				LICENS	SEE EVENT	REPORT	(LER)				
Facility Na	me (1)	Braidwood	d Unit 1					Ocket Num	ol 01 4	Page (3)	
litle (4)	Reactor	Trip Due	Negative Rate	Trip as a Re	esult of I	mprope	er Admin	nistrative Co	ontrols		
Event Dat	e (5)	1	LER Number (6)	)	Repor	t Date	(7)	Other I	aciliti	es Involved (8)	
Month   Day	Year	Year	/// Sequential	/// Revision /// Number	Month	Day	Year	Facility	lames	Docket Number(s)	
	1.							None		0 5 0 0 0 0	1
018 11	1 8 8	81.8	01116	01 0	018	21 3	818			01 51 01 01 01 1	1
OPERATING MODE (S POWER LEVEL (10)	1 0		THIS REPORT IS (Check one or s 20.402(b) 20.405(a)( 20.405(a)( 20.405(a)( 20.405(a)( 20.405(a)(	SUBMITTED PL           nore of the 1           1)(i)         4           1)(ii)         5           1)(iii)         5           1)(iv)         5	JRSUANT TO following) 20.405(c)(1 50.36(c)(2 50.73(a)(2 50.73(a)(2 50.73(a)(2 50.73(a)(2	(11) () () ()(i) ()(i) ()(ii) ()(ii)	X 51 51 51 51 51 51 51 51 51 51 51 51 51 5	0.73(a)(2)(i) 0.73(a)(2)(v) 0.73(a)(2)(v) 0.73(a)(2)(v) 0.73(a)(2)(v) 0.73(a)(2)(v) 0.73(a)(2)(v)	<pre>() ) (ii) (iii)(A) (iii)(B) )</pre>		i fy
Name				LICENSE	CONTACT	FOR TH	IS LER	(12)	TEI	LEPHONE NUMBER	*****
Cr	ig Chova	n. Shift COMPL	Engineer ETE ONE LINE FO	OR EACH COMPO	Ext. 2	URE DE	SCRIBE	8   D IN THIS REP	1 5 PORT (1)	4  5  8  -  2  8	0
CAUSE SY	STEM CO	MPONENT	MANUFAC- RI	TO NPRDS	ALL CAL	ISE	SYSTEM	COMPONENT	TURE	AC- REPORTABLE / R TO NPRDS /	11
X A	A F	ULL	\$  1  5  6	N			+				33
		SUPPLE	MENTAL REPORT	EXPECTED (14)	)				Expect	ted  Month   Day	Ye
	· vas. co	mplete E	XPECTED SUBMIS	STON DATE)		0			Submis: Date	(15)	

On August 11, 1988, the Rod Control Urgent Alarm annunciated three times. Also, the C-11 Rod Stop alarm was annunciating prior to 2.73 steps on Bank D control rods. Preparations were made to check the power cabinet 18D fuses. Separate discussions were held to discuss the details of checking the fuses. The unit operator was instructed to place rod control in MAXUAL and to perform no rod movement during the time the power disconnect switch was open. The unit operator was occupied with normal duties and observed that the C-11 Rod Stop alarm had annunciated again and attempted to clear it by manually moving rods. When rod motion was demanded, the stationary gripper coils deenergized and Control Bank D Group 1, rods dropped. This caused an automatic reactor trip on High Flux Negative Rate. Contributing to this event was a lack of administrative controls to indicate an off-normal system status. The cause of the Rod Urgent alarms is attributed to two bad fuses in cabinet 18D which were replaced. This event has been reviewed with the individuals involved. Additional administrative controls will be developed to aid the operator in identifying off-normal equipment status. This report will be included in the licensed operator Required Reading Program. There have been no previous occurrences.

