



UNION ELECTRIC COMPANY

1901 Gratiot Street, St. Louis

Donald F. Schnell
Vice President

February 24, 1986

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

ULNRC-1263

DOCKET NUMBER 50-483
CALLAWAY PLANT
RELOAD LICENSE AMENDMENT REQUEST
ADDITIONAL INFORMATION

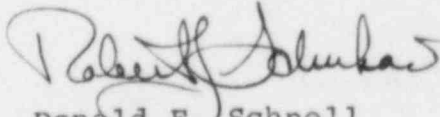
- References:
- 1) ULNRC-1207 dated 11/15/85
 - 2) ULNRC-1227 dated 12/13/85
 - 3) ULNRC-1247 dated 1/28/86
 - 4) ULNRC-1258 dated 2/18/86

The referenced letters transmitted the reload license amendment request for Callaway Cycle 2 along with additional information to support this request. The attachment to this letter documents additional information provided in a follow-up phone call on February 20, 1986, involving NRC, Westinghouse and Union Electric.

In response to your question as to the proprietary nature of Reference 4, we have received Westinghouse concurrence that Table 1 from this reference is not proprietary and may be used in the NRC Safety Evaluation Report.

If there are further questions, please contact us.

Very truly yours,

for 
Donald F. Schnell

DS/kc

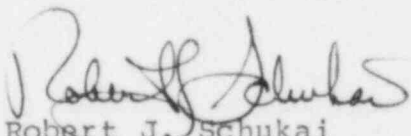
Attachment

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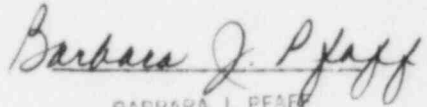
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STATE OF MISSOURI)
) S S
CITY OF ST. LOUIS)

Robert J. Schukai, of lawful age, being first duly sworn upon oath says that he is General Manager-Engineering (Nuclear) for Union Electric Company; that he has read the foregoing document and knows the content thereof; that he has executed the same for and on behalf of said company with full power and authority to do so; and that the facts therein stated are true and correct to the best of his knowledge, information and belief

By 
Robert J. Schukai
General Manager-Engineering
Nuclear

SUBSCRIBED and sworn to before me this *24th* day of *February*, 1986.


BARBARA J. PFAFF
NOTARY PUBLIC, STATE OF MISSOURI
MY COMMISSION EXPIRES APRIL 22, 1989
ST. LOUIS COUNTY

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ADDITIONAL INFORMATION FOR CYCLE 2
RELOAD LICENSE AMENDMENT REQUEST

1. The latest Callaway precision flow calorimetric done in December, 1984, showed a total RCS flow of 4.11×10^5 gpm. Recent computer flow calorimetrics indicate this has not degraded since the December, 1984, calorimetric.
2. Average coolant velocity for the fuel rods
Assuming full OFA core - 15.4 ft/sec
Assuming full LOPAR core - 16.3 ft/sec
3. Minimum calculated DNBR during steamline break transient (based on the W-3 DNBR correlation and standard thermal hydraulic design procedures) is greater than or equal to 1.80 for both typical and thimble flow channels.