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AP600 REACTOR COOLANT SYSTEM LEAK PRA EVALUATION

October 1993

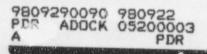


Table of Contents

		PAGE
1.0	Introduction	1
2.0	Purpose	1
3.0	Analysis Create the RCS Leak Event Tree	2
3.2	Quantify the Core Damage Frequency for the RCS Leak Event	2 3
3.3	Update the Containment Event Trees (CET)	3
4.0	Conclusion	3

PAGE

1.0 INTRODUCTION

During an NRC/EPRI meeting on the Regulatory Treatment of Non-Safety Systems (RTNSS) held on November 18, 1992, R. Youngblood of Brookhaven National Laboratories presented an evaluation of reliability goals and safety systems performance. This presentation included the identification of 39 reactor coolant system (RCS) leakage events with leak rates between 1 and 100 gallon per minute (gpm) and 48 events with leak rates less than 1 gpm at pressurized water reactors (PWR) during the period from 1987 to 1992 which included approximately 421 reactor years of PWR operation. Apparent concern over such events with respect to the AP600 design is that since the AP600 chemical and volume control system is a nonsafety-related system, these leakage events are not adequately addressed by the AP600 Probabilistic Risk Assessment (PRA). The analysis presented in the AP600 PRA report covers the range of loss of coolant accidents from a 160 gpm leak (very small LOCA category) up through the double-ended severance of the largest pipe in the primary coolant system. This RCS leakage event is now included in the AP600 baseline PRA which improves accuracy of the focused PRA for the regulatory treatment of nonsafety systems.

2.0 PURPOSE

The purpose of this document is to discuss the resolution of the RCS leakage issue identified above. The two ranges of leak rates identified are addressed separately.

For the 0 to 1 gpm leak range, a qualitative evaluation is performed. For a 1 gpm leak, with no makeup flow, it takes more than 60 hours to empty the pressurizer and more than 190 hours to drain the volume of water from a core makeup tank (CMT) to the automatic depressurization system (ADS) actuation water level setpoint. That is, with both CMTs operating, it would take more than 18 days to get to the ADS actuation setpoint. Failure to recognize and identify such a leak and neither correct the situation nor manually bring the plant to a safe shutdown condition is sufficiently unlikely that it is not in the scope of typical risk assessment studies.

For the 1 to 100 gpm leak rate range, a quantitative evaluation of the RCS leakage event is performed. The analysis includes: calculating a leakage initiating event frequency, calculating the probability of repairing the chemical and volume control system (CVCS) makeup system, developing an RCS leak event tree, quantifying the event tree, and finally incorporating the results into the current Baseline FRA results. The release frequency is also reanalyzed due to the additional core damage sequences that propagate through to the containment event trees.

This RCS leakage event is an addition to the other initiating events currently included in the baseline AP600 PRA. The core damage frequency and the release frequency associated with this event are a small addition to those values submitted to the NRC on June 26, 1992 in the AP600 Probabilistic Risk Assessment Report. The AP600 PRA report tables dealing with core damage frequency (i.e., Tables 7-1 and 8-1) change as a result of including the RCS leak event. The revisions to those tables are included in this letter as Tables 1 and 2. The increase in release frequency is sufficiently small that it does not change any reported results.

3.0 ANALYSIS

The analysis of core damage frequency and release frequency for the RCS leakage event is performed in the distinct steps described in Subsections 3.1, 3.2 and 3.3.

3.1 Create the RCS Leak Event Tree

The RCS leak event, defined as an RCS leak rate less than or equal to 100 gpm combined with failure of the CVCS to provide makeup to the RCS, is actually a very small loss of coolant accident with the distinction that the leak is sufficiently small that a "long" period of time is required to deplete the primary coolant system to the extent that the water level in the core makeup tanks will cause actuation of the ADS. The length of this time period is a function of the leak rate but in any case is long enough to permit repair of the CVCS. A new event tree, Reactor Coolant System Leak, was created and is shown in Figure 1. This event tree is based on Figure F-18 of the AP600 PRA report (very small loss of coolant accident event tree). Figure F-18 was modified to account for the possibility of repairing the CVCS if the core makeup tanks operated successfully. The RCS leak initiating event is given the acronym of SL.

