

U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Report No. 50-341/OL-88-01

Docket No. 50-341

License No. NPF-43

Licensee: The Detroit Edison Company
6400 North Dixie Highway
Newport, MI 48166

Facility Name: Fermi 2 Nuclear Plant

Examination Administered At: Fermi 2 Nuclear Plant, Michigan

Examination Conducted: August 2, 1988

Chief Examiner:

G. M. Nejeft
G. M. Nejeft, Principal
Examiner, Operator Licensing
Section No. 1

August 31, 1988
Date

Approved By:

M. J. Jordan
Michael Jordan, Chief
Operator Licensing Section No. 1

8/31/88
Date

Examination Summary

Examination administered on August 2, 1988 (Report No 50-341/OL-88-01)

One written examination was given to a reactor operator (RO) candidate as a retake.

Results: The candidate successfully passed this examination.

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REPORT DETAILS

1. Examiners

G. Nejfelt, Chief Examiner

2. Exit Meeting

At the conclusion of the site visit, the examiner met with facility training staff. The following personnel attended this exit meeting.

Facility Representatives

G. R. Overbeck, Director, Nuclear Training

R. W. Bovinet, Work Leader, Operator Training Program

NRC Representative

G. M. Nejfelt, Chief Examiner

The following items are discussed during the exit meeting on August 3, 1988:

- a. Timely submission of examination comments from the facility training staff. These comments were received in the Regional Office on August 19, 1988.
- b. Materials that were prepared for the tentatively scheduled Requalification Examinations starting the week of October 31, 1988 were discussed. Most of the written examination questions were found to be satisfactory, except for static simulator questions that could be answered without the use of the simulator. No Job Performance Measures (JPMs) were available for review.

3. Examination Review

Responses to licensee's comments, concerning the written RO examination, are provided in Attachment 1. Minor changes made without facility comments by the examiner are provided in Attachment 2. Also, note that six comments were made concerning the examination materials provided (see Attachment 1 NRC Response to Questions 2.06.a, 2.07, 2.10.a, 3.02, 3.10, and 4.03).

Attachments:

1. RO Examination Comments
and Resolutions
2. RO Answer Key Modification
Without Facility Comment

ATTACHMENT 1

REACTOR OPERATOR (RO)

EXAMINATION COMMENTS AND RESOLUTIONS

FERMI 2 RO EXAMINATION OF AUGUST 2, 1988

The following represents the facility comments and the NRC resolution to those comments made as a result of the current examination review policy.

1.05.c

Facility Comment: Comment answer for Part (c) is 518 seconds . . . a math error was made in the answer key.

NRC Resolution: Comment is accepted. The answer key is changed from "318 seconds" to "518 seconds" to correct the typographical error.

2.01

Facility Comment: Correct answer for Part (c) of 2.01 should be 1153 +0/-20 psig and 441 +20/-0 psig. This licensing class was taught allowable values as listed in Technical Specifications.

References:

Reactor Recirculation System Handout, Page 18
Fermi 2 Technical Specification (TS),
Section 3.3.3
Fermi 2 TS, Section 3.3.4
Pump Trip System Instrumentation

NRC Resolution: Comment is accepted, since the allowable maximum value agrees with the setpoint value and is more conservative than the setpoint value. The setpoint values of 1133 psig and 441 have been changed to the allowable values and tolerances provided above.

2.02.a

Facility Comment: In Part (a), the answer states that the backup air supply for the inboard main steam isolation valves (MSIVs) and the normal air supply for the outboard MISVs is instrument air. This answer could be misleading. The term used at Fermi 2 for this instrument air is called Control Air. Specifically, INTERRUPTIBLE CONTROL AIR (IAS).

References:

Student Handout, Main Steam System and Bypass System, Page 14.
Drawing 6m721-5730-5.

NRC Resolution:

Comment is accepted. The use of the alternate names for instrument air (e.g., control air and Interruptible Control Air) are in addition to the answer as acceptable alternate wording.

2.02.b

Facility Comment:

Part (b) answer should be "130 VDC and 120 VAC (RPS M-G Set)."

Reference:

Student Handout, Main Steam System and Bypass System, Page 14.

NRC Resolution:

Comment is accepted. Typographical error is changed from "125 VDC . . ." to "130 VDC . . ."

2.02.d

Facility Comment:

In Answer (d), the statement is made that the isolation will not reset. Assuming the isolation signal itself is clear pushing the RESET pushbutton without first pushing the close pushbuttons on the MISVs will cause the MISVs to open.

References:

Student Handout, Main Steam and Bypass, Page 14.
LER 50-341/87-037.
61721-2095-14, 15, 17, and 18.

NRC Resolution:

Comment is accepted, only if, the candidate stated that the solenoids for the MISV were deenergized and reenergized in an unorthodox manner (e.g., LER 50-341/87-037).

2.06.a

Facility Comment:

On Part (a), the answer key should be changed to accept the power supplies to the RPS M-G Sets as correct actions for normal supplies. These would be:

1. DIV I (RPS M-G Set A) - 480 VAC Motor Control Center (MCC) 72B-4C Pos.
2. DIV II (RPS M-G Set B) - 480 VAC MCC 72E-5B Pos 1C-R.

References:

Nuc. Prod. Sys. Operati Proc. 23.316,
Pages 8 and 9.
DECo Drawings 6I721-2151-1 and 2.

NRC Resolution:

Comment is accepted for both the primary and alternate RPS power supplies, if the candidate's answer is in the same detail. Student Handout, RPS, PIC-C71, Revision 3, Page 4, substantiate the answer key as originally written.

2.06.bFacility Comment:

Part b should have four answers:

1. Process Radiation Monitoring System.
2. Power Range Monitoring System.
3. Nuclear Steam Supply Shutoff System.
4. Reactor Protection System.

Reference:

DECo Drawing 6I721-2151-1 and 2.

NRC Resolution:

Comment is not accepted. The question asked for systems that received power from RPS. Also, if schematics are provided as supporting material, please ensure that the information is large enough and clear enough to be used without a tedious effort to extract the information.

2.07Facility Comment:

Using the reference given on Pages 8 and 14, there is no mention of High Reactor Pressure causing a Standby Gas Treatment (SBGT) System auto start. There is a misprint on Page 14 that may lead one to believe Part (a), High Radiation (ARM) on the Refuel floor, may be a correct response. 4.a.4 on Page 14 should be Fuel Pool Ventilation Exhaust Radiation High as shown by Page 8 of the SBGT system, Student Handout and Alarm Response Procedure 17D14. Therefore, the answer key should read:

will: b, g

will not: a, c, d, e, f, h

References:

Standby Gas Treatment System Student Handout,
Pages 8 and 14.

Alarm Response Procedure 17D14.

NRC Resolution: Comment is accepted and the answer key modified as above. Revision of Student Handout, SBT, T46-00, Revision 4, Page 14, is needed.

2.08

Facility Comment: Even though not listed in the given reference, the word chugging is often used to describe the condition of injecting the boron too fast. So an alternative phrase at the end of the answer would also be: Could Cause Power Chugging.

NRC Resolution: Comment is accepted. The word "chugging" is added to the answer key as an alternative wording.

2.10.a

Facility Comment: To be consistent with Question 2.01, the answer for (a) should be 441 +20/-0 psig.

NRC Resolution: Comment is accepted. The answer key is changed to the allowable value and tolerances provided above.

Note that the asterisk used in Table 4 of Student Handout, RHR, Revision 4, Page 48 is not defined.

2.10.c

Facility Comment: Part (c) should read:

F015A/B: Remove initiation signal and push Initiation Signal Reset Push Button and Leak Detection Line Break Push Button.

F017A/B: Five minutes after initiation signal is received.

Reference:

HRH Student Handout, Pages 32, 33, and 48.

NRC Resolution:

Comments is not accepted. References did not support revising answer key.

2.10.dFacility Comment:

Part (d) should be changed to read:

F017A/B can be closed (throttled) to control injection flow (Reactor Level).

This is because the push button for closure of the valve (F017A/B) will allow full closure if pushed long enough. Also, the way the question is worded (close inhibit) would lead one to think along the lines of open/close not throttled.

References:

RHR Student Handout, Pages 32, 33, and 48.

Fermi 2 Technical Specification 3.3.3 Table 3.3.3-2.

NRC Resolution:

Comment does not warrant changing answer key, since "throttling" can be considered synonymous with "closing" for this question.

3.02Facility Comment:

Using the Student Handout on RWM referenced, Fermi 2 Technical Specifications and Proposed Design Change (PDC) 7030 it is possible to answer Part "a" several different ways. The first way would be a word definition as stated in the answer key. The other way would be with specific numbers. Per the Student Text and Fermi 2 Technical Specification the MINIMUM allowable reset power level is 20% for the LPSP. The actual per PDC 7030 is 30%. the LPAP per the Student Handout is 35%. But per PDC 7030, the actual value set in the Control Room is 40%. Therefore, a suggested addition to the provided answer would be:

- a. (1) . . . similar. Also accept $25 \pm 5\%$.
- b. (2) . . . similar. Also accept 35% to 40%.

References:

Rod Worth Minimizer Student Handout, Page 4.
 Fermi 2 Technical Specifications 3.1.4.1 Rod
 Worth Minimizer
 Proposed Design Change (PDC) 7030, Revision B.

NRC Resolution:

Comment is partially accepted. The question elicited more than a numerical values for Low Power Point (LPSP) and Low Power Alarm Point (LPAP). Partial credit will be given for range of values stated above.

Training material is needed to be revised with plant changes.

3.04.cFacility Comment:

Using the referenced Automatic Depressurization System Handout, Figure 2 and Detroit Edison Drawings 61721-2095-02 and 07 the correct answer for "c" should be ADS valves remain as is. The reason is that BOTH Core Spray Pumps in Division II must be running to satisfy the ADS logic.

Reference:

Automatic Depressurization Handout, Pages 7, 8, and Figure 2.
 Detroit Edison Drawings 61721-2095-02 and 07.

NRC Resolution:

Comment is not accepted. With no 130 VDC, the solenoids for ADS valves will deenergize and close valves (e.g., remove fuses to shut stuck open SRV).

3.08Facility Comment:

Using the Power Range Monitor and Technical Specification reference plus Technical Specification Page 3/4 3-45 there are three parts of the answer that needs to be expounded.

1. For the information provided, Answer "c" could be YES or NO. True eight inops LPRM inputs to CHANNEL D APRM will leave it with 14 operable inputs (minimum required to be operable). The questions of how many operable LPRM inputs per level is not addressed. Therefore since eight LPRM's are inop, it could be assumed there are < 2 operable per level which would make that APRM inop per Technical Specification.

Regarding Answers "b" and "f." Technical Specifications Page 3/4 3-44 lists the allowable value of Reactor Coolant System Recirculation Flow Comparator as \leq 11% flow deviation. Which is what the Student Handout on Power Range Monitor Page 13 used. The \leq means that value has been set at or below this value per Tech Specs. So if the actuation is set at 11% (the Tech Spec allowable value class was instructed to memorize), there would not be a rod block flow reference off normal. Therefore Answers 6 and 7 could be NO.

References:

Power Range Monitor Student Handout Page 13
Technical Specification Pages 3/4 3-5 and 3/4 3-44.

NRC Resolution:

For Item "c," your comment is accepted. Part "c" is deleted because of the question ambiguity; and the question point value is reduced from 3.00 points to 2.50 points.

For Items "b" and "f" your comment is not accepted. As stated above, the greater than or equal to 11% difference is signal between Flow Unit "A" and Flow Unit "B" was memorized by the licensing class. Unless the candidate clearly stated or assumed flow deviation less than 11%, no change to the answer key is made.

3.10

Facility Comment:

Per the Recirculation Flow Control System Pages 10, 11, and 12, there are three flow limiters. No. 1 limiter first in line before the individual pump controller and Limiter 2 and 3 are after the Error Limiting Network and Individual Speed Controllers.

