



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA  
DOCKET NO. 50-321  
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 153  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated December 21, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 153, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 2, 1988

PD#II-3/DRP-I/II  
MRood  
03/27/88

AD/DRP I/II  
GLainas  
03/28/88

*MC*  
PD#II-3/DRP-I/II  
LCrocker:sw  
03/28/88

*Am*  
NRR:MEB  
TMarsh  
02/17/88  
4/12

*JEM*  
OGC-WFlint  
4/15/88  
03/1/88

*DM*  
PD#II-3/DRP-I/II  
DMatthews  
03/1/88  
4/27

ATTACHMENT TO LICENSE AMENDMENT NO. 153

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

Remove  
Page

3.7-14

Insert  
Page

3.7-14

4.7.D.1. Surveillance of Operable Valves (Continued)

- b. At least once per operating cycle the reactor coolant system instrument line excess flow check valves shall be tested for proper operation.
- c. At least once per quarter, all normally open, power-operated isolation valves (except for the main steam line power-operated isolation valves) shall be fully closed and reopened.
- d. The isolation time of each main steam line isolation valve, shall be determined to be within its limit when tested, pursuant to Specification 4.6.K.
- e. At least once per week the main steam line power-operated isolation valves shall be exercised one at a time by partial closure and subsequent reopening.

3.7.D.2. Operation with Inoperable Valves

In the event any isolation valve specified in Table 3.7-1 becomes inoperable, reactor power operation may continue provided at least one isolation valve in each line having an inoperable valve is in the mode corresponding to the isolated condition.

3. Shutdown Requirements

If Specification 3.7.D.1. and 3.7.D.2. cannot be met, an orderly shutdown shall be initiated and the reactor shall be placed in the Cold Shutdown Condition within 24 hours.

2. Surveillance of Lines with an Inoperable Valve

Whenever an isolation valve listed in Table 3.7-1 is inoperable the position of at least one other isolation valve in each line having an inoperable isolation valve shall be verified to be in its isolated position daily.