This study is done for a breach of the primary coolant system of such size that results in an initial leak flow rate of 50 gpm. For such a leak, successful operation of the CMTs would preclude ADS actuation for more than 5 hours, during which time the CVCS could be repaired. This time is calculated on the basis of the pressurizer emptying to the safety injection signal setpoint and then one CMT injecting to get to the ADS actuation setpoint for CMT water level. By using 1 out of 2 CMTs as the success criterion, fault tree CM2SL from the "June 26, 1992" PRA could be used for this study. The probability of failing to repair the CVCS in 4 to 6 hours was calculated to be 0.50 and this value is used in the core damage frequency analysis. Repair of the CVCS is given the basic event identifier OTH-CVCSFIX.

The initiating event frequency for the RCS leak event (IEV-SL) is based on data reported by EPRI for PWRs operating during the 1987 to 1992 time period. These data include 39 leakage events that had a leak rate in the 1 gpm to 100 gpm range. The these 39 events were evaluated to determine whether or not each event was possible on the AP600 design. As a result, 30 of the events were screened out (for example, RCS pump seal leaks were eliminated because AP600 does not have RCP seals) leaving nine events that are possible on the AP600. Of these 9 RCS leakage events, 5 occurred at-power and 4 occurred during shutdown. The impact of these leaks is different for at-power and shutdown operation; The time available for corrective action at shutdown is significantly longer than at-power and therefore contribute less to any change in core damage frequency. The maximum leak flow rate of the 5 at-power leakage events that could have occurred on the AP600 is 10 gpm. Because of this, using a leak flow rate of 50 gpm in the calculations to represent the 1 to 100 gpm range is reasonable and conservative with respect to historical data.

The RCS leak initiating event is defined as an event having leakage from the RCS less than 100 gpm combined with failure of the chemical and volume control system (CVCS) makeup pumps. The 5 events from the EPRI data base represent 421 reactor years of PWR operation so the frequency of an RCS leak for AP600 is 5/421 = 1.2E-2 events per year.

The unavailability of the makeup system (where success of the system is either of the two makeup pumps starting and running for 24 hours) is simply taken as the failure probability of the CSLOCA fault tree from the June 26, 1992 PRA. This value is 5.4E-3. Therefore, the RCS leak initiating event frequency is

(1.2E-2) (5.4E-3) = 6.5E-5 events per year.

A revised list of AP600 initiating event group frequencies is shown in Table 1.

3.2 Quantify the Core Damage Frequency for the RCS Leak Event

The RCS leak event tree was quantified and the resulting core damage frequency is 1.6E-9 events per year. The core damage frequency associated with the RCS leak initiating event is combined with the other AP600 initiating events and the revised total core damage frequency is 3.4E-7 events per year. Table 2 shows the revised list of initiating events contributing to core damage (Baseline, at-power conditions) and Table 3 shows the revised dominant accident sequence cutsets for this case. An importance analysis is done to rank the revised set of initiating events and the results are shown in Table 4.

3.3 Update the Containment Event Trees (CET)

The revised release frequency is calculated by quantifying the containment event trees with the core damage frequency information that includes the RCS leak event. The only containment event tree release frequency that was changed by the addition of the RCS leak initiating event is impaired containment (CI). The CI release frequency was 2.0132-8 in the June 26, 1992 analysis and increases to 2.017E-8 in this revised analysis due to the addition of the RCS leak event.

4.0 CONCLUSION

The evaluation of relatively small reactor coolant system leaks was performed in a conservative manner with respect to the assumed leak rate and probability of returning the failed primary coolant makeup system back into service. The results of this study show a 0.5 percent increase in core damage frequency and a 0.2 percent increase in the frequency of release from containment. This category of initiating events will be included in future AP600 PRA studies and PRA reports.

TABLE 1 AP600 INITIATING EVENT GROUP FREQUENCIES

EVENT	FREQUENCY (Event/Year)
Manual Shutdown	negligible
Transients	
Turbine trip or spurious reactor trip	1.40
Loss of feedwater flow	0.46
Secondary to primary side power mismatch	0.054
Core power excursion	4.47E-3
Spurious S-signal	0.085
Loss of component cooling system	0.014
Loss of service water system	0.026
Loss of compressed air	0.014
Main steam line break downstream of main steam isol. valves	6.0E-4
Main steam line break upstream of main steam isol. valves	3.7E-4
Main steam line safety valve stuck open	1.2E-3
Loss of Offsite Power	0.086
Loss of Coolant Accidents	
Large loss of coolant accident	9.7E-5
Medium loss of coolant accident	5.6E-4
Core makeup tank line break	1.3E-4
Safety injection line break	1.2E-4
Small loss of coolant accident	5.2E-4
Very small loss of coolant accident	5.5E-4
Reactor coolant system leak	6.5E-5
Passive residual heat removal tube rupture	5.0E-3
Steam generator tube rupture	5.2E-3
Reactor vessel rupture	3.0E-8
Interfacing loss of coolant accidents	negligible
Large break outside containment	negligible
Anticipated Transients Without Scram	0.53