Their setpoints per Proposed Design Change 8294 are:

Limiter No. 1 - 30%
Limiter No. 2 - 42%
Limiter No. 3 - 48%

The individual pump controllers do not have any limits on them. The limiters are separate units. So Answer "a" should be as above. Also, Answer "b" for limiter No. 1 should be . . . not fully open OR feedwater . . .

References:

Recirculation Flow Control Student Handout,
Pages 10 through 12.
Detroit Edison Drawing 6I721-2105-7.
Proposed Design Change 8294.

NRC Resolution:

For Part (a), comment is accepted. The answer key is revised to change the setpoint values and to added Limiter No. 3 based on Potential Design Change (PDC) No. 8294 that superseded Student Handout, Recirculation Flow Control System (B31), Revision 4, which was used to prepare the question. Also, the point value for Part (a) was reduced from 1.00 point to 0.75 point with each limiter setpoint with 0.25 point.

For Part (b), comment is accepted based upon Schematic Drawing, 6I721-2105-7, Revision 01; and the answer key is changed from ". . . and . . ." to ". . . or . . ." with either answer worth 1.00 point. Student Handout Recirculation Flow Control System (B31), Revision 4, Section D.4.a(a) is in error.

3.11Facility Comment:

Using the Rod Sequence Control System Student Handout Pages 4 and 6. The answer in "2" would more correctly be: "Neither Sequence Groups B 1-2 and B 3-4 Control Rods are fully out, and."

The answer is more complete because it recognizes that A and B Sequences are broken into Groups A 1-2, A 3-4, B 1-2, and B 3-4 on the Rod Sequence Select Switch.

Reference:

Rod Sequence Control System Student Handout,
Pages 4 and 6.

NRC Resolution:

Comment is not accepted because it is moot. Nothing is gained by specifying the subgroups for the "A" and "B" Rod Control Sequence Groups.

4.01Facility Comment:

The answer in the above question was not specific. Therefore, answers should also include:

1. During plant startup (when RHR shutdown cooling is secured).
2. During plant shutdown (prior to RHR shutdown cooling being started).
3. During a loss of RHR shutdown (when RHR is not able to be restored)

References:

General Operating Procedures 22.000.03, Page 9
and 22.000.10, Page 11.
Abnormal Operating Procedures 20.205.01, Page 1.

NRC Resolution:

Comment is accepted. The question could be construed for plant situations when operation of recirculation pumps may be required to operate. The answer and key point distribution are revised to read:

1. Preventing temperature stratification or in the Reactor Vessel (0.50).
2. Retaining solids in suspension until they can be removed (by the RWCU System to prevent their deposition in the bottom of the Reactor Vessel of the CRD mechanisms) (0.50).
3. During plant startup (0.25), when RHR shutdown cooling is secured (0.25).
4. During plant shutdown (0.25), prior to RHR shutdown cooling being started (0.25).
5. During a loss of RHR shutdown cooling (when RHR cannot be restored) (0.50).

(Note: Only two of the items above are required for full credit.)

4.03Facility Comment:

Although not listed as an indication in Section 3.0 of AOP 20.106.04, there is another indication listed in Step 2.1.1. This step has the operator check LRPM (POWER) as the Rod is moved to verify the rod is recoupled. This method could also be used prior to the rod reaching Position 48 by looking for no LPRM change during rod movement.

Reference:

AOP 20.106-04, Page 3.

NRC Resolution:

Comment is partially accepted. POM 20.106.104, Revision 4, Section 2.1.1 must include the condition of Section 2.1.2. Therefore, the answer key is revised to read:

- "4. Observe a response to the rod movement through Nuclear Instrumentation (NI) response (0.25).
and
Demonstrates that the Control Rod will not go to the withdraw overtravel position (0.25).
(Any three of the above for full credit)."

Note: Use of the NI response as an indication should be added to Section 3.0 of POM 20.106.04.

4.11Facility Comment:

Using the reference (AOP 20.710.01 Pages 1 and 2) the answer key should be expanded to include:

If Reactor Building Vent Exhaust Radiation Monitor upscale trip occur, verify the following automatic action occur:

1. Reactor Building Ventilation System tripped.
2. Reactor Building Divisions I and II Supply and Exhaust isolation valves close.
3. Pri Containment Purge and Vent Valves close.
4. SBT System Auto Starts.
5. CCHVAC System aligns to Emergency Recirculation Mode.

Notify the NSS of the event, actions taken, and that it may be required to classify the event in accordance with Emergency Plan Implementing Procedure EP 101, "Classification of Emergencies."

Reference:

AOP 20.710.01 Refueling Floor High Radiation,
Revision 5, Pages 1 and 2.

NRC Resolution:

Comment is partially accepted. The notification of the Nuclear Shift Supervisor (NSS) is added to the answer key as an alternative answer. However, the question asked for immediate operator actions for the "Refueling Flow High Radiation alarm." Therefore no credit is given for the "Reactor Building Vent Exhaust Rod Monitor" upscale trip immediate operator actions.

4.13

Facility Comment:

Per Tech Spec 1.36, Page 1-6 O-Rings, Bellows, and Welds should be acceptable for "Sealing Mechanism."

Reference:

Tech Spec 1.36, Page 1-6.

NRC Resolution:

Comment is acceptable. Answer key Item 5 is revised with these examples to read:

"5. The sealing mechanism associated with each secondary containment penetration (e.g., welds, bellows, or O-rings) is operable."

ATTACHMENT 2

FERMI 2

REACTOR OPERATOR EXAMINATION

AUGUST 2, 1988

- CHANGES MADE TO ANSWER KEY WITHOUT PROMPTING FROM FACILITY -

<u>Question Number</u>	<u>Comment</u>
1.11	Answer key asterisk at point (0, 1) is deleted, as a typographical error. Also, the range for the allowable value on the x-axis is increased from "34-36" to "32-38," because the graphical accuracy intended would not be achieved without the use of graph paper and straight edge ruler.
1.12	Point value for Question 1.12 was erroneous given as "2.00 points." The point value for this question has been changed to "1.50 points" to agree with the answer key.
1.13	Point values redistributed between Parts (a) and (b) to reflect effort needed in response. Part (a) value was changed from "1.00 point" to "1.50 points" and Part (b) value was changed for "1.00 point" to "0.50 point."

ENCLOSURE (2)

U. S. NUCLEAR REGULATORY COMMISSION
REACTOR OPERATOR LICENSE EXAMINATION

FACILITY: FERMI_2
 REACTOR TYPE: BWR-GE4
 DATE ADMINSTERED: 88/08/02
 EXAMINER: E. A. HARE
 CANDIDATE: _____

INSTRUCTIONS TO CANDIDATE:

Use separate paper for the answers. Write answers on one side only. Staple question sheet on top of the answer sheets. Points for each question are indicated in parentheses after the question. The passing grade requires at least 70% in each category and a final grade of at least 80%. Examination papers will be picked up six (6) hours after the examination starts.

CATEGORY	% OF TOTAL	CANDIDATE'S SCORE	% OF CATEGORY VALUE	CATEGORY
25.48	25.95			
25.28	25.86			
	25.88			1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW
24.50	24.88			
24.10	24.88			2. PLANT DESIGN INCLUDING SAFETY AND EMERGENCY SYSTEMS
23.98	24.30	24.55		
24.14	24.27			
24.00	24.87			3. INSTRUMENTS AND CONTROLS
24.24	24.75			
24.50	24.20			4. PROCEDURES - NORMAL, ABNORMAL, EMERGENCY AND RADIOLOGICAL CONTROL
98.47	24.38			
98.97	24.72			
100.1				Totals
98.72				%
		Final Grade		

All work done on this examination is my own. I have neither given nor received aid.

Candidate's Signature

MASTER COPY

NRC RULES AND GUIDELINES FOR LICENSE EXAMINATIONS

During the administration of this examination the following rules apply:

1. Cheating on the examination means an automatic denial of your application and could result in more severe penalties.
2. Restroom trips are to be limited and only one candidate at a time may leave. You must avoid all contacts with anyone outside the examination room to avoid even the appearance or possibility of cheating.
3. Use black ink or dark pencil only to facilitate legible reproductions.
4. Print your name in the blank provided on the cover sheet of the examination.
5. Fill in the date on the cover sheet of the examination (if necessary).
6. Use only the paper provided for answers.
7. Print your name in the upper right-hand corner of the first page of each section of the answer sheet.
8. Consecutively number each answer sheet, write "End of Category __" as appropriate, start each category on a new page, write only on one side of the paper, and write "Last Page" on the last answer sheet.
9. Number each answer as to category and number, for example, 1.4, 6.3.
10. Skip at least three lines between each answer.
11. Separate answer sheets from pad and place finished answer sheets face down on your desk or table.
12. Use abbreviations only if they are commonly used in facility literature.
13. The point value for each question is indicated in parentheses after the question and can be used as a guide for the depth of answer required.
14. Show all calculations, methods, or assumptions used to obtain an answer to mathematical problems whether indicated in the question or not.
15. Partial credit may be given. Therefore, ANSWER ALL PARTS OF THE QUESTION AND DO NOT LEAVE ANY ANSWER BLANK.
16. If parts of the examination are not clear as to intent, ask questions of the examiner only.
17. You must sign the statement on the cover sheet that indicates that the work is your own and you have not received or been given assistance in completing the examination. This must be done after the examination has been completed.

18. When you complete your examination, you shall:

a. Assemble your examination as follows:

(1) Exam questions on top.

(2) Exam aids - figures, tables, etc.

(3) Answer pages including figures which are part of the answer.

b. Turn in your copy of the examination and all pages used to answer the examination questions.

c. Turn in all scrap paper and the balance of the paper that you did not use for answering the questions.

d. Leave the examination area, as defined by the examiner. If after leaving, you are found in this area while the examination is still in progress, your license may be denied or revoked.

QUESTION 1.01 (1.50)

Increasing recirculation pump speed will cause WHAT change (INCREASE, DECREASE, or REMAIN THE SAME) in each of the following parameters? (Assume normal operating conditions) (1.5)

- a. actual bundle power
- b. critical power
- c. critical power ratio

QUESTION 1.02 (1.98)

Match each of the power distribution limits [1, 2, & 3] with its associated FAILURE MECHANISM in Column A and its associated LIMITING CONDITION in Column B. (2.0)

1. Linear Heat Generation Rate (LHGR)
2. Average Planer Linear Heat Generation Rate (APLHGR)
3. Minimum Critical Power Ratio (MCPR)

Column A FAILURE MECHANISM	Column B LIMITING CONDITION
A1. FUEL CLAD CRACKING DUE TO LACK OF COOLING	B1. 1% PLASTIC STRAIN
A2. FUEL CLAD CRACKING DUE TO HIGH STRESS FROM PELLET EXPANSION	B2. PREVENT TRANSITION BOILING
A3. GROSS CLAD FAILURE DUE TO DECAY HEAT AND STORED HEAT FOLLOWING A LOCA	B3. LIMIT CLAD TEMP TO 2200 F

QUESTION 1.03 (1.00)

Using the steam tables, calculate a reactor cooldown rate (F/hr) for a reactor pressure decrease from 1000 psig to 250 psig in one hour and forty five minutes (105 minutes total) SHOW ALL WORK for full credit.

QUESTION 1.04 (2.50)

For each of the following events, state which COEFFICIENT of reactivity (fuel temperature, moderator temperature, void) would act FIRST to change reactivity.

