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MAY 05 1988

Docket No. 50-395
License No. NFF-12

South Carolina Electric and Gas Company
ATTN: Mr. D. A. Nauman, Vice President
Nuclear Operations (Mail Code 6ul)
P. O. Box 88
Jenkinsville, SC 29065

Gentlemen:

SUBJECT: NRC INSPECTION REPORT NO. 50-395/88-06

This refers to the Nuclear Regulatory Commission (NRC) inspection conducted by W. M. Sartor on March 15-18, 1988. The inspection included a review of activities authorized for your V. C. Summer facility. At the conclusion of the inspection, the findings were discussed with those members of your staff identified in the enclosed inspection report.

Areas examined during the inspection are identified in the report. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observation of activities in progress.

Within the scope of the inspection, no violations or deviations were identified. However, there were areas identified as requiring further review and evaluation. These items are identified as Inspector Followup Items in Appendix A to this letter, and are also discussed in the enclosed appraisal report.

You are requested to submit a written statement within 45 days of the date of this letter, which details the action you plan to take, and the estimated date of completion with regard to the findings identified in Appendix A.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and its enclosure will be placed in the NRC Public Document Room.

The responses directed by this letter and its enclosure are not subject to the clearance procedures of the Office of Management and Budget as required by the Paperwork Reduction Act of 1980, Pub. L. No. 96-511.

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Should you have any questions concerning this letter, please contact us.

Sincerely,

Douglas M. Collins, Chief
Emergency Preparedness and
Radiological Protection Branch
Division of Radiation Safety
and Safeguards

Enclosure:
Inspection Report

- cc w/encl:
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5/3

NRR
By Ed Williams, telephone
4/29/88
BTravers
4/29/88

APPENDIX A

Appraisal Open Items

1. Review the specifications and documentation on the steam line monitors to determine if the monitors can be made effective in providing dose assessment data at lower than PAG limits (IFI 50-395/88-06-01).
2. Locate all available documentation pertaining to validation, verification, and methodology for all dose assessment models (computer and manual) and centralize the maintenance of it into the Plant Record System (PRS) (IFI 50-395/88-06-02).
3. Establish a periodic calculational comparison between dose assessment models (e.g., EARS, EPP-005, State, NRC) and document the reasons for significant differences (IFI 50-395/88-06-03).