TABLE 2 INITIATING EVENTS CONTRIBUTING TO CORE DAMAGE (Base Case - At Power)

INITIATING EVENT	CORE DAMAGE FREQUENCY (Events per Year)	PERCENTAGE OF TOTAL
Transients (except LOOP): Turbine/reactor trip (TT) Others	4.3E-8 2.9E-8	12.8 8.9
LOOP (TE)	3.0E-9	0.9
Small LOCA (S2)	2.3E-8	6.9
Very small LOCA (S2S)	1.2E-8	3.5
Reactor coolant system leak (SL)	1.6E-9	0.5
PRHR tube rupture (S2P)	4.2E-8	12.4
Medium LOCA (S1)	1.2E-8	3.5
Safety injection line break (S1S)	7.3E-8	21.6
CMT line break (S1C)	2.7E-9	0.8
Large LOCA (A)	1.6E-8	4.7
SG tube rupture (V2)	2.6E-9	0.8
ATWS (TFA)	4.6E-8	13.8
Vessel rupture (VR)	3.0E-8	8.9
TOTAL	3.4E-7	100.0

	SEQUENCE	PERCENT	SEQUENCE				SEQUENCE
NUMBER	PROBABILITY	CONTRIB	DESCRIPTION				IDENTIFIER
	*********		*************	**************			**************
1			FAILURE OF	I OF I GRAVIII	INJECTION LINE		SYS-IWIA
2			AMSAC SYSTEM	FAILS			SYS-AMSAC
3	3.23E-08	8.97	PASSIVE RHR TUBE CSLOCA AND PRI	RUPTURE TOP EVENT NODES	INITIATING EVENT	OCCURS	IEV-S2P SYS-XCSLPRI
						(S2 W/PRHR & CMT (LOCA/TRANSIENT)	
4	3.06E-08	8.49	TURBINE/REACTOR COND AND SFW FAILURE OF 2/2 MANUAL ADS MANUAL ADS ACTUA	TRIP /L.RCS FLOW AND PRT SYSTEMS CMTS FOR ACT. FAILS FULL FAILS PART'L RCS	INITIATING EVENT FAIL TRANSIENTS RCS DEPRESSURIZ. DEPRESSURIZATION		IEV-TT SYS-XCSP SYS-CM2AB SYS-ADT SYS-AD1
5	3.00E-08	8.33	VESSEL RUPTURE				IEV-VR
6	2.58E-08	7.15	CLASS 2 EARLY FAILURE TO FAILURE OF IRWST	CONTAINMENT RECOVER CONTAIN. WATER MAKEUP FOR	LEAK OCCURS ISOLATION IN LONG TERM CORE	IEV FREQUENCY THE LONG TERM COOLING (S2 W/PRHR & CMT	SYS-IEC2E SYS-CIR SYS-IWTM
7	1.51E-08	4.19	LARGE LOCA FAILURE OF	INITIATING EVENT 2 OF 2 GRAVITY	OCCURS INJECTION LINES		IEV-A SYS-IW2AB
8			AUTOMATIC ADS	ACT. FAILS PARTL	RCS DEPRESSURIZ.	(S2 W/PRHR & CMT	SYS-ADV
9	1.11E-08	3.08	LOSS OF FW TO COND AND SFW FAILURE OF 2/2 MANUAL ADS MANUAL ADS ACTUA	STEAM CENERATOR AND PRT SYSTEMS CMTS FOR ACT. FAILS FULL FAILS PART'L RCS	INITIATING EVENT FAIL TRANSIENTS RCS DEPRESSURIZ. DEPRESSURIZATION		IEV-TF SYS-XCSP SYS-CM2AB SYS-ADT SYS-AD1
10			TURBINE / REACTOR	TRIP /L. RCS FLOW	INITIATING EVENT	OCCURS (TRANS WITH CMT) (LOCA/TRANSIENT)	TEN mm
11	6.62E-09	1.84				OCCURS	