- a. Control rod drop at power (0.5)
- b. SRV opening at power (0.5)
- c. Loss of shutdown cooling when shutdown (0.5)
- d. One recirc pump trips while at 50% power (0.5)
- e. Loss of one feedwater heater at 100% power (extraction steam isolated) (0.5)

QUESTION 1.05 (2.50)

~~The reactor is brought critical at 40% on IRM range 2 with the shortest permissible stable positive period allowed by GCP 22.000.03 "Startup From Cold Shutdown to Rated Power". Reactor power is determined to be 40% on range 0 of IRM. SHOW ALL WORK.~~

Answer each of the following questions, concerning reactor period

- a. What is the shortest permissible stable reactor period, as stated in the caution of POM 22.000.03, STARTUP FROM COLD SHUTDOWN TO RATED POWER? (0.5)
- b. What is the doubling time for a constant reactor period of 2 minutes? (0.5).
(i.e., 40% on Range 8 of IRM)
- c. How long will it take for power to reach the point of adding heat[^] if a period of 75 seconds is maintained. (1.5)

With reactor power initially determined to be 40% on Range 2 of IRM.

QUESTION 1.06 (2.00)

ANSWER each of the following questions given that the reactor is at 100% power and 1000 psig, when a relief valve inadvertently opens.

- a. STATE the tailpipe temperature, assuming atmospheric pressure in the Suppression Pool and No Reactor Depressurization (0.5)
- b. If the Suppression Pool Pressure were to INCREASE, STATE whether the Tailpipe Temperature would INCREASE, DECREASE, or REMAIN THE SAME. (0.5)
- c. If the reactor starts to depressurize when the valve is opened, STATE whether the Tailpipe Temperature will INITIALLY INCREASE, DECREASE, or REMAIN THE SAME. (0.5)
- d. STATE the Reactor Pressure at which the Tailpipe Temperature would be at its MAXIMUM value (during the depressurization). (0.5)

(ASSUME A SATURATED SYSTEM AND INSTANTANEOUS HEAT TRANSFER)

QUESTION 1.07 (1.00)

The reactor trips from full power, equilibrium XENON conditions. Twenty-four (24) hours later the reactor is brought critical and power level is maintained on range 5 of the IRMs for several hours. Which of the following statements is CORRECT. (1.0)

- a. Rods will have to be withdrawn due to XENON build-in.
- b. Rods will have to be rapidly inserted since the critical reactor will cause a high rate of XENON burnout.
- c. Rods will have to be inserted since XENON will closely follow its normal decay rate.
- d. Rods will approximately remain as is as the XENON establishes its equilibrium value for this power level.

QUESTION 1.08 (2.00)

Answer each of the following questions either TRUE or FALSE

A reactor heat balance was manually calculated during the midnight to 8 a.m. shift, because the Process Computer was inoperable. The gain adjustment factors were computed, but the APRM gain adjustments have not been made.

- a. If the feedwater flow rate used in the heat balance calculation was LOWER than the actual feedwater flow rate, then the actual power is HIGHER than the currently calculated power. (0.5)
- b. If the reactor recirculation pump heat input used in the heat balance calculation was OMITTED, then the actual power is LOWER than the currently calculated power. (0.5)
- c. If the steam flow used in the heat balance calculation was LOWER than the actual steam flow, then the actual power is LOWER than the currently calculated power. (0.5)
- d. If the RWCU return temperature used in the heat balance calculation was LOWER than the actual RWCU return temperature, then the actual power is HIGHER than the currently calculated power. (0.5)

QUESTION 1.09 (2.00)

Concerning the Net Positive Suction Head (NPSH):

- a. DEFINE NPSH
- b. For each of the following, INDICATE whether the available NPSH at the suction of the recirculation pump would INCREASE/DECREASE/REMAIN THE SAME: (1.5)
 - (1) The Feedwater Flow is INCREASED
 - (2) The Recirculation Flow is INCREASED
 - (3) The Vessel Pressure is INCREASED from 200 psig to 800 psig

QUESTION 1.10 (1.00)

SELECT the answer below that would typically coincide with the MAXIMUM Control Rod Worth, during a reactor startup (1.0)

- a. Cold Shutdown prior to the startup
- b. Heatup in Progress (approximately 1% Reactor Thermal Power (RTP))
- c. Heatup Complete (approximately 1% RTP)
- d. 50% RTP
- e. 100% RTP

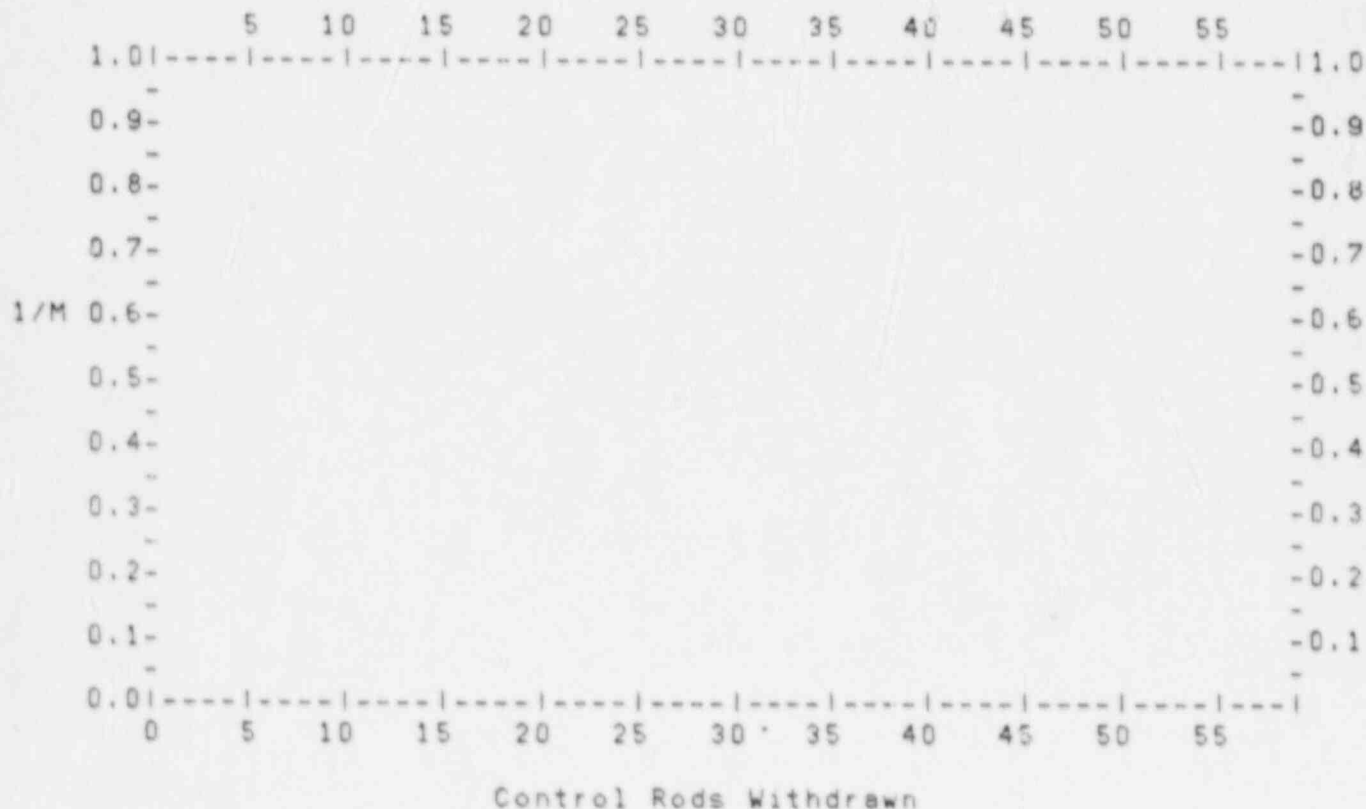
QUESTION 1.11 (2.00)

USE the 1/M plot and PREDICT the number of control rods required to be withdrawn to achieve criticality. *PLOT each data point given below on graph*

- NOTES: 1. CR = Count Rate
2. USE THE FIGURE BELOW TO SKETCH YOUR SOLUTION

Each CR reading is recorded following a 5 rod withdrawal with CRD representing 100% rod density.

Number of Control Rods Withdrawn	Count Rate
0	40
5	50
10	89
15	129
20	191
25	333
30	800



QUESTION 1.12 ^{1.50} (~~2.00~~)

An increase in void content in an operating reactor causes a negative reactivity insertion. DESCRIBE THREE (3) effects which cause the void reactivity coefficient (α_v) to be negative.

QUESTION 1.13 (2.00)

Concerning the effects of Control Rods on reactor power:

- a. EXPLAIN how a control rod withdrawal at certain power levels could result in a reactor power decrease (reverse power effect). (1.5) (1.0)
- b. Which one of the following would be the rod movement sequence most likely to cause the reverse power effect? (1.5) (1.0)

- (1) Deep Rod - 10 notch movement
- (2) Deep Rod - 1 or 2 notch movement
- (3) Shallow Rod - 10 notch movement
- (4) Shallow Rod - 1 or 2 notch movement

QUESTION 1.14 (1.00)

MULTIPLE CHOICE (Select the ONE correct answer.)

The Doppler Coefficient of Reactivity correlates the change in fuel temperature to a reactivity insertion.

Which statement is TRUE concerning Doppler Coefficient?

- a. The coefficient becomes less negative with fuel burnup, and more negative with control rod withdrawal.
- b. The coefficient becomes more negative with fuel temperature increase and less negative with void fraction increase.
- c. The coefficient becomes less negative with control rod withdrawal, and more negative with fuel temperature increase.
- d. The coefficient becomes more negative with void fraction increase and less negative with fuel temperature increase.

QUESTION 1.15 (1.50)

Suppose β_{eff} over core life decrease from 0.0072 to 0.0055. With equal insertions of 0.001 dK/K of positive reactivity:

- a. Calculate the change in the reactor period over core life. (1.0)
- b. What is the cause for this change in β_{eff} ? (5.0)

QUESTION 2.01 (~~2.50~~)^{2.01}

- a. What will cause an RECIRC SYS A (B) STARTUP SEQUENCE INCOMPLETE annunciator when starting a recirculation pump? (0.5)
~~(1.0)~~
- b. What automatic action occurs, if an incomplete start sequence is detected? (0.5)
- c. Describe any automatic actions that occur in the recirculation system as a result of REACTOR PRESSURE. INCLUDE SETPOINTS/SEQUENCES AS APPLICABLE. (1.0)

QUESTION 2.02 (3.00)

Answer the following questions concerning the Main Steam Isolation Valves (MSIV)

- a. State the applicable NORMAL and BACKUP pneumatic supplies to the Inboard MSIVs and to the Outboard MSIVs. (1.0)
- b. What provides the power for the MSIV normal operating solenoids and logics? (1.0)
- c. How does the loss of power to one of the MSIV solenoids affect opening and closing ability of that MSIV? (0.5)
- d. An MSIV isolation (Group I) has occurred. The operator attempts to reset the isolation without depressing the MSIV close pushbuttons. Does the isolation reset, and if so, what happens to the MSIVs? (0.5)

QUESTION 2.03 (2.00)

Describe the features of the Safety/Relief Valves (SRV) that results in increasing the amount of energy released per SRV opening. (2.0)

QUESTION 2.04 (2.00)

- a. The RCIC system has received a valid initiation signal and is ramping up in speed when the RCIC oil pump fails. HOW will the turbine respond (assuming the turbine does not trip on low oil pressure) and WHY? (1.0)
- b. The HPCI system has received a valid initiation signal and its auxiliary oil pump fails to start. HOW will the turbine respond and WHY? (1.0)

QUESTION 2.05 (2.50)

The Control Rod Drive Hydraulic System interfaces with (interacts with) several in-plant systems/equipment. EXPLAIN the interaction with each of the following systems.