KET 1

FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)							Page (3)				
Braidwood Unit 1				Year /// Sequential /// Number		14/1	/// Revision /// Number								
		0151010	0 0 4 5 6	81	8	-	01	116	-	01	0	01 3	OF	01	3
Α.	PLANT CONDITIONS PRIOR T	0 EVENT:	(2113) COURS	are 1	Genci		0 11	che ce							
	Unit: Braidwood 1	Event Date:	August 11, 19	88_			Even	t Time:	_13	116					
	Reactor Mode: _1_	Mode Name: .	Power Operatio	<u>n</u>		P	ower	Level:	_10	005					
	PCS TABL Temperature/Pre	NOT /NOT													

B. DES AIPTION OF EVENT:

There were no systems or components inoperable at the beginning of the event which contributed to the severity of the event.

On August 11, 1988, during normal operations on the day shift the Rod Control Urgent Alarm [AA] annunciated three times. It was determined that the problem was a phase failure on the movable gripper coil in power cabinet iBD. It was decided to check and if necessary replace the fuses in the cabinet.

During the day shift and on previous shifts, the C-11 Rod Stop alarm was annunciating prior to 223 steps on Bank D control rods. The alarm was also clearing at the wrong setpoint.

At approximately 1300, preparations were made with the Station Control Roor Engineer (SCRE), the unit Nuclear Station Operator (NSO), an extra NSO and the Technical Staff to check the power cabinet IBD fuses. Separate dicussions were held by the SCRE, prior to the start of the work; one discussion with the unit NSO and the other discussion with the extra NSO and the Technical Staff to discuss the details of checking the fuses.

At approximately 1311, the unit NSO was instructed to place rod control in MANUAL and to perform no rod movement during the time the power disconnect switch was open to replace the movable gripper coil fuses. The extra NSO and the Technical Staff Engineer met the Shift Engineer (SE) by the 18D power cabinet. The Technical Staff Engineer showed the SE what the job entailed and the SE left. The extra NSO opened the disconnect switch at power cabinet 18D and proceeded to take voltage readings across the fuses to ensure it was de-energized. The unit NSO was occupied with normal duties and observed that the C-11 Rod Stop alarm had annunciated again.

At 1316, while the extra NSO was taking the voltage reading across the third fuse, the unit NSO attempted to clear the C-11 Rod Stop alarm by moving manually moving rods.

When rod motion was demanded the stationary gripper coils deenergized. Since power was removed from the movable gripper coils for cabinet 18D, they could not energize.

As a result, Control Bank D Group 1, rods D4 and M12 dropped. This caused an automatic reactor trip on High Flux Negative Rate.

The appropriate NRC notification via the ENS phone system was made at 1458 on August 11, 1988, pursuant to 10CFR50.72(b)(2)(ii).

This event is being reported pursuant to 10CFR50.73(a)(2)(iv) = Any event or condition that resulted in manual or automatic actuation of any engineered safety feature, including the reactor protection system.

FACTUATY NAME (1)	L DOCKET NUMBER (2)	I LER N	UMBER	(6)			P	age (	3)
Braidwood Unit 1		Year	144	Sequential Number	144	Revision Number			
	01510101014151	5 8 1 8	-	0 1 1 1 6	-	0 1 0	01 3	OF	01

TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [xx]

## C. CAUSE OF EVENT:

The root cause of the reactor trip was a cognitive personnel error by the licensed un. "50.

Contributing to this event was a lack of administrative controls to indicate an off-normal system status.

The cause of the C-11 Rod Stop alarm annunciating prematurely on day shift was the failure to reset the Pulse/Analog converter when resetting the Rod Urgent Failure alarms which had occurred earlier in the day.

The cause of the Rod Urgent alarms is attributed to two bad fuses in cabinet 180.

#### D. SAFETY ANALYSIS:

There was no effect on plant or public safety as all systems operated as designed in response to the High flux Negative Rate. This event is enveloped by the Final Safety Analysis Report (FSAR). A dropped Rod Control Cluster Assembly (RCCA) bank typically results in a reactivity insertion greater than 5E-3 delta K/K which will be detected by the power range negative neutron flux rate trip circuitry. The reactor is tripped within approximately 2.5 seconds following the drop of a RCCA bank. The core is not adversely affected during this period, since power is decreasing rapidly.