			DELETE NON- AUTOMATIC ADS AUTOMATIC ADS	SENSICAL CUTSETS ACT. FAILS FULL ACT. FAILS PARTL	FROM V2 RCS DEPRESSURIZ. RCS DEPRESSURIZ.		DEL-V2DEL SYS-ADS SYS-ADV
12	6.13E-09	1.70	MEDIUM LOCA RNS FAILS TO AUTOMATIC ADS	INITIATING EVENT OPERATE IN ACT. FAILS FULL	OCCURS INJECTION MODE RCS DEPRESSURIZ.	(LOCA/TRANSIENT) (S1 WITH CMT)	IEV-SI SYS-RNR SYS-ADM
13	5.44E-09	1.51	MEDIUM LOCA RNS FAILS TO FAILURE OF	INITIATING EVENT OPERATE IN 2 OF 2 GRAVITY	OCCURS INJECTION MODE INJECTION LINES	(LOCA/TRANSIENT)	IEV-S1 SYS-RNR SYS-IW2AB
14	5.33E-09	1.48	VERY SMALL LOCA RNS FAILS TO FAILURE OF	INITIATING EVENT OPERATE IN 2 OF 2 GRAVITY	OCCURS INJECTION MODE INJECTION LINES	(LOCA/TRANSIENT)	IEV-S2S SYS-RNR SYS-IW2AB
15	5.23E-09	1.45	PASSIVE RHR TUBE ADV EVENT TREE	RUPTURE NODE IS	INITIATING EVENT SUCCESSFUL	OCCURS (AUTO DEPRESS.)	IEV-S2P DEL-ADV

			TABLE 3						
DOMINANT	ACCIDENT	SEQUENCE	CUISETS	-	INCLUDES	RCS	LEAK	EVENT	

NUMBER	SEQUENCE PROBABILITY	CONTRIB	DESCRIPTION				SEQUENCE IDENTIFIER
				TOP EVENT NODES			SYS-XPRIRNR SYS-ADS
16	5.04E-09	1.40	SMALL LOCA RNS FAILS TO FAILURE OF	INITIATING EVENT OPERATE IN 2 OF 2 GRAVITY	OCCURS INJECTION MODE INJECTION LINES	(LOCA/TRANSIENT)	IEV-S2 SYS-RNR SYS-IW2AB
17	3.55E-09	. 98	VERY SHALL LOCA CVCS MAKE-UP AUTOMATIC ADS RMS FAILS TO	INITIATING EVENT FAILS DURING ACT. FAILS FULL OPERATE IN	S2S/S2P (VERY RCS DEPRESSURIZ.	SMALL LOCA) (S2 W/PRHR & CMT (LOCA/TRANSIENT)	IEV-S2S SYS-CSLOCA SYS-ADS SYS-RNR
18	3.49E-09	.97	SAFETY INJECTION AUTOMATIC ADS	LINE BREAK ACT. FAILS FULL	INITIATING EVENT RCS DEPRESSURIZ	OCCURS (S1 WITH CMT)	IEV-SIS SYS-ADM
19	3.01E-09	. 84	LOSS OF FW TO COND AND SFW AUTOMATIC ADS RNS FAILS TO	ACT. FAILS FULL	RCS DEPRESSURIZ.	OCCURS (TRANS WITH CHT; (LOCA/TRANSIENT)	IEV-TF SYS-XCSP SYS-ADA SYS-RNR
20	2.578-09	.71	PASSIVE RHR TUBE PRI AND RNR FAILURE OF	RUPTURE TOP EVENT NODES 2 OF 2 GRAVITY	INITIATING EVENT FAIL INJECTION LINES	OCCURS	IEV-S2P SYS-XPRIRNR SYS-IW2AB
21	2.398-09	. 66	STEAM GENERATOR DELETE NON- SGTR CONTINUES AUTOMATIC ADS AUTOMATIC ADS	SENSICAL CUTSETS		OCCURS SEQUENCES (S2 W/PRHR & CMT (S2 W/PRHR & CMT	
22	2.13E-09	.59	FAULURE OF 212	PRHR SYSTEM TO CHTS FOR SMALL	REMOVE DEC. HEAT	EVENT OCCURS FROM RPV (TRANS) (TRANS W/O CMT) (TRANS W/O CMT)	SYS-IECTSOV SYS-PRT SYS-CM2SL SYS-ADT SYS-AD1
23	1.73E-09	. 48	SMALL LOCA FAILURE TO MANUAL ADS ACTUA MANUAL ADS ACT	INITIATING EVENT TRIP ALL FOUR FAILS FULL RCS FAILS PARTL RCS	RCS PUMPS DEPRESSUR, SLOCA	W/PRER, W/O CMT) W/PRER W/O CMT)	IEV-S2 SYS-RCSL SYS-ADN SYS-ADZ
24	1.41E-09	. 39	CMT LINE BREAK RNS FAILS TO AUTOMATIC ADS	INITIATING EVENT OPERATE IN ACT. FAILS FULL	INJECTION MODE	(LOCA/TRANSIENT) (SI WITH CMT)	IEV-SIC SYS-RNR SYS-ADM
25	1.25E-09	. 35	CMT LINE BREAK	INITIATING EVENT			IEV-SIC