- a. Control Air System
- b. Reactor Building Closed Cooling Water
- c. Recirculation Pumps
- d. Reactor Protection System
- e. Reactor Manual Control System

QUESTION 2.06 (2.50)

Answer each of the following questions concerning the Reactor Protection System (RPS)

- a. STATE the normal and alternate power supplies to RPS (1.0)
- b. List three (3) systems that receive electrical power from RPS (1.5)

QUESTION 2.07 (2.00)

State whether the following conditions or signals WILL or WILL NOT cause initiation of the Standby Gas Treatment System (SBGT) system: (2.0)

NOTE: Do not consider setpoints. If the indicated parameters will initiate the system, assume the setpoint has been reached.

- a. High radiation (ARM) on the Refuel Floor
- b. High radiation in the reactor building ventilation exhaust
- c. High particulate activity in the drywell
- d. Low flow in the offgas system
- e. High reactor pressure
- f. Low RPV pressure
- g. Low RPV water level
- h. Main Steam Line high radiation

QUESTION 2.08 (2.00)

The Standby Liquid Control System (SLC) injects sodium pentaborate solution into the reactor coolant. The pumps are designed to limit the boron injection rate. WHY (i.e. what is the basis) are there limits at which the solution must be injected (both upper and lower limits must be discussed for full credit)? (2.0)

QUESTION 2.09 (2.00)

- a. What is the purpose of the Rod Block Monitoring System? (1.0)
- b. The RBM gain change is done so that the RBM output will be equal to or greater than the reference APRM output. WHAT is the reason for changing the gain? (1.0)

QUESTION 2.10 (2.50)

In regards to the RHR loop injection valves, F015A/B and F017A/B:

- a. WHAT interlock must be satisfied to open BOTH injection valves in a loop? (0.5)
- b. WHAT is the purpose of this opening interlock? (0.5)
- c. Following an automatic initiation, WHAT INTERLOCK(S) must be satisfied to close each of these valves? (1.0)
- d. WHAT function is allowed after satisfying the close inhibit interlock(s)? (0.5)

QUESTION 2.11 (2.00)

Concerning the RWCU System:

- a. LIST four (4) conditions that will automatically isolate the RWCU system. (SETPOINTS NOT REQUIRED) (1.0)
- b. During startup, while blowing down the reactor, the rate of blowdown is limited. What limits the rate of blowdown and what is the reason for this limit. (1.0)

QUESTION 3.01 (2.00)

State whether each of the following parameters directly initiates a scram, rod block, both, or neither. Setpoints are not required. (2.0)

- a. Main steam line radiation
- b. Neutron flux
- c. Reactor vessel high level
- d. Recirculation flow

QUESTION 3.02 (2.50)

Answer each of the following questions concerning the Rod Worth Minimizer (RWM) system

- a. DEFINE or DESCRIBE (1.0)
 - (1) Low Power Set Point (LPSP)
 - (2) Low Power Alarm Point (LPAP)
- b. Reactor power is 17% and the RWM is operable.
 - (1) How many withdraw errors will the RWM allow? (0.5)
 - (2) How many insert errors will the RWM allow? (0.5)
 - (3) What restrictions are imposed on rod movement if the allowable number of errors is exceeded? (0.5)

QUESTION 3.03 (3.99)

Assume the feedwater level control system is being operated in 3-element control using reactor level detector channel "A". Reactor power is at 85%, steady state. For each of the instrument or control signal failures listed below, state how reactor level will initially respond (increase, decrease, or remain constant) and briefly explain why, in terms of what is happening in the Level Control System and Feedwater System immediately following the failure. *No SCRAM occurs.*

NOTE: A block diagram of the feedwater level control system is attached (figure 1)

- a. Channel "A" reactor level detector signal fails low.
- b. Loss of signal to "B" feedwater control valve M/A transfer station.
- c. "B" feedwater flow signal fails high.

QUESTION 3.04 (2.50)

For the following situations, state whether the Automatic Depressurization System (ADS) relief valves will OPEN, CLOSE or REMAIN AS IS. Consider each set of conditions separately.

- a. ADS initiating signal sealed in, ADS valves open . . . reactor water level then rises to 177 inches. (0.5)
- b. ADS initiating signal sealed in, ADS valves open . . . ADS timer reset buttons are then depressed. (0.5)
- c. ADS initiating signal sealed in, ADS valves open . . . then a DC power failure occurs that affects all busses supplying ADS valves. (0.5)
- d. ADS initiating parameters present, a loss of the pneumatic supply to the drywell has occurred, 120 second timer timing out . . . then the 120 second timer times out. (0.5)
- e. ADS initiating parameters present, all ECCS pumps are secured except for CS pump B which is running with a discharge pressure of 195 psig, 120 second timer timing out . . . then the 120 second timer times out. (0.5)

QUESTION 3.05 (2.50)

The Core Level Indicator (LI-R610) located on Control Room Panel H11-P601 has an indicating range from -150 to +50 inches.

- a. WHAT is INSTRUMENT ZERO for this level sensor?
(Provide the core component located at Instrument Zero.) (0.5)
- b. Is this level sensor temperature compensated? (0.5)
- c. MATCH the core conditions in Column 1 with the correct response of the core level indicator (LI-R610) in Column 2. (1.5)

COLUMN 1	COLUMN 2
1. LPCI is the only system injecting into the vessel	a. Full Scale
2. No recirculation flow exists, no systems are injecting to the RPV, and the reactor is at atmospheric pressure	b. Downscale
3. Both recirculation pumps are at 45% speed	c. Actual Level
	d. 14 inches HIGHER than actual level
	e. 14 inches LOWER than actual level

QUESTION 3.06 (1.00)

HOW would an SRM detector respond to a pinhole leak which caused a gradual decrease in Argon gas pressure?

- a. Gamma and neutron sensitivity would DECREASE.
- b. Gamma sensitivity would DECREASE but neutron sensitivity would REMAIN THE SAME.
- c. Gamma sensitivity would REMAIN THE SAME but neutron sensitivity would DECREASE.
- d. BOTH gamma and neutron sensitivity would REMAIN THE SAME.

QUESTION 3.07 (1.0)

LIST three (3) conditions and their setpoints (if applicable) that will automatically start station air compressors. (1.5)

QUESTION 3.08

Consider the following information:

- the Reactor is at 100% power
- APRM CHANNEL D is reading 102%
- FLOW UNIT A is reading 90%
- FLOW UNIT B is reading 100%
- 8 LPRM inputs to APRM CHANNEL D are bypassed

STATE whether each of the following will occur. (YES or NO) occur.

- a) RPS DIV I trip (1/2 scram) (0.5)
- b) Control Rod Withdrawal Block (0.5)
- ~~c) APRM D Inop~~ *deleted* (0.5)
- d) APRM D upscale high (0.5)
- e) APRM D upscale Hi Hi (0.5)
- f) Flow Reference Off Normal (0.5)

QUESTION 3.09 (1.50)

- a. WHAT automatic actions occur when the scram discharge volume high level scram bypass switch is placed in BYPASS and the scram is reset.
- b. WHAT position(s) must the reactor mode switch be in to allow BYPASS of the scram discharge volume high level scram function?
- c. WHAT additional protective function is inserted when the scram bypass switch is in the BYPASS position?

QUESTION 3.10 (3.00)

There are THREE (3) speed limiters in the Recirculation flow control system which function to limit the maximum speed demand.

- a. WHAT is the maximum speed demand limit imposed by EACH limiter?
(LIST EACH LIMITER AND THE APPLICABLE SETPOINT.) (1.0)
- b. During what plant conditions are the #1 & #2 limiters in control and imposing limitations on Recirculation pump speed?
(SETPOINTS REQUIRED) (2.0)

QUESTION 3.11 (1.50)

STATE three (3) conditions that would cause the Rod Sequence Control System Annunciator, ALL A/B SEQUENCE RODS NOT FULL OUT, to alarm.

QUESTION 4.01 (1.00)

Operation of the Recirculation Pumps at a suction pressure below 300 psig should be minimized since such operations can contribute to shortening seal life. However, STATE two reasons that may require recirculation pump operations at low pressure.

QUESTION 4.02 (1.00)

STATE two (2) alternate methods of scrambling the reactor per AOP 20.000.19, "Shutdown From Outside The Control Room".

QUESTION 4.03 (1.50)

During reactor startup, a control rod is to be withdrawn. LIST three possible indications that the control rod is uncoupled per AOP 20.106.04, "Uncoupled CRD".

QUESTION 4.04 (1.00)

SOP 23.202, "High Pressure Coolant Injection System", cautions not to defeat the automatic function of an ECCS by placing the controls in MANUAL or OFF unless confirmed by at least two independent indications. STATE these two (2) indications.

QUESTION 4.05 (2.00)

For each of the following conditions (a through d) INDICATE which of the corresponding procedures (1 through 4) would need to be entered. (More than one procedure may apply for each condition. If none of the procedures are required to be entered then state "NONE").

CONDITIONS

- a. Drywell pressure has increased to 3 psig
- b. Reactor water level cannot be determined
- c. Reactor water level decreasing to +160"
- d. Drywell Equipment Sump High Level Annunciator alarms

PROCEDURES

1. 29.000.01- Level/Pressure Control
2. 29.000.02- Cooldown
3. 29.000.03- Primary Containment Control
4. 29.000.04- Contingency For RPV Flooding

QUESTION 4.06 (2.00)

Per EOP 29.000.08, "Reactivity Control Procedure", if the MSIVs are open and the Main Condenser is available, the operator is to verify or manually runback the Reactor Recirculation Pumps to minimum speed before tripping them. Explain why these pumps are runback prior to tripping them. (2.0)

QUESTION 4.07 (1.50)

Answer the following TRUE or FALSE with regards to the Criteria for Standing Orders.

- a. Standing Orders should be used to provide additional guidance on administrative matters.
- b. Standing Orders may conflict with procedural requirements.
- c. Standing Orders can be issued prior to approval by the OE or AOE.

QUESTION 4.08 (2.00)

Match the following tag descriptions (a through d) with the correct type of tag (1 through 4)

DESCRIPTION

- a. May be placed to identify a short term condition or to explain limitations.
- b. Used to warn against the operation of electrical or mechanical equipment which could injury personnel.
- c. Used to mark the unsafe condition of equipment such as tools and ladders.
- d. Used for the protection of equipment or when determined to be required be the NSS.

TYPE OF TAG

- 1. Red Tag
- 2. Safety Tag
- 3. Information Tag
- 4. Equipment Protection Tag

QUESTION 4.09 (2.00)

Per GOP 22.000.03, "Startup From Cold Shutdown To Rated Power", when the reactor is critical, four items must be logged in the Control Room Log Book. LIST these four (4) items (some "items" may include more than one entry).

QUESTION 4.10 (2.00)

LIST the following system/equipment in the order they are taken out of service during a reactor shutdown from rated power with MSIVs open per GOP 22.000.10 Shutdown From Rated Power To Cold Shutdown.

1. Off Gas System
2. Last Reactor Feed Pump
3. Last Heater Feed Pump
4. Steam Jet Air Ejectors
5. Main Turbine

QUESTION 4.11 (2.00)

A local Area Radiation Monitor ARM alarms on the refueling floor. Upon receiving indication or notification of a Refueling Floor high radiation condition, STATE the four (4) immediate operator actions in the control room?