#### E. CORRECTIVE ACTIONS:

The immediate corrective actions were to establish stable plant conditions following the reactor trip.

The fuses in cabinet 18D were replaced and the system line-up was returned to normal.

Action to prevent recurrence include:

- 1. This event has been reviewed with the individuals involved.
- Additional administrative controls will be developed to aid the operator in identifying and maintaining off-normal equipment status. I've controls being considered are fabrication of a movable guard and additional guidance on pre-job briefing and communication requirements. This will be tracked to completion by Action Item 456-200-88-18201.
- This report will be included in the Licensed Operator Required Reading Program. This will be tracked by Action Item 456-200-88-18202.

## F. PREVIOUS OCCURRENCES:

There have been eight previous reactor trips due to personnel error. In each case corrective actions were implemented addressing both root and contributing causes. However, the cognitive personnel error in this event is different in that it involved a conditioned response to an alarm. Previous corrective actions are not applicable to this event.

### G. COMPONENT FAILURE DATA:

Manufacturer	Nomenclature	Model Number	MFG Part Number
Chase-Shwmut	Fuse	Type 1 30 amp	A60X30



Commonwealth Edison Braidwood Nuclear Power Station Route #1, Box 84 Braceville, Illinois 60407 Telephone 815/458-2801

BW/88-1022

August 29, 1988

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Dear Sir:

The enclosed Licensee Event Report from Braidwood Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2) (iv) which requires a 30 day written report.

This report is number 88-016-00; Docket No. 50-456.

Very truly yours,

Herenio

R. E. Querio Station Manager Braidwood Nuclear Station

REQ/AJS/jab (7126z)

Enclosure: Licensee Event Report No. 88-016-00

cc: NRC Region III Administrator NRC Resident Inspector INPO Record Center CECo Distribution List

1522

1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1		DEVIATION REPORT				
'		DVR NO. 20 - 01 - 88 STA UNIT YEA				
PART 1   TITLE OF DE Rx Trip Due to Negati	VIATION ve Rate Trip			OCCURRED	04TE TIM	6 E
SYSTEM AFFECTED	PLANT STATU	S AT TIME OF EVENT	WORK	N/A REQUEST NO.	TESTING	1_x_1N0

# DESCRIPTION OF EVENT

At 1316, 8-11-88, Unit 1 Rx tripped. The first out annunciator was NIS power range neutron flux regative rate reactor trip. The negative rate trip occurred due to Control Bank D Group 1, being dropped. The control rods were dropped when movable gripper coil, in power cabinet 18D, bus disconnect fuses were being replaced and simultaneous rod movement occurred. The Reactor Trip with Lo Tave 514°F caused FW icolation and subsequent S/G Lo Lo level caused AFW actuation.

POTENTIALLY SIGNIFI	CANT EVENT PER NSD DIRECTIVE A-07	1I YES 1I	10
10CFR50.72 NRC RED PHONE	11 HOUR	J. Kuchenbecker	8-11-88
NOTIFICATION MADE	1_X_1 4 HOOK 1	RESPONSIBLE SUPERVISOR	DATE

None

I       I       NON REPORTABLE EVENT         I       30 DAY REPORTABLE/10CFR50.73(a)(2)(iv)         I       5 DAY REPORT PER 10CFR21         I       I         I	NOTIFICATION Resident Inspector REGION III I.J. Maiman/D.P. Galle NSD CECO CORPORATE NOTIFICATION I IF ABOVE NOTIFICATION IS F	8/12/88 DATE 8/12/88 DATE IN MADE PER 10CFR21	1200 TIME 1200 TIME
L.E.R. #_8_80_1_6	TELECOPYN/A CECO CORPORATE OFFICER	DATE	TIME
PRELIMINARY REPORT COMPLETED AND REVIEWED Robert - OPERATIN	), Ungeran		
INVESTIGATION REPORT & RESOLUTION ACCEPTED BY STATION REVIEW	Miller erevere St. d Chartie 97 STATION MANAGER	- stailse 11/38 DATE	
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