9	1			RNS FAILS TO FAILURE OF	OPERATE IN 2 OF 2 GRAVITY	INJECTION MODE INJECTION LINES	(LOCA/TRANSIENT)	SYS-RNR SYS-IW2AB
	26	1.19E-09	. 33	SMALL LOCA FAULURE OF 2/2 MARUAL ADS ACTUA MANUAL ADS ACT.	INITIATING EVENT CMTS FOR SMALL FAILS FULL RCS FAILS PARTL RCS	OCCURS LOCAS DEPRESSUR. SLOFA DEPRESSUR. (S.		IEV-S2 SYS-CM2SL SYS-ADN SYS-ADZ
	27	1.19E-09	• .33			INITIATING EVENT RCS PUMPS RCS DEPRESSURIZ. DEPRESSURIZATION	(TRANS W/O CMT)	IEY-TS SYS-XWF SYS-RCSL SYS-ADT SYS-AD1
	28	1.18E-09	. 33	SECONDARY TO COND AND SFW FAILURE OF 2/2	PRIMARY SIDE AND PRT SYSTEMS CMTS FOR	POWER MISMATCH FAIL TRANSIENTS	IN. EVENT OCCURS	IEV-TM SYS-XCSP SYS-CM2AB

NUMBER	SEQUENCE PROBABILITY		SEQUENCE DESCRIPTION				SEQUENCE IDENTIFIER
				ACT. FAILS FULL FAILS PART'L RCS	RCS DEPRESSURIZ. DEPRESSURIZATION	(TRANS W/O CMT) (TRANS W/O CMT)	
29	1.16E-09	. 32	VERY SMALL LOCA CVCS MAKE-UP FAILURE TO MANUAL ADS ACTUA MANUAL ADS ACT.	INITIATING EVENT FAILS DURING TRIP ALL FOUR FAILS FULL RCS FAILS PARTL RCS	OCCURS S2S/S2P (VERY RCS PUMPS DEPRESSUR. SLOCA DEPRESSUR. (S2	SMALL LOCA) W/PRHR, W/O CMT) W/PRHR W/O CMT)	IEV-S2S SYS-CSLOCA SYS-RCSL SYS-ADH SYS-ADZ
30			FAILURE OF SFW FAULURE OF 2/2 MANUAL ADS ACTUA MANUAL ADS	TO SUPPLY SG A CMTS FOR TE FAILS FULL RCSL ACT. FAILS PARTL	30 MINUTES REMOVE DEC. HEAT FROM CST - DEPRESSURIZATION RCS DEPRESSURIZ.	OCCURS FOLLOWING LOSP FROM RPV (LOSP) NO DST (TE W/O CMT) (TE W/O CMT)	SYS-SFWP SYS-CM2P SYS-ADL SYS-ADR
31	9.07E-10	. 25				OCCURS AIR- FROM RPV (TRANS) (TRANS W/O CMT) (TRANS W/O CMT)	
32	8.34E-10	. 23	OTH-CVCSFIX	ACT. FAILS FULL ACT. FAILS PARTL	RCS DEPRESSURIZ. RCS DEPRESSURIZ.	(S2 W/PRJR & CMT (S2 W/PRHR & CMT	IEV-SL OTH-CVCSFIX SYS-ADS SYS-AD↓
33	8.30E-10	.23		MAIN CONDENSOR POWER MISMATCH PRHR SYSTEM TO	AVAILABLE EVENTS IN WHICH REMOVE DEC. HEAT	IN. EVENT OCCURS SFW IS ALSO LOST FAOM RPV (TRANS) (TRANS W/O CMT) (TRANS W/G CMT)	
34	8.24E-10	.23	FAILURE OF 2/2	CMTS FOR LOCAS		OCCURS SFW IS ALSO LOST FROM RPV (TRANS) (TRANS W/O CMT) (TRANS W/O CMT)	CHC CHOX
35	7.14E-10	. 20	PASSIVE RHR TUBE ADZ EVENT TREE	RUPTURE NODE IS	INITIATING EVENT SUCCESSFUL	OCCURS (AUTO DEPRESS.)	IEV-S2P DEL-ADZ

