

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) NORTH ANNA POWER STATION UNITS 1 AND 2	DOCKET NUMBER (2) 0 5 0 0 0 3 3 8	PAGE (3) 1 OF 0 3
---	--------------------------------------	----------------------

TITLE (4)
MISSED SURVEILLANCE - POST ACCIDENT CONTAINMENT PRESSURE TRANSMITTERS

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
04	05	88	88	01	8	00	05	05	NORTH ANNA, UNIT 2		0 5 0 0 0 3 3 9
											0 5 0 0 0 1 1 1

OPERATING MODE (9) 1

POWER LEVEL (10) 1 10 10

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

20.402(b)	20.405(e)	50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(e)(1)	50.73(a)(2)(v)	73.71(e)
20.405(a)(1)(ii)	50.36(e)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME G. E. Kane, Station Manager	TELEPHONE NUMBER
	AREA CODE 7 0 3 8 9 4 - 5 1 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1630 hours, on April 5, 1988, with Units 1 and 2 at 100 percent power (Mode 1), it was discovered that the channel check surveillance requirement for containment post accident pressure instruments were not being performed. The surveillance requirements of Technical Specification 4.3.3.6 requires a channel check at least once per 31 days for these channels. The containment wide range post accident pressure channels were installed in May 1981 for Unit 1 and July 1981 for Unit 2 in order to comply with the requirements of NUREG-0578 and NUREG-0737. This event is reportable pursuant to 10CFR50.73(a)(2)(i)(B).

The cause of the missed surveillances is attributed to the failure to recognize that the modification that installed the containment wide range pressure transmitters was implemented to meet the requirements of Technical Specification 3.3.3.6 for wide range post accident pressure instrumentation.

As an immediate corrective action, the Action Statement of Technical Specification 3.3.3.6 was entered.

There were no safety implications as a result of the event. The required channel calibrations were being performed in accordance with Technical Specification requirements. The health and safety of the general public were not affected at any time during this event.

8805120282 880505
PDR ADOCK 05000338
S DCD

IF22 4

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) NORTH ANNA UNITS 1 AND 2	DOCKET NUMBER (2) 0 5 0 0 0 3 3 8	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 8	- 0 1 8	- 0 0	0 2	OF	0 3

TEXT (if more space is required, use additional NRC Form 306A's) (17)

1.0 Description of Event

At 1630 hours, on April 5, 1988, with Units 1 and 2 at 100 percent power (Mode 1), it was discovered that the channel check surveillance requirement for the containment post accident pressure channels were not being performed. This discovery was made during the replacement of the wide range containment pressure transmitter (EIIS System Identifier IK, Component Identifier PT) on Unit 1. The transmitters were being replaced because they were nearing the end of their environmentally qualified service time. Additional investigation also determined that the requirement was not being fulfilled for Unit 2. The surveillance requirements for containment pressure are stated in Technical Specification 4.3.3.6 and require a channel check at least once per 31 days. The containment wide range pressure channels were installed in May 1981 for Unit 1 and July 1981 for Unit 2 in order to comply with the requirements of NUREG-0578 and NUREG-0737. Containment pressure channels (PT-LM-110A and B and PT-LM-210A and B) were installed to meet these requirements and are in addition to the existing containment pressure channels that provide signals to the Engineered Safety Features (ESF) Actuation Logic.

This event is reportable pursuant to 10CFR50.73(a)(2)(1)(B).

2. Safety Consequences and Implications

There were no significant safety consequences as a result of the missed channel check surveillance. Although not identified as part of the formal Technical Specification surveillance program, investigation has revealed that instrument channel calibrations had been performed at 18 month intervals since the installation of the wide range post accident containment pressure transmitters. Therefore, the channel calibration requirements of Technical Specification 4.3.3.6 were met. The transmitter output is provided directly to the control room and is monitored by the control room operators. As a result, instrument drift or failure would have been easily detectable. Also, two of four narrow range pressure channels that provide signals to the ESF Actuation Logic were being used as an interim measure to meet the intent of Technical Specification 3.3.3.6.

3. Causes of the Event

The cause of the missed surveillances is attributed to the failure to recognize that the modification that installed the containment wide range pressure transmitters was implemented to meet the requirements of Technical Specification 3.3.3.6 for wide range post accident containment pressure instrumentation. During the review of the design change, personnel did not conclude that a new procedure was required to perform the Technical Specification required channel check or channel calibration.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) NORTH ANNA UNITS 1 AND 2	DOCKET NUMBER (2) 0 5 0 0 0 3 3 8	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 8	- 0 1 8	- 0 0	0 3	OF 0 3

TEXT (if more space is required, use additional NRC Form 366A's) (17)

4. Immediate Corrective Action

At 1630 hours, on April 5, 1988, the Action Statement of Technical Specification 3.3.3.6 was entered for each unit because the Technical Specification channel check was not being performed. This action requires that with the number of operable channels less than the minimum required, restore the operable channels within 48 hours or be in hot shutdown within the next 12 hours. At 1518 hours, on April 6, 1988 the 48 hour Action Statement on the Unit 1 Accident Monitoring Instrumentation was cleared. At 1432 hours, on April 6, 1988, the 48 hour Action Statement on the Unit 2 Accident Monitoring Instrumentation was cleared. The Action Statements were cleared by completion of the transmitter replacement and the performance of the Technical Specification required channel check.

5. Additional Corrective Actions

The Control Room Operator logs, 1-LOG-4 and 2-LOG-4, have been permanently revised to perform channel checks once per 8 hour shift.

6. Actions Taken To Prevent Recurrence

No further actions are required since previous programmatic upgrades to the design change review process would minimize the probability of a similar occurrence.

7. Similar Events

No similar reportable events involving missed surveillances as a result of an inadequate design change implementation review have been identified.

8. Additional Information

The Instrument Department procedures for calibrating the containment wide range pressure transmitters will be included in the Periodic Test Scheduling System to ensure proper scheduling.

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION
P. O. BOX 402
MINERAL, VIRGINIA 23117

May 5, 1988

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. N-88-019
NO/JHL: nih
Docket Nos. 50-338
50-339

License Nos. NPF-4
NPF-7

Dear Sirs:

The Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Units 1 and 2.

Report No. LER 88-018-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very truly yours,

for *ME Bowling*
G. E. Kane
Station Manager

Enclosure

cc: U. S. Nuclear Regulatory Commission
101 Marietta Street, N. W.
Suite 2900
Atlanta, Georgia 30323

Mr. J. L. Caldwell
NRC Senior Resident Inspector
North Anna Power Station

IEE
11