

April 29, 1988

Docket No. 50-267

Mr. R. O. Williams, Jr.
Vice President, Nuclear Operations
Public Service Company of Colorado
P. O. Box 840
Denver, Colorado 80201-0840

Dear Mr. Williams:

SUBJECT: FIRE PROTECTION PLAN APPROVAL (TAC NO. 66508)

We are continuing our review of your submittal dated December 15, 1987 (P-87422) concerning the Fort St. Vrain (FSV) Fire Protection Plan (FPP). In order to continue this review, we are requesting certain additional information, as described in the enclosure to this letter. Some of the information requested results from the staff's on-site audit of the FSV-FPP on April 4-6, 1988.

Please provide the information requested within 45 days of the date of this letter. The information requested in this letter affects fewer than 10 respondents, therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,
/s/

Kenneth L. Heitner, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Public Service Company of Colorado

Fort St. Vrain

cc:

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Denver, Colorado 80202-2413

REQUEST FOR ADDITIONAL INFORMATION
FORT St. VRAIN NUCLEAR STATION
FIRE PROTECTION PROGRAM PLAN REVIEW

1. During the 1983 Appendix R audit, a concern was raised concerning the routing of the power feed for the electric fire pump through the area containing the diesel fire pump. During the site visit of April 4-6, 1988 it was demonstrated that a redundant feed from the ACM had been added, however, it appeared that the electrical control circuit for the electric fire pump still ran through the diesel room. Please provide conduit routing diagrams and a description which demonstrates the separation of all necessary power and control cables for the electric pump from the room containing the diesel fire pump.
2. Provide drawings and any other necessary documentation which demonstrates that detectors throughout the plant are located in compliance with NEPA 72A and 72E. This information should also address the adequate spacing of detectors when ionization and beam detectors are used in the same area.
3. Provide justification for the use of non-IEEE 383 cables in the plant. This response should address items such as the location of this cable, combustible loadings in these areas, and why this cable does not impact plant safety.
4. Submit drawings which illustrate the installed configurations of fire dampers inside barriers. Also provide a description of the several types of dampers used in the plant and information which demonstrates that these dampers are rated equivalent to the barrier.
5. During the April 4-6, 1988 site visit, the Appendix A comparison portion of the program plan was discussed with licensee representatives. The concern raised by the NRC was that a number of sections within this comparison use words such as "the intent of this guideline has been met", however, no explanation is presented of why the guideline is not strictly met. For sections where full compliance does not exist, provide, within the comparison, an explanation of the deviation and adequate justification for the deviation. Where justification is contained in lengthy analyses, reference to these documents would be satisfactory.
6. It could not be determined during the April 4-6, 1988 visit whether or not the recently installed emergency lighting

system provides adequate illumination for safe shutdown manual operator actions and for access and egress to areas where these manual actions take place. It is understood that this lighting system will be evaluated by the licensee during an upcoming drill. Although no action is necessary at this time, the issue of the adequacy of the emergency lighting system will remain an open item pending the performance of this evaluation.