QUESTION 4.12 (2.00)

Match the following Emergency Classifications (a - d) with the appropriate description of that event (1 - 4). (2.0)

- | | |
|----------------------|--|
| a. General Emergency | 1. Events which involve actual or imminent substantial core degradation or melting has occurred with a potential for loss of containment integrity. |
| b. Site Emergency | 2. Any condition that involves an actual or potential substantial degradation of the level of safety of the plant. |
| c. Alert | 3. Events which involve likely or actual major failures of plant functions needed for the protection of the public. |
| d. Unusual Event | 4. Any station related event which indicates a potential degradation of the level of safety of the plant, but which is not likely to affect onsite personnel or the public or result in radioactive releases requiring offsite monitoring. |

QUESTION 4.13 (2.50)

List five of the six conditions that must be met to establish Secondary Containment Integrity. (2.5)

QUESTION 4.14 (2.00)

TRUE or FALSE

(2.0)

A General Radiation Work Permit (RWP) should be used for the following:

1. To enter posted Radiation Areas
2. To enter posted Contaminated Areas
3. To enter posted Airborne Radioactivity Area
4. To enter posted Neutron Radiation Area

(***** END OF CATEGORY 4 *****)
(***** END OF EXAMINATION *****)

ANSWER 1.01 (1.50)

- a. increase (0.5)
- b. increase (0.5)
- c. decrease (0.5)

REFERENCE

GE Thermodynamic, Heat Transfer, and Fluid Flow Text pg.9-85, 9-86, 9-92
 FERMI Nuclear Power Plant Thermal Sciences pg.10-10, 10-15
 K/A 293009 K1.23 (2.8/3.2)
 293009K123 ..(KA's)

ANSWER 1.02 (1.98)

	FAILURE MECHANISM	LIMITING CONDITION
	-----	-----
1. LHGR	A2	B1
2. APLHGR	A3	B3
3. MCPR	A1	B2

(6 @ 0.33 each = 1.98)

REFERENCE

FERMI Nuclear Power Plant Thermal Sciences pg 10-10, 10-15
 K/A 293009 K1.08 (3.0/3.4), K1.12 (2.9/3.5), K1.20 (3.1/3.6)
 293009K120 293009K112 293009K108 ..(KA's)

ANSWER 1.03 (1.00)

Obtain corresponding temp. from the steam tables by interpolation
 1000 psig = 546.3 deg F (.25)
 250 psig = 406.0 deg F (.25)
 Temp. change: 546.3 - 406.0 = 140.3 deg F (.25)
 Rate of cooldown: 140.3/1.75 = 80.2 deg F/hr (.25)

REFERENCE

Steam Tables

K/A 293003 K1.23 (2.8/3.1)

239003K123 ..(KA's)

ANSWER 1.04 (2.50)

- a. Doppler or fuel temperature
- b. Void
- c. Moderator temperature
- d. Void
- e. Moderator temperature
(5 at 0.5 each = 2.5)

REFERENCE

FERMI Reactor Theory Fundamental pg 7.5, 7.6

K/A 292004 K1.14 (3.3/3.3)

292004K114 ..(KA's)

ANSWER 1.05 (2.50)

- a. From POM 22.000.03, shortest permissible stable period equals 50 (+/-5) seconds. (0.5)
- b. Doubling time = (2 min)(60 s/1 min)/1.44 = 83.3 +/- 4 seconds. (0.5)
- c. 40% range 2 is equal to 0.04% on range 8 (0.25)
 $P(0) = 0.04$ $P(t) = 40$ Period = 75 seconds
 $P(t) = P(0) e^{(t/\text{period})}$
 $40 = 0.04 e^{(t/75 \text{ sec})}$
Time = ~~410~~ (+/- 20) seconds (1.25)

518

REFERENCE

GOP 22.000.03 rev 15 pg 13

FERMI Reactor Theory Fundamentals pg 10.9

K/A 292003 K1.08 (2.7/2.8)

292003K108 ..(KA's)

ANSWER 1.06 (2.00)

- a. 296 deg F (+/- 15 deg F) (0.5)
- b. Increase (0.5)
- c. Increase (0.5)
- d. 450 psia (+/- 50 psia) (0.5)

REFERENCE

Mollier diagram
Steam tables
System Book 1, 03-15-05, MS and Bypass System, Rev. 4, pg 31.
K/A 293003 K1.23 (2.8/3.1)
293003K123 ..(KA's)

ANSWER 1.07 (1.00)

c.

REFERENCE

FERMI Reactor Theory Fundamentals pg 9.8
K/A 292006 K1.07 (3.2/3.2)
292006K107 ..(KA's)

ANSWER 1.08 (2.00)

- a. FALSE (0.5)
- b. TRUE (0.5)
- c. FALSE (0.5)
- d. FALSE (0.5)

REFERENCE

FERMI Nuclear Power plant Thermal Sciences pg 11-9
K/A 293007 K1.13 (2.3/2.9)
293007K113 ..(KA's)

ANSWER 1.09 (2.00)

- a. $P_{act} - P_{sat}$ (or pressure measured at the inlet of the pump) (0.5)
- b. (1) INCREASE (More Subcooling at the pump suction)
- (2) DECREASE (Reduced pressure at the eye of the pump results in being closer to saturation pressure)
- (3) DECREASE (Further to saturation temperature and increased discharged fluid density causing less static head)
(3 at 0.5 each = 1.5)

REFERENCE

FERMI Nuclear Power Plant Thermal Sciences pg 17-20
K/A 293006 K1.10 (2.7/2.8, K1.08 (2.5/2.6)
293006K108 293006K110 ..(KA's)

ANSWER 1.10 (1.00)

c (1.0)

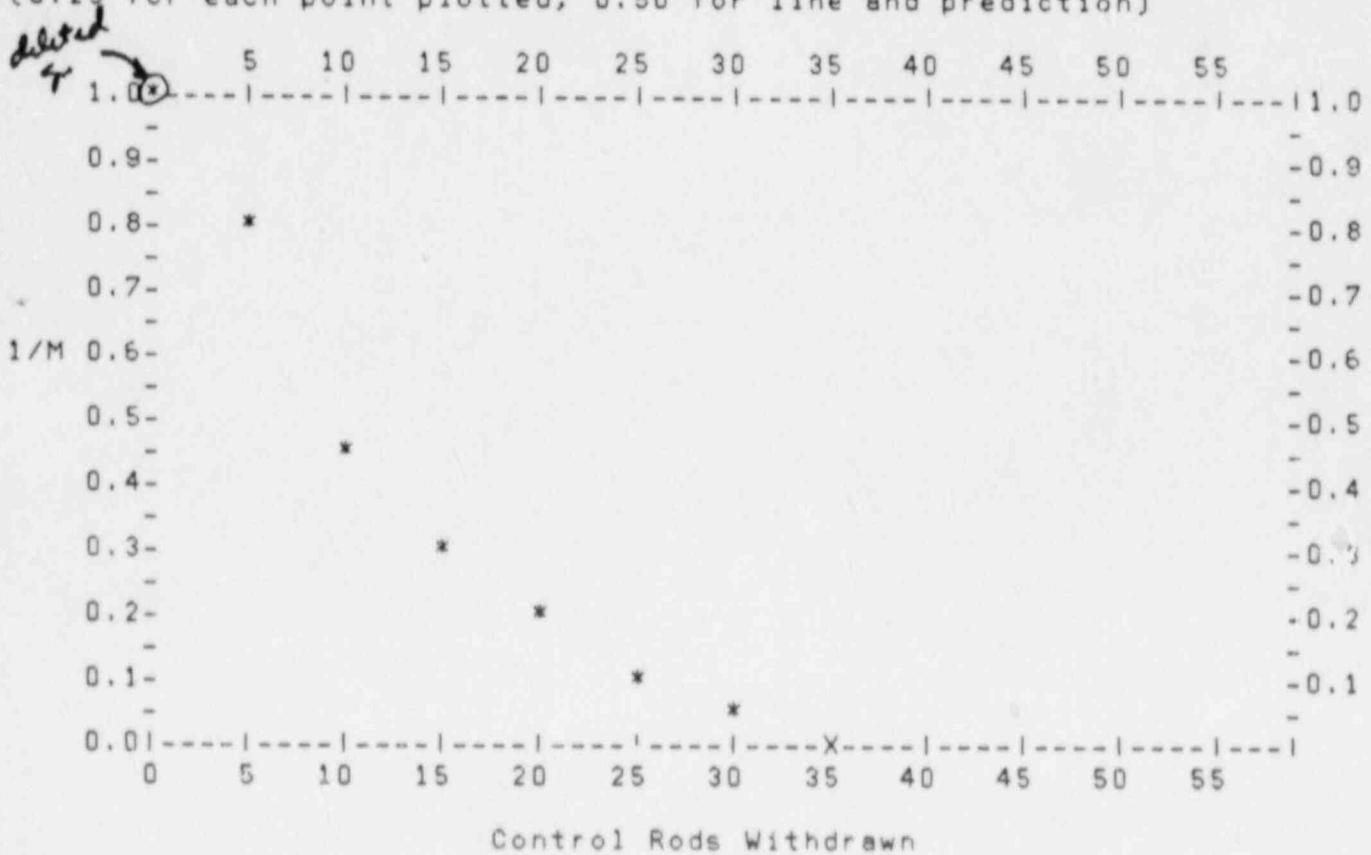
REFERENCE

General Electric Reactor Theory, Chapter 5
FERMI Reactor Theory Fundamentals pg 12.1
K/A 292005 K1.09 (2.5/2.6)
292005K109 ..(KA's)

ANSWER 1.11 (2.00)

32-37

Drawing a straight line between the last two *'s predicts ~~34-36~~ ³²⁻³⁷ control rods must be withdrawn.
(0.25 for each point plotted, 0.50 for line and prediction)



REFERENCE

GE BWR Academic Series, Reactor Theory, Chap. 3, pg. 13-15,
FERMI Reactor Theory Fund. pg 11.11
K/A 292008 K1.04 (3.3/3.4)
292008K104 ..(KA's)

ANSWER 1.12 ^{1.50} (~~2.00~~)

- (1) More neutrons will be captured in the resonant peaks of the fuel (uranium and plutonium (0.5) (as the slowing down length increases).
- (2) Thermal neutron leakage increases (0.5)
- (3) Fast neutron leakage increases (0.5).

REFERENCE

FERMI Reactor Theory Fundamentals pg 5.13
K/A 292002 K1.03 (2.0/2.1) 292002 K1.04 (1.9/2.0) 292004 K1.03 (2.6/2.7)
292004 K1.10 (3.2/3.2)
292004K110 292004K104 292002K104 292002K103 ..(KA's)

ANSWER 1.13 (2.00)

a. The negative reactivity added by the increased voids generated by the rod withdrawal is greater than the positive reactivity added by the reduced rod absorption.

(1.5)
(1.0)

b. (4)

(1.5)
(1.0)

REFERENCE

FERMI Reactor Theory Fund. pg 12.1 - 3
K/A 292005 K1.04 (3.5/3.5)
292005K104 ..(KA's)

ANSWER 1.14 (1.00)

d (1.0)

REFERENCE

FERMI Reactor Theory Fundamentals pg 8.22
K/A 292004 K1.05 (2.9/2.9)
292004K105 ..(KA's)

ANSWER 1.15 (1.50)

a. BOL: $t = B - P/u * P = (0.0072 - 0.001)/(0.1)(.001)$ (0.33)
 $= 62 \text{ seconds}$

EOL $t = (0.0055 - 0.001)/(0.1)(.001) = 45 \text{ seconds}$ (0.33)

CHANGE $62 - 45 = 17 \text{ seconds}$ (0.34)

b. Buildup of Pu-239 coupled with the burnout of U-235 causes a decrease in the effective delayed neutron fraction (β_{eff}). (0.5)

REFERENCE

FERMI Reactor Theory Fundamentals pg. 10.4, 10.12
K/A 292003 K1.06 (3.7/3.7)
292003K106 ..(KA's)

ANSWER 2.01 (~~2.50~~) ^{2.00}

- a. M-G set failed to complete startup sequence in <15 seconds 0.5
(1.0)
- b. The recirc M-G drive motor Lockout Relay trips (0.5)
- c. The recirc pumps trip (0.25) on high reactor pressure of 1153 +0/-20 g
~~1122~~ psig (0.25). The recirc pumps discharge valves shut (0.25) if a LPCI initiation signal is present (0.125) and reactor pressure decreases to 441 psig (0.125). g

REFERENCE

Recirculation System, pg 10, table 3 and 4
 K/A 202001 K1.12 (3.5/3.6), K1.27 (4.1/4.3), K4.07 (2.8/2.9)
 K4.14 (4.0/4.1), K6.06 (3.1/3.1)
 202001K606 202001K414 202001K407 202001K127 202001K112
 ..(KA's)

ANSWER 2.02 (3.00)

- a. The pneumatic supply for the inboard MSIVs is Nitrogen (0.34) with a backup supply from instrument air (0.33). The supply for the outboard MSIVs is instrument air (0.33). (other names: control air and interruptible control air (IAS)) g
- b. ~~125~~¹³⁰ VDC (0.5) and 120 VAC (0.5) (RPS acceptable for the 120 VAC)
- c. The loss of one solenoid will not affect the ability of the valve to open or close (0.5). Both solenoids must de-energize to close the valve. If either is energized, the valve will open.
- d. The isolation will not reset (0.5). (Other answers are possible, if the candidate clearly states assumptions made (e.g., unorthodox loss and restoration of power to MSIV solenoids, LER 50-341/87-837)).