P			FAILURE TO PRHR ISOLATION MANUAL ADS ACTUA	TRIP ALL FOUR FAILURE FOLLOW. FAILS FULL RCS		CVS UNAVAILABLE W/PRHR, W/O CMT)	SYS-RCSL SYS-PRII SYS-ADN
			RNS FAILS TO	OPERATE IN	INJECTION MODE	(LOCA/TRANSIENT)	SYS-RNR
36	6.885-10	. 15	VERY SMALL LOCA ADV EVENT TREE AUTOMATIC ADS RNS FAILS TO	INITIATING EVENT NODE IS ACT. FAILS FULL OPERATE IN	OCCURS SUCCESSFUL RCS DEPRESSURIZ. INJECTION MODE	(AUTO DEPRESS.) (S2 W/PRHR & CMT (LOCA/TRANSIENT)	IEV-S2S DEL-ADV SYS-ADS SYS-RNR
37	6.64E-10	.18	CONSEQUENTIAL FAILURE OF RNS FAILS TO	SC TUBE RUPTURE 2 OF 2 CRAVITY OPERATE IN	INITIATING EVENT INJECTION LINES INJECTION MODE	OCCURS (LOCA/TRANSIENT)	SYS-IECV2 SYS-IW2AB SYS-RNR
38	6.50E-10	. 18	SMALL LOCA AUTOMATIC ADS RNS FAILS TO	INITIATING EVENT ACT FAILS FULL OPERATE IN	OCCURS RCS DEPRESSURIZ. INJECTION MODE	(S2 W/PRHR & CMT (LOCA/TRANSIENT)	IEV-S2 SYS-ADS SYS-RNR

