schedule.

The licensee is required to develop and implement the administrative controls identified in Section 6 of the related Safety Evaluation within four months from the date of this amendment.

D. This license is effective as of the date of issuance and shall expire at midnight on October 29, 2014.

FOR THE ATOMIC ENERGY COMMISSION

/s/

Date of Issuance:

October 29, 1974

UNIT 2 LICENSE