REFERENCE

Main Steam System and Bypass System, Sys Bk 1, 03-15-05, Rev 4, pgs 14 & 20
 K/A 239001 K1.12 (2.5/2.6), 2.01 (3.2/3.3), K4.01 (3.8/3.8),
 K6.01 (3.1/3.3)
 239001K601 239001K401 239001K201 239001K112 ..(KA's)

ANSWER 2.03 (2.00)

The logic (Low-Low Set) is armed by actuation of any SRV (0.5) and a high reactor pressure scram signal (0.5), the logic will lower the opening and closing setpoints of two SRVs (0.5). The valve blowdown is also increased (0.5)

REFERENCE

Main Steam System and Bypass System pg 10
K/A 239002 K4.01 (3.9/4.0), K4.02 (3.4/3.6)
239002K402 239002K401 ..(KA's)

ANSWER 2.04 (2.00)

- a. The turbine will speed up and trip on overspeed (0.5) because the governor valve will fail open (0.5).
- b. The turbine will not start (0.5) because the governor and stop valves require oil pressure to open (0.5) which is normally supplied by the auxiliary oil pump.

REFERENCE

RCIC, Figures 3 and 7
HPCI, Figures 4 and 7
K/A 217000 A2.07 (3.1/3.1), 206000 K4.14 (3.4/3.4),
206000 K5.05 (3.3/3.3)
206000K505 206000K414 217000A207 ..(KA's)

ANSWER 2.05 (2.50)

- a. Services the flow control valves, scram valves and scram discharge volume vent and drain valves.
- b. The CRD pumps are cooled by the Reactor Building Closed Cooling water system.
- c. The CRD hydraulic system supplies recirculating pump seal purge water.
- d. The RPS provides signals to energize or de-energize scram pilot and scram valves and backup scram valves (to insert rods on a scram.)
- e. The Reactor Manual Control System provides signals to the hydraulic control unit, to position directional control valves to control rod motion.

(5 at 0.50 each = 2.5)

REFERENCE

CRDH System pg 23 and 24
 K/A 201001 K1.03 (3.1/3.1), K1.06 (2.8/2.8), K1.07 (3.4/3.4)
 K1.08 (3.4/3.4), K1.09 (3.1/3.2)
 201001K109 201001K108 201001K107 201001K106 201001K103
 ..(KA's)

ANSWER 2.06 (2.50)

- a. Normal - RPS M-G sets (0.5)
 Alternate - 480V MCC (~~T2C-2D POS 2 and T2-4A POS 2~~) (0.5)
- b. 1. Process Radiation Monitoring System (0.5)
 2. Power Range Monitoring System (0.5)
 3. Nuclear Steam Supply Shut-off System (0.5)

REFERENCE

RPS pg 5, 28
 K/A 212000 A2.02 (3.7/3.9)
 212000A202 ..(KA's)

(DIV I (RPS M-G Set A) - 480 MCC, 72B-4C Pos 2)
 (DIV II (RPS M-G Set B) - 480 MCC, ~~72F-4B Pos 2~~
 72E-5B).

(DIV I (RPS Alternate Transformer) -
 480VAC Dist Cabinet 72C-2D Pos 2)
 (DIV II (RPS Alternate Transformer B) -
 480VAC Dist. Cabinet 72F-4B Pos 2)

ANSWER 2.07 (2.00)

will: ^{deleted g.} b, → g

will not: a, c, d, f, h _g
(0.25 each = 2.0)

REFERENCE

SBG system pg 14
K/A 261000 K401 (3.7/3.8)
261000K401 ..(KA's)

ANSWER 2.08 (2.00)

The system discharge boron injection is limited such that the rate of increase in the concentration of natural boron in the primary coolant water is fast enough to ensure a negative reactivity insertion rate greater than positive reactivity addition rate due to plant cooldown (1.0), yet slow enough to ensure sufficient mixing so boron does not recirculate through the reactor core in uneven concentrations (could cause power cycling). (1.0).

(2.0)

REFERENCE

chugging a

SLC System pg 5
K/A 211000 K4.05 (3.4/3.6), A1.07 (4.3/4.4)
211000A107 211000K405 ..(KA's)

ANSWER 2.09 (2.00)

- a. 1. To prevent local fuel damage that may result from a single rod withdrawal error. (0.5)
- 2. Provides a signal used by the operator to evaluate the change in local relative power level during control rod movement. (0.5)
- b. The local power may be lower than the core average power. (1.0)

REFERENCE

RBM pg 4 and 7
K/A 215002 K1.02 (3.2/3.1), G004 (3.3/3.4)
215002G004 215002K102 ..(KA's)

ANSWER 2.10 (2.50)

- a. Reactor Pressure less than 441^{+20/-0} psig (0.5)
- b. Prevent over pressurization of the low pressure piping upstream of the injection valves. (0.5)
- c. F015A/B: Remove initiation signal (0.5)
F017A/B: 5 minutes after initiation signal is received (0.5)
- d. F017A/B can be throttled to control injection flow (0.5)

REFERENCE

RHR pg 32
K/A 203000 K4.02 (3.3/3.4), K4.10 (3.9/4.1)
203000K410 203000K402 ..(KA's)

ANSWER 2.11 (2.00)

- a. 1. Low reactor water level (level 2)
2. High temperature at outlet of the non-regenerative heat exchanger (140 deg F)
3. SLC initiation
4. High ambient temperature (>183 F)
5. High differential monitoring temperature (>53 F)
6. High differential flow comparison (>63.4 gpm) and 60 second time delay
(any 4 at 0.25 each = 1.0)
- b. Non-regenerative heat exchanger outlet temperature <130 deg F (0.5)
To prevent damage to the ion exchange resin (0.5)

REFERENCE

RWCU pg 14 and 18
K/A 204000 K4.01 (2.5/2.5), K4.03 (2.9/2.9), K4.04 (3.5/3.6)
204000K404 204000K403 204000K401 ..(KA's)

ANSWER 3.01 (2.00)

- a. scram
 - b. both
 - c. neither
 - d. rod block
- [4 at 0.5 each = 2.0]

REFERENCE

RPS, pg 6
 RBM, pg 16
 K/A 212000 K1.01 (3.7/3.9), K1.03 (3.4/3.6), K1.05 (3.3,3.6),
 K1.14 (3.6/3.7)
 212000K114 212000K105 212000K103 212000K101 ..(KA's)

ANSWER 3.02 (2.50)

- a. (1) The LPSP is defined as the power level ^(25+/-5%) below which the RWM program is enforcing adherence to the control rod movement as compared to the rod sequence (0.5) or similar. R
- (2) The LPAP is the power level ^(375+/-2.5%) above which all RWM blocks, alarms, and error displays are discontinued (0.5) or similar. R
- b. (1) 1 (0.5)
- (2) 3 (0.5)
- (3) Rod blocks will occur on all rods (0.25) except for the rod(s) required to correct the insert or withdraw errors (0.25).

REFERENCE

Rod Worth Minimizer, pg 4, 15
 K/A 201006 K4.01 (3.4/3.5), K4.02 (3.5/3.5), K4.06 (3.2/3.4),
 K4.07 (3.1/3.2)
 201006K407 201006K406 201006K402 201006K401 ..(KA's)

ANSWER 3.03 (3.99)

- a. Causes reactor level to increase [0.33] due to the level control system having a level error, level set, indicated level [0.5] resulting in the feedwater control valves opening to match new higher level [0.5].
- b. Reactor level should remain constant [0.33] because the "B" M/A transfer station will lock-up [0.5]. The "A" feedwater control valve will control level [0.5].
- c. Causes reactor level to decrease [0.33] due to the level control system having a steam flow/feed flow error, steam flow < feed flow [0.5] resulting in the feedwater control valves to close to match new lower level [0.5].

REFERENCE

Reactor Vessel Level Control System pg 12, 13, and figure 1
 K/A 259001 K1.08 (3.2/3.2), K1.09 (2.9/3.0), K3.01 (3.8/3.8)
 K3.02 (3.7/3.7)
 259001K302 259001K301 259001K109 259001K108 ..(KA's)

ANSWER 3.04 (2.50)

- a. ADS valves remain as is (0.5)
 b. ADS valves close (0.5)
 c. ADS valves close (0.5)
 d. ADS valves open (0.5)
 e. ADS valves open (0.5)

REFERENCE

Automatic Depressurization System, pg 2, R, 9, and figure 6
 K/A 218000 K5.01 (3.3/3.8), K4.04 (3.5/3.1), A2.05 (3.4/3.6),
 K6.02 (4.1/4.1), A1.05 (4.1/4.1)
 218000K602 218000K501 218000K404 218000A205 218000A105
 ..(KA's)

ANSWER 3.05 (2.50)

- a. Top of active fuel (0.50)
- b. Yes (0.50)
- c. 1. d
2. c
3. e

(3 at 0.5 each = 1.5)

REFERENCE

FERMI RPV Process Instrumentation pg 6.

K/A 216000 K1.05 (3.7/3.9), K1.22 (3.6/3.8), K1.23 (3.3/3.4)

K5.01 (3.1/3.2), K4.14 (3.3/3.4)

216000K123 216000K105 216000K414 216000K501 216000K122

..(KA's)

ANSWER 3.06 (1.00)

a

REFERENCE

SRM pg 17

K/A 291002 K1.22 (3.0/3.1), K1.19 (3.0/3.1), 215004 K5.01 (2.6/2.6)

215004K501 291002K119 291002K122 ..(KA's)

ANSWER 3.07 (1.50)

- a. Motor overload (0.25); 175 % (0.25)
 - b. High frame oil temperature (0.25); 150 deg F (0.25)
 - c. Low lube oil pressure (0.25); 10 psig (0.25)
 - d. Ground fault (0.50)
- (any three = 1.50)

REFERENCE

Compressed Air System pg 7
 K/A 295019G011 (3.9/4.1), G009 (3.4/3.4)
 295019G009 295019G001 ..(KA's)

- ANSWER 3.08 ^{2.50} ~~(3.00)~~ *gr*
- a) NO (0.5)
- b) YES (0.5)
- ~~c) NO~~ *deleted gr* (0.5)
- d) NO (0.5)
- e) NO (0.5)
- f) YES (0.5)

REFERENCE

Power Range Monitor (PRM) pg 13
 FERMI Tech Specs pg 3/4 3-5
 K/A 215005 K101 (4.0/4.0), K4.01 (3.7/3.7), K4.02 (4.1/4.2)
 215005K402 215005K401 215005K101 ..(KA's)

- ANSWER 3.09 (1.50)
- a. Opens the scram discharge volume vent and drain valves (0.5)
- b. Shutdown [0.25] or Refuel [0.25] (0.5)
- c. Rod Withdrawal Block (0.5)

