

AUG 30 1988

Docket No. 50-333

Power Authority of the State of New York
James A. FitzPatrick Nuclear Power Plant
ATTN: Mr. Radford J. Converse
Resident Manager
P. O. Box 41
Lycoming, New York 13093

Gentlemen:

Subject: Inspection No. 50-333/88-07

This refers to your letter dated August 8, 1988, in response to our letter dated July 7, 1988.

Thank you for informing us of the corrective and preventive actions documented in your letter. These actions will be examined during a future inspection of your licensed program.

Your cooperation with us is appreciated.

Sincerely,

Original Signed By:
Ronald R. Bellamy

Ronald R. Bellamy, Chief
Facility Radiological Safety
and Safeguards Branch
Division of Radiation Safety
and Safeguards

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AUG 30 1988

cc:

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A. Klausmann, Senior Vice President - Appraisal and Compliance Services
R. L. Patch, Quality Assurance Superintendent
George M. Wilverding, Manager Nuclear Safety Evaluation
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R. Burns, Vice President Nuclear Operation
Dept. of Public Service, State of New York
State of New York, Department of Law
Licensing Project Manager, NRR
Public Document Room (PDR)
Local Public Document Room (LPDR)
Nuclear Safety Information Center (NSIC)
NRC Resident Inspector
State of New York
FEMA II

bcc:

Region I Docket Room (with concurrences)
Management Assistant, DRMA (w/o encl)
Section Chief, DRP
Robert J. Bores, DRSS
B. Clayton, EDO
W. Travers, NRR/PEPB

RI:DRSS
Fox/mk

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Bellamy

8/26/88

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08/15/88



Radford J. Converse
Resident Manager

August 8, 1988
JAFF-88-0744

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Mail Stop P1-137
Washington, D.C. 20555

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333
EMERGENCY RESPONSE FACILITIES INSPECTION NO. 88-07
RESPONSE TO OPEN ITEMS

REFERENCE: 1. NRC letter, R. R. Bellamy to R. J. Converse
dated July 7, 1988 regarding NRC inspection
50-333/88-07, Emergency Response Facilities

Dear Sir:

The NRC transmitted inspection Report No. 50-333/88-07 as an attachment to Reference 1. This inspection report summarized the results of an NRC inspection of the FitzPatrick Emergency Response Facilities (ERF) during the week of May 16-20, 1988.

The report identified three open items. This letter describes the Authority's corrective action along with an implementation schedule for each.

The three open items identified are restated below:

1. Dose calculation procedures do not contain a method for calculating offsite doses resulting from a real time accident in which the source term is leakage through the drywell boundary and whether any other credible, unmonitored accident release pathways exist for which similar methods should be developed (50-333/88-05-01).
2. The validation and verification documentation for the Class A Dose Assessment Model is incomplete (50-333/88-05-02).
3. There is no formalized method for controlling and documenting the maintenance of the dose assessment model (i.e. MMRAS and IDAC) (50-333/88-05-03).

TO: U.S.N.R.C.
FROM: R. CONVERSE
SUBJECT: INSPECTION NO. 88-07

August 8, 1988
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Response to Open Items (Corrective Action and Schedule)

1. A plant procedure will be developed for determining radiological source terms for use in dose calculations based on drywell radiation monitor readings and the maximum allowable containment leak rate as specified in the FitzPatrick Technical Specifications. This procedure will also include steps to identify credible, unmonitored accident release pathways.

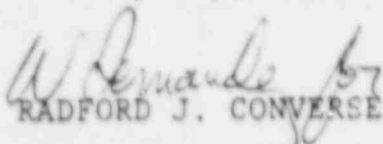
This procedure will be implemented by January, 1989.

2. A Validation and Verification program will be developed for the computer programs that are part of the FitzPatrick Meteorological Monitoring and Radiological Dose Assessment System (MMRAS). A methodology similar to that used to validate the FitzPatrick Offsite Dose Calculation Manual (ODCM) will be used. The validation and verification methodology will be documented in a program report. The program will also develop and document a detailed test plan.

This program will be complete by May, 1989.

3. A plant procedure for the control and maintenance of computer software is currently under development. This procedure will be implemented to control revisions to both MMRAS and IDAC (Interim Dose Assessment System) computer programs.

This procedure will be implemented for MMRAS and IDAC by May, 1989.


RADFORD J. CONVERSE

RJC/AHZ/mam

CC: H. Abelson (U.S.N.R.C.)
W. Russell (U.S.N.R.C. Region I)
A. Luptak (U.S.N.R.C. Resident Inspector)
W. Fernandez
D. Lindsey
E. Mulcahy
V. Walz
M. Prarie
D. Ackley
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J. Kelly (WPO)
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