			TABLE 3						
DOMINANT	ACCIDENT	SEQUENCE	CUTSETS	-	INCLUDES	RCS	LEAK	EVENT	

	NUMBER	SEQUENCE PROBABILITY		SEQUENCE				SEQUENCE IDENTIFIER
**	39	6.43E-10	. 18	LOUS OF SERVICE SFW AND PRT FAIL TRE OF 2/2 MANUEL ADS	WATER SYSTEM SYSTEMS FAIL	INITIATING EVENT	OCCURS (TRANS W/O CHT)	SYS-XWP
ALC: NO ALC: NO	40	6.18E-10	17	SPUTIOUS "S" SFW AND PRT AUTOMATIC ADS AUTOMATIC ADS	SIGNAL SYSTEMS FAIL	INITIATING EVENT	OCCURS	IEV-TS SYS-XWP
and the second	41	5.82E-10	. 16	CVCS MAKE-UP AUTOMATIC ADS AUTOMATIC ADS	INITIATING EVENT FAILS DURING ACT. FAILS FULL ACT. FAILS PARTL	OCCURS S2S/S2P (VERY RCS DEPRESSURIZ. RCS DEPRESSURIZ.	SMALL LOCA) (S2 W/PRHR & CMT (S2 W/PRHR & CMT	IEV-S2S SYS-CSLOCA SYS-ADS SYS-ADV
States Second	42			LOSS OF OFFSITE GRID IS NOT FAILURE OF THE FAILURE OF SFW AUTOMATIC ADS AUTOMATIC ADS	PGWER RECOVERED WITHIN PRHR SYSTEM TO TO SUPPLY SG A ACT. FAILS FULL ACT. FAILS PARTL	INITIATING EVENT 30 MINUTES REMOVE DEC. HEAT FROM CST - RCS DEPRESSURIZ. RCS DEPRESSURIZ.	OCCURS FOLLOWING LOSP FROM RPV (LOSP) NO DST (LOSP WITH CMT) (TE WITH CMT)	IEV-TE OTH-RG5 SYS-PRP SYS-SFWP SYS-ADAL SYS-ADRA
A NUMBER OF THE OWNER.	43	5.38E-10	. 15	TURBINE/REACTOR STEAM DUMP AND FWT AND SFWTD FAILURE OF 2/2 MANUAL ADS MANUAL ADS ACTUA	TRIP /L.RCS FLOW MAIN CONDENSOR AND PRT SYSTEMS CMTS FOR ACT. FAILS FULL FAILS PART'L RCS	INITIATING EVENT AVAILABLE FAIL TRANSIENTS RCS DEPRESSURIZ. DEPRESSURIZATION	(TRANS W/O CMT) (TRANS W/O CMT)	IEV-TT DEL-COND SYS-XFSP SYS-CM2AB SYS-ADT SYS-AD1
and a set of the	44			CONSEQUENTIAL FAILURE OF THE AUTOMATIC ADS AUTOMATIC ADS	MSL SV STUCK PRHR SYSTEM TO ACT. FAILS FULL ACT. FAILS PART'	OPEN INITIATING REMOVE DEC. HEAT RCS DEPRESSURIZ. RCS DEPRESSURIZ.	EVENT OCCURS FROM RPV (TRANS) (TRANS WITH CMT) (TRANS WITH CMT)	SYS-IECTSOV SYS-PRT SYS-ADA SYS-ADIA
CAN DIS TRANS	45	3.54E-10	. 10	CONS®QUENTIAL RNS FAILS TO AUTOMATIC ADS	MEDIUM LOCA OPERATE IN ACT. FAILS FULL	INITIATING EVENT INJECTION MODE RCS DEPRESSURIZ.	OCCURS (LOCA/TRANSIENT) (S1 WITH CMT)	SYS-IECSI SYS-RNR SYS-ADM
a state to be	46	3.53E-10	. 10	CONSEQUENTIAL RNS FAILS TO FAILURE OF	MEDIUM LOCA OPERATE IN 2 OF 2 GRAVITY	INITIATING EVENT INJECTION MODE	OCCURS (LOCA/TRANSIENT)	SYS-IECSI SYS-RNR SYS-IW2AB
Mississi	47	3.19E-10	. 09	SECONDARY TO COND AND SFW AUTOMATIC ADS	PRIMARY SIDE AND PRT SYSTEMS ACT. FAILS FULL	POWER MISMATCH FAIL RCS DEPRESSURIZ.	IN. EVENT OCCURS (TRANS WITH CMT)	IEV-TM SYS-XCSP SYS-ADA

				RNS FAILS TO	OPERATE IN	INJECTION MODE	(LOCA/TRANSLENT)	SYS-RHR
	48	3.06E-10	.08	LOSS OF FW TO	STEAM GENERATOR	INITIATING EVENT	OCCURS	IEV-TF
					SYSTEMS FAIL CMTS FOR ACT. FAILS FULL		IS ALSO LOST (TRANS W/O CMT) (TRANS W/O CMT)	OTH-VAL2 SYS-XWP SYS-CM2AB SYS-ADT SYS-AD1
4	49	2.92E-10	.08		COOLING SYSTEM AND PRT SYSTEMS CMTS FOR ACT. FAILS FULL	INITIATING EVENT FAIL TRANSIENTS RCS DEPRESSURIZ.	OCCURS (TRANS W/O CMT)	IEV-TCW SYS-XCSP SYS-CM2AB SYS-ADT
	50	2.918-10	.08	IEV-SL OTH-CVCSFIX RNS FAILS TO FAILURE OF	OPERATE IN 2 OF 2 CRAVITY	INJECTION MODE INJECTION LINES	(LOCA/TRANSIENT)	IEV-SL OTH-CVCSFIX SYS-RNR SYS-IW2AB