REFERENCE

CRD Hydraulics, pg 12
 K/A 201001 A1.05 (3.3/3.4), K4.06 (3.8/3.9)
 201001K406 201001A105 ..(KA's)

ANSWER 3.10 (2.00)

27

- a. ~~Output of the individual pump controller [0.25] 80% [0.25] (0.5)~~ 4
- #1 Limiter < 30% (0.25)
- #2 Limiter < ~~45%~~ 42% (0.25)
- #3 Limiter < 48% (0.25)
- b. Limiter #1: Recirc. pump discharge valve [0.25] not fully open [0.25] 2
- OR feedwater flow [0.25] < 20% [0.25] (1.0)
- Limiter #2: "A" or "B" Reactor feed pumps [0.25] not running rated flow [0.25] and a reactor vessel [0.25] level less than level 4 (192.5") [0.25] is received. (1.0)

REFERENCE

Recirculation Flow Control System pg 10-12; and PDC #294. 2

K/A 202002 K4.02 (3.1/3.2), K4.07 (2.8/2.9)

202002K407 202002K402 ..(KA's)

ANSWER 3.11 (1.50)

- Power less than 35%
- Neither all A sequence control rods or all B sequence control rods are fully out, and
- Power is greater than 30% (or the Reactor Mode Switch is in the REFUEL position)
(3 at 0.5 each = 1.5)

REFERENCE

Rod Sequence Control System pg 8

K/A 201004 G008 (3.7/3.4)

201004G008 ..(KA's)

- ANSWER 4.01 (1.00) Restored [0.50].
- 3. During plant starting [0.25], when RHR shutdown cooling is secured [0.25].
 - 4. During plant shutdown [0.25] prior to RHR shutdown cooling being started [0.25].
 - 5. During a loss of RHR shutdown cooling (when RHR cannot be restored) [0.50].
- [NOTE: Only 2 of the items above are required for full credit].

- 1. Preventing temperature stratification in the Reactor Vessel. (0.5)
- 2. Retaining solids in suspension until they can be removed (by the RWCU system to prevent their deposition in the bottom of the Reactor Vessel of the CRD Mechanisms). (0.5)

REFERENCE

SOP 23.138.01 Rev. 21, Reactor Recirculation System, pg 7
K/A 202001 K1.05 (3.4/3.4), K4.12 (3.2/3.5)
202001K412 202001K105 ..(KA's)

ANSWER 4.02 (1.00)

- 1. Scram the reactor at H11-P608RR by taking one operable APRM's Mode switch out of OPERATE position in Div. I and DIV. II (0.5)
- 2. Trip the main turbine at H11-P632 by removing the relay cover from TTR1 or TTR2 and then push back on the relay trip coil bar at the top until the trip flag falls. (0.5)

REFERENCE

AOP 20.000.19, Shutdown From Outside The Control Room
K/A 295016 G006 (4.1/4.1), AK2.02 (4.0/4.1)
295016K202 295016G006 ..(KA's)

ANSWER 4.03 (1.50)

- 1. Control Rod Overtravel annunciator
- 2. Loss of position indication, (past position 48), when fully withdrawn
- 3. Control Rod Drift annunciator

REFERENCE

AOP 20.106.04, Uncoupled CRD
K/A 201003 G015 (3.8/3.9)
201003G015 ..(KA's)

- 1. Observe a response to the rod movement through Nuclear Instrumentation (NI) Response [0.25] AND Demonstrate that the control rod will go to the withdraw overtravel position [0.25]. [Any 3 of the above items for full credit].

ANSWER 4.04 (1.00)

1. Misoperation in automatic was initiated
2. Adequate core cooling is assured
(2 at 0.5 = 1.0)

REFERENCE

SOP 23.202, HPCI System pg 13
K/A 206000 A2.17 (3.9.4.3)
206000A217 ..(KA's)

ANSWER 4.05 (2.00)

- a. 3
- b. 4
- c. 1
- d. none
(4 at 0.5 each = 2.0)

REFERENCE

EOP 29.000.01 pg 1, 29.000.03 pg 1, 29.000.04 pg 1
K/A 295024 G011 (4.3/4.5), 295031 G011 (4.2/4.6), 295036 G011 (3.8/4.1)
295036G011 295031G011 295024G011 ..(KA's)

ANSWER 4.06 (2.00)

Running back the recirculation pumps prior to tripping them minimizes the heat load added to the suppression pool [1.0]. If the pumps were tripped at higher speeds it may cause a severe enough transient to trip the main turbine and lift the safety/relief valves which would add heat to the torus [1.0]. (2.00)

REFERENCE

EOP 29.000.08 rev 2, pg 2
OC&P Course: ATWAS Study Guide Section X
K/A 294001 K1.09 (3.4/3.8)
294001K109 ..(KA's)

ANSWER 4.07 (1.50)

- a. True
- b. False
- c. False

REFERENCE

Admin Proc. 21.000.01, Conduct of Shift Operations, pg 12
K/A 294001 A1.03 (2.7/3.7)
294001A103 ..(KA's)

ANSWER 4.08 (2.00)

- a. 3
- b. 1
- c. 2
- d. 4

(4 at 0.5 each = 2.0)

REFERENCE

Admin Proc 12.000.012 rev 15, Tagging and Protective Barrier System, pg7
K/A 294001 K1.02 (3.9/4.3)
294001K102 ..(KA's)

ANSWER 4.09 (2.00)

1. Time
2. Rod Sequence, Rod Group, Rod, and Rod Position
3. Reactor Coolant Temperature (as indicated by RWCU inlet or Reactor Recirc. loop temperature)
4. Reactor period

(4 at 0.5 each = 2.0)

REFERENCE

GOP 22.000.03 rev 15, Startup From Cold Shutdown To Rated Power pg 1.
K/A 294001 A1.06 (3.4/3.6)
294001A106 ..(KA's)

ANSWER 4.10 (2.00)

- 5, 2, 4, 1, 3
(0.4 each, subtract 0.4 for each one out of order up to the value 2.0)

REFERENCE

GOP 22.000.10 rev 6, pg 8, 10, 11
K/A 294001 A1.13 (4.5/4.3)
294001A113 ..(KA's)

ANSWER 4.11 (2.00)

1. Announce the event over the Hi-comm.
2. Sound the "Plant Area Alarm".
3. Notify Health Physics.
4. Notify Security.

~~(4 at 0.5 each = 2.0)~~ 8

5. Notify Nuclear Shift Supervisors (NSS)

[Any 4 of the items above with each worth 0.50 point].

REFERENCE

AGP 20.710.01. Refueling Floor High Radiation rev 5, pg 1
K/A 295023 6010 (3.9/3.9)
2950236010 ..(KA's)

ANSWER 4.12 (2.00)

- a. 1
 - b. 3
 - c. 2
 - d. 4
- (0.5 each)

REFERENCE

EP 102, 103, 104, 105
K/A 294001 A1.16 (2.9/4.7)
294001A116 ..(KA's)

ANSWER 4.13 (2.50)

1. All secondary containment penetrations required to be closed during accident conditions are either:
 - a. Capable of being closed by an operable containment automatic isolation system, or
 - b. Closed by at least one manual valve, blind flange, or deactivated automatic damper secured in its closed position, (except as provided in Table 3.6.5.2-1 of Spec 3.6.5.2)
2. All secondary containment hatches and blowout panels are closed and sealed
3. The standby gas treatment system is operable (or in compliance with the requirements of Spec 3.6.5.3)
4. At least one door in each access to the secondary containment is closed.
5. The sealing mechanism associated with each secondary containment penetration is operable.
(e.g., welds, bellows, or O-rings)
6. The pressure within the secondary containment is less than or equal to the value 0.125 inch of vacuum water gauge (required by Spec 4.6.5.1.a)

(any 5 at 0.5 each = 2.5)

REFERENCE

Tech Spec. 1.36 pg 1-6
K/A 290001 6011 (3.3/4.2)
2900016011 ..(KA's)

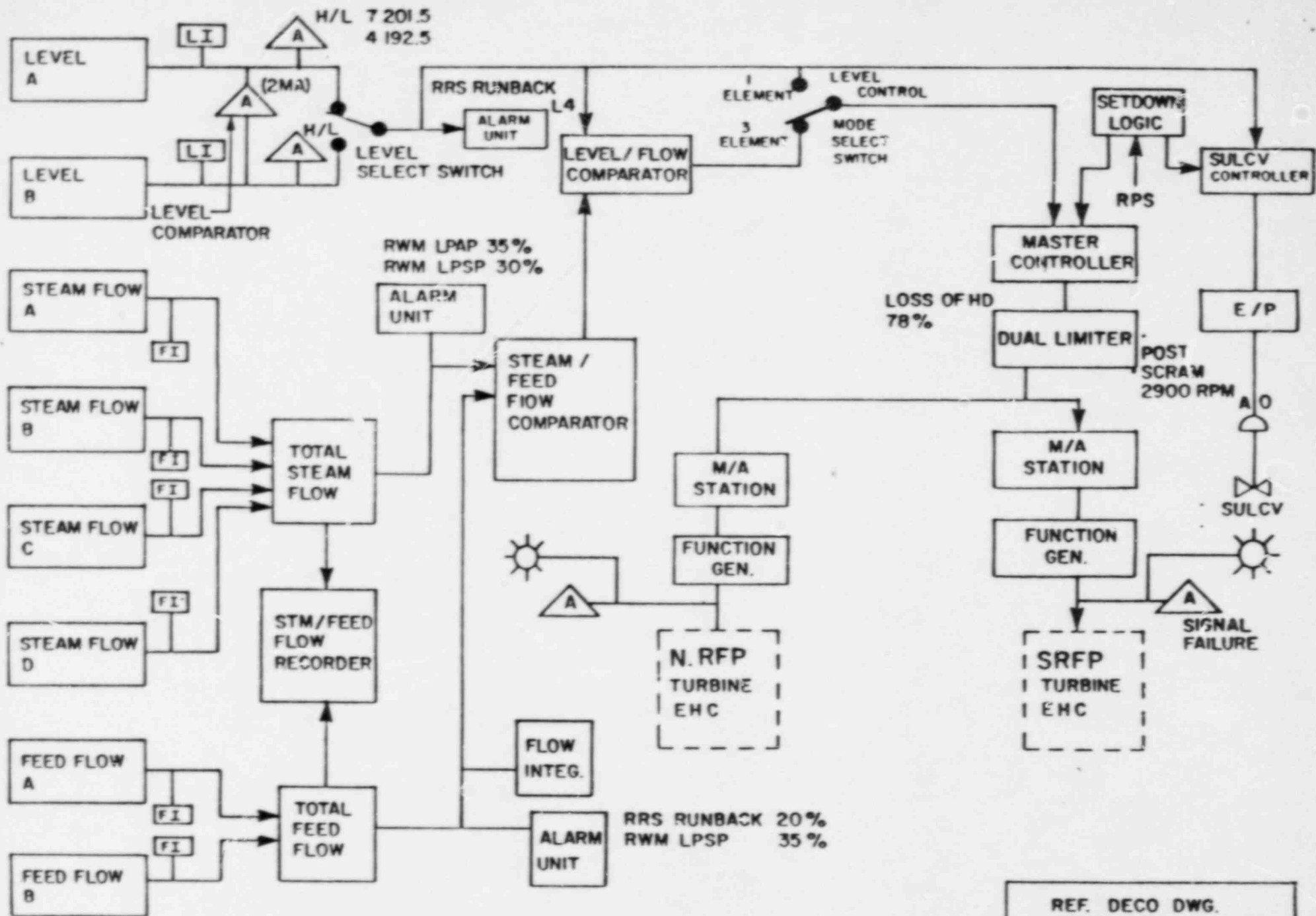
ANSWER 4.14 (2.00)

