NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Susquehanna Steam Electric Station - Unit 2	0 5 0 0 0 3 8 8	1 OF 0 1

TITLE (4)

HPCI Inop. Due to Leaking Pressure Control Valve.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 1	2 2	8 6	8 6	0 0 2	0 0	0 2	2 0	8 6		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
	1	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	0 6 6	20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
L.A. Kuczynski, Nuclear Plant Specialist-Level III	AREA CODE 7 1 1 7 5 4 2 - 3 7 5 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	B Q	P C V	T 0 2 0	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 22, 1986, the routinely scheduled quarterly flow surveillance test of the High Pressure Coolant Injection (HPCI) system (EIIS Code: BQ) commenced at 1905. The system was shutdown at 1910 when PCV-2F035 developed a leak of approximately 5 gpm due to a tear in its main diaphragm. (PCV-2F035, Target Rock Model 75KK-404, controls the flow of coolant to the HPCI lube oil cooler and the HPCI barometric condenser.) HPCI was declared inoperable at 2310 after an evaluation of the leak was completed by Operations and Engineering personnel. The diaphragm was replaced on January 23, 1986, and the system was returned to operable status at 0731 on January 24, 1986, after the successful completion of the surveillance test.

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PP&L Pennsylvania Power & Light Company
Two North Ninth Street - Allentown, PA 18101 • 215/770-5151

February 20, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 86-002-00
ER 100450 FILE 841-23
PLAS- 148

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 86-002-00. This event was determined reportable per 10CFR50.73(a)(2)(v), in that a single train safety system (the High Pressure Coolant Injection System) was declared inoperable due to a leaking pressure control valve.

T.M. Crimmins, Jr.
Superintendent of Plant-Susquehanna

LAK/pjg

cc: Dr. Thomas E. Murley
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