1. False
2. False
3. False
4. False
(4 at 0.5 each = 2.0)

REFERENCE

Admin Proc 12.000.013 rev 8, pg 7
K/A 294001 K1.03 (3.3/3.8)
294001K103 ..(KA's)

(***** END OF CATEGORY 4 *****)
(***** END OF EXAMINATION *****)



REF. DECO DWG.
 61721-2126-1 REV. E
 03-15-46-01-A1

FIGURE 1

DATA SHEET

REACTION THEORY FORMULAS:

$$P = P_0 e^{t/\tau}$$

$$P = P_0 10^{\text{SUR}(t)}$$

$$P = \frac{\Sigma \bar{\nu}_{th} V}{3.12 \times 10^{10} \text{ fissions/sec}}$$

$$\text{SUR} = 26.06/\tau$$

$$P_{th} = \frac{1}{1 + (B^2 L_{th}^2)} = e^{-(B^2 L_{th}^2)}$$

$$\rho = \frac{1^*}{\tau} + \frac{\bar{\beta}_{eff}}{1 + \tau}$$

$$P_f = e^{-(B^2 L_f^2)}$$

$$\rho = \frac{K - 1}{K}$$

$$p = e^{-[N][I_{eff}]/\beta \Sigma_s}$$

$$\Delta \rho = \ln \frac{K_{final}}{K_{initial}}$$

$$C_1 (1 - K_{eff1}) = C_2 (1 - K_{eff2})$$

$$\tau = \frac{\bar{\beta}_{eff} - \rho}{\lambda \rho}$$

$$m = \frac{1}{1 - K} = \frac{C_{final}}{C_{initial}}$$

$$\tau = \frac{1^*}{\rho}$$

$$\alpha_T = \frac{1}{f} \frac{\Delta f}{\Delta t} + \frac{1}{p} \frac{\Delta p}{\Delta t} - \beta^2 \left(\frac{\Delta L_f^2}{\Delta t} + \frac{\Delta L_{th}^2}{\Delta t} \right)$$

$$K_{eff} = \epsilon P_f p P_{th} f \eta$$

$$P_1 = P_0 \frac{\bar{\beta}_{eff} - \rho_0}{\bar{\beta}_{eff} - \rho_1}$$

DATA SHEET

THERMODYNAMICS AND FLUID MECHANICS EQUATIONS:

$$\dot{Q} = \dot{m} \Delta h$$

$$\dot{Q} = U A (\Delta T_m)$$

$$\dot{Q} = \dot{m} c_p (\Delta T)$$

$$\eta = \frac{\dot{Q}_{in} - \dot{Q}_{out}}{\dot{Q}_{in}}$$

$$\eta_p = \frac{W_{actual}}{W_{supplied}}$$

$$\dot{m} = \rho A V$$

$$\dot{m} = K A \sqrt{\Delta P_x \rho}$$

$$\Delta T_m = \frac{\Delta T (in) - \Delta T (out)}{\ln \left(\frac{\Delta T (in)}{\Delta T (out)} \right)}$$

$$T_{cl} - T_{ps} = \frac{Gr^2}{4k}$$

$$\dot{Q} = \frac{A \Delta T_{total}}{\frac{\Delta x_a}{K_a} + \frac{\Delta x_b}{K_b} + \dots + \frac{\Delta x_n}{K_n}}$$

$$\dot{Q} = \frac{2 \pi L \Delta T}{\frac{1}{K} + \frac{\ln R_2/R_1}{K_2} + \frac{\ln R_3/R_2}{K_3}}$$

$$\dot{Q} = \alpha \delta A R^4$$

$$\eta = \frac{(h_{in} - h_{out})_{real}}{(h_{in} - h_{out})_{ideal}}$$

$$\frac{P_1 V_1}{T_1} = \frac{P_2 V_2}{T_2}$$

$$\rho_1 A_1 V_1 = \rho_2 A_2 V_2$$

$$\dot{m}_{nc} = K A_Q \sqrt[3]{\dot{Q}} = K A \Delta T \sqrt{\Delta T} = K A \Delta p \sqrt{\Delta P}$$

$$G = \frac{\tau_{f,th}}{8.8 \times 10^9}$$

$$\dot{Q} = \frac{k A \Delta T}{\Delta x}$$

DATA SHEET

CENTRIFUGAL PUMP LAWS:

$$\frac{N_1}{N_2} = \frac{\dot{m}_1}{\dot{m}_2}$$

$$\frac{(N_1)^2}{(N_2)^2} = \frac{H_1}{H_2}$$

$$\frac{(N_1)^3}{(N_2)^3} = \frac{P_1}{P_2}$$

RADIATION AND CHEMISTRY FORMULAS:

$$R/hr = 6CE/d^2$$

$$I_x = I_0 e^{-mx}$$

$$C_1 V_1 = C_2 V_2$$

$$G = \frac{\text{Dilution Rate}}{\text{Volume}}$$

$$I = I_0 \left(\frac{1}{10}\right)^n$$

$$C = C_0 e^{-Gt}$$

$$A = A_0 e^{-\lambda t}$$

$$A = \lambda N$$

CONVERSION:

$$1 \text{ gm/cm}^3 = 62.4 \text{ lbm/ft}^3$$

$$\text{Density of water (20 C)} = 62.4 \text{ lbm/ft}^3$$

$$1 \text{ gal} = 8.345 \text{ lbm}$$

$$1 \text{ ft}^3 = 7.48 \text{ gal}$$

$$\text{Avogadro's Number} = 6.023 \times 10^{23}$$

$$1 \text{ gal} = 3.78 \text{ liters}$$

$$\text{Heat of Vapor (H}_2\text{O)} = 970 \text{ Btu/lbm}$$

$$1 \text{ lbm} = 454 \text{ grams}$$

$$\text{Heat of Fusion (ICE)} = 144 \text{ Btu/lbm}$$

$$e = 2.72$$

$$1 \text{ AMU} = 1.66 \times 10^{-24} \text{ grams}$$

$$\pi = 3.14159$$

$$\text{Mass of Neutron} = 1.008665 \text{ AMU}$$

$$1 \text{ KW} = 738 \text{ ft-lbf/sec}$$

$$\text{Mass of Proton} = 1.007277 \text{ AMU}$$

$$1 \text{ KW} = 3413 \text{ Btu/hr}$$

$$\text{Mass of Electron} = 0.000549 \text{ AMU}$$

$$1 \text{ HP} = 550 \text{ ft-lbf/sec}$$

$$\text{One atmosphere} = 14.7 \text{ psia} = 29.92 \text{ in. Hg}$$

$$1 \text{ HP} = .746 \text{ KW}$$

$$^\circ\text{F} = 9/5 \text{ }^\circ\text{C} + 32$$

$$1 \text{ HP} = 2545 \text{ Btu/hr}$$

$$^\circ\text{C} = 5/9 (\text{ }^\circ\text{F} - 32)$$

$$1 \text{ Btu} = 778 \text{ ft-lbf}$$

$$^\circ\text{R} = \text{ }^\circ\text{F} + 460$$

$$1 \text{ MEV} = 1.54 \times 10^{-16} \text{ Btu}$$

$$^\circ\text{K} = \text{ }^\circ\text{C} + 273$$

$$h = 4.13 \times 10^{-21} \text{ M-sec}$$

$$1 \text{ W} = 3.12 \times 10^{10} \text{ fissions/sec}$$

$$g_c = 32.2 \text{ lbm-ft/lbf-sec}^2$$

$$c^2 = 931 \text{ MEV/AMU}$$

$$1 \text{ inch} = 2.54 \text{ cm}$$

$$C = 3 \times 10^8 \text{ m/sec}$$

$$\sigma = 0.1714 \times 10^{-8} \text{ Btu/hr ft}^2 \text{ R}^4$$

DATA SHEET

AVERAGE THERMAL CONDUCTIVITY (K)

Material	K
Cork	0.025
Fiber Insulating Board	0.028
Maple or Oak Wood	0.096
Building Brick	0.4
Window Glass	0.45
Concrete	0.79
1% Carbon Steel	25.0
1% Chrome Steel	35.00
Aluminum	118.00
Copper	223.00
Silver	235.00
Water (20 psia, 200 degrees F)	0.392
Steam (1000 psia, 550 degrees F)	0.046
Uranium Dioxide	1.15
Helium	0.135
Zircaloy	10.0

MISCELLANEOUS INFORMATION:

$$E = mc^2$$

$$KE = 1/2 mv^2$$

$$PE = mgh$$

$$V_f = V_0 + at$$

Geometric Object	Area	Volume
Triangle	$A = 1/2 bh$	////////////////////////////////////
Square	$A = S^2$	////////////////////////////////////
Rectangle	$A = L \times W$	////////////////////////////////////
Circle	$A = \pi r^2$	////////////////////////////////////
Rectangular Solid	$A = 2(L \times W + L \times H + W \times H)$	$V = L \times W \times H$
Right Circular Cylinder	$A = (2 \pi r^2)h + 2(\pi r^2)$	$V = \pi r^2 h$
Sphere	$A = 4 \pi r^2$	$V = 4/3 (\pi r^3)$
Cube	////////////////////////////////////	$V = S^3$

DATA SHEET

MISCELLANEOUS INFORMATION (continued):

				10 CFR 20 Appendix B			
			Table I		Table II		
Material	Half-Life	Gamma Energy MEV per Disintegration		Col I Air uc/ml	Col II Water uc/ml	Col I Air uc/ml	Col II Water uc/ml
Ar-41	1.84 h	1.3	Sub	2×10^{-6}	-----	4×10^{-8}	-----
Co-60	5.27 y	2.5	S	3×10^{-7}	1×10^{-3}	1×10^{-8}	5×10^{-5}
I-131	8.04 d	0.36	S	9×10^{-9}	6×10^{-5}	1×10^{-10}	3×10^{-7}
Kr-85	10.72 y	0.04	Sub	1×10^{-5}	-----	3×10^{-7}	-----
Ni-65	2.52 h	0.59	S	9×10^{-7}	4×10^{-3}	3×10^{-8}	1×10^{-4}
Pu-239	2.41×10^4 y	0.008	S	2×10^{-12}	1×10^{-4}	6×10^{-14}	5×10^{-6}
Sr-90	29 y	-----	S	1×10^{-9}	1×10^{-5}	3×10^{-11}	3×10^{-7}
Xe-135	9.09 h	0.25	Sub	4×10^{-6}	-----	1×10^{-7}	-----
Any single radionuclide with $T_{1/2} > 2$ hr which does not decay by alpha or spontaneous fission				3×10^{-9}	9×10^{-5}	1×10^{-10}	3×10^{-6}

Neutron Energy (MEV)	Neutrons per cm^2 equivalent to 1 rem	Average flux to deliver 100 mrem in 40 hours
thermal	570×10^6	670
0.02	400×10^6	280 (neutrons)
0.5	43×10^6	30 -----
10	24×10^6	17 $\text{cm}^2 \times \text{sec}$

Linear Absorption Coefficients μ (cm^{-1})				
Energy (MEV)	Water	Concrete	Iron	Lead
0.5	0.090	0.21	0.63	1.7
1.0	0.067	0.15	0.44	0.77
1.5	0.057	0.13	0.40	0.57
2.0	0.048	0.11	0.33	0.51
2.5	0.042	0.097	0.31	0.49
3.0	0.038	0.088	0.30	0.47

Table 2: Saturated Steam: Pressure Table

Table with 13 columns: Abs Press Lb/Sq In p, Temp Fahr t, Specific Volume (Sat Liquid v_f, Evap v_fg, Sat Vapor v_g), Enthalpy (Sat Liquid h_f, Evap h_fg, Sat Vapor h_g), Entropy (Sat Liquid s_f, Evap s_fg, Sat Vapor s_g), and Abs Press Lb/Sq In p. Rows range from 0.0001 to 3200 lb/sq in.

*Critical pressure