TABLE 3 DOMINANT ACCIDENT SEQUENCE CUTSETS - INCLUDES RCS LEAK EVENT

NUMBER	SEQUENCE PROBABILITY		SPQUENCE DESCRIPTION				SEQUENCE
	*********		************	*************			
51	2.85E-10	. 08	SFW AND PRT	SYSTEMS FAIL		OCCURS (TRANS W/O CMT) (TRANS W/O CMT)	SYS-XWP
52	2.71E-10	. 08	PASSIVE RHR TUBE CSLOCA AND PRII AUTOMATIC ADS AUTOMATIC ADS	RUPTURE TOP EVENT NODES ACT. FAILS FULL ACT. FAILS PAKIL	INITIATINC EVENT FAIL RCS DEPRESSURIZ. KCS DEPRESSURIZ.	OCCURS (S2 W/PRHR & CMT (S2 W/PRHR & CMT	IEV-S2P SYS-XCSLPRII SYS-ADS SYS-ADV
53	2.59E-10	.07				OCCURS (ATWS)	
54	2.55E-10	.07				W/PRHR, W/O CMT) W/PRHR W/O CMT)	
55			CONSEQUENTIAL FAILURE OF THE FAULURE OF 2/2	SC TUBE RUPTURE PRHR SYSTEM TO CMTS FOR SMALL	INITIATING EVENT REMOVE DEC. HEAT LOCAS	OCCURS FROM RPV (S2)	SYS-IECV2 SYS-PRL SYS-CM2SL
56	2.478-10	.07	MAIN STEAMLINE FAILURE OF THE FAULURE OF 2/2 MANUAL ADS MANUAL ADS ACTUA	SV STUCK OPEN PRHR SYSTEM TO CMTS FOR SMALL ACT. FAILS FULL FAILS PART'L RCS	INITIATING EVENT REMOVE DEC. HEAT LOCAS RCS DEPKESSURIZ. DEPRESSURIZATION	(LOCA/TRANSIENT) OCCURS FROM RPV (TRANS) (TRANS W/O CMT) (TRANS W/O CMT)	IEV-TSOV SYS-PRT SYS-CM2SL SYS-ADT SYS-AD1
57	2.27E-10	.06	LOSS OF COMPRES. FAILURE OF 2/2 FAILURE OF THE AUTOMATIC ADS AUTOMATIC ADS	AIR SYSTEM SFW TRAINS PRHR SYSTEM TO ACT. FAILS FULL ACT. FAILS PART'	INITIATING EVENT -NO INSTRUMENT REMOVE DEC. HEAT RCS DEPRESSURIZ. RCS DEPRESSURIZ.	OCCURS AIR- FROM RPV (TRANS) (TRANS WITH CMT) (TRANS WITH CMT)	IEV-TCA SYS-SFM SYS-PRT SYS-ADA SYS-ADIA
58	2.208-10	.06	CONSEQUENTIAL DELETE NON- FAILURE TO MANUAL ADS ACTUA MANUAL ADS ACT.	SG TUBE RUPTURE SENSICAL CUTSETS TRIP ALL FOUR FAILS FULL RCS FAILS PARTL RCS	INITIATING EVENT FROM V2 RCS PUMPS DEPRESSUR. SLOCA DEPRESSUR. (S2	OCCURS SEQUENCES W/PRHR, W/O CMT) W/PRHR W/O CMT)	SYS-IECV2 DEL-V2DEL SYS-RCSL SYS-ADN SYS-ADZ
59			SECONDARY TO STEAM DUMP AND			IN. EVENT OCCURS	IEV-TM DEL-COND

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-			FRACTION OF FAILURE OF THE AUTOMATIC ADS	PRHR SYSTEM TO	REMOVE DEC. HEAT	SFW IS ALSO LOST FROM RPV (TRANS) (TRANS WITH CMT)	OTH-VAL3 SYS-PRT SYS-ADA
			AUTOMATIC ADS	ACT. FAILS PART'	RCS DEPRESSURIZ.	(TRANS WITH CMT)	SYS-ADIA
60	1.958-10	.05	TURBINE/REACTOR COND AND SFW AUTOMATIC ADS AUTOMATIC ADS	AND PRT SYSTEMS ACT FAILS FULL	FAIL RCS DEPRESSURIZ.	OCCURS (TRANS WITH CMT) (TRANS WITH CMT)	IEV-TT SYS-XCSP SYS-ADA SYS-ADIA
61	1.95E-10	.05	SPURIOUS "S" FPACTION OF FAILURE OF THE AUTOMATIC ADS AUTOMATIC ADS	PRHR SYSTEM TO ACT. FAILS FULL	EVENTS IN WHICH REMOVE DEC. HEAT RCS DEPRESSURIZ.	OCCURS SFW IS ALSO LOST FROM RPV (TRANS) (TRANS WITH CMT) (TRANS WITH CMT)	IEV-TS OTH-VAL4 SYS-PRT SYS-ADA SYS-ADIA
62	1.905-10	. 05	PASSIVE RHR TUBE CSLOCA AND PRI1	RUPTURE TOP EVENT NODES	INITIATING EVENT FAIL	OCCURS	IEV-S2P SYS-XCSLPRII

			TABLE 3						
DOMINANT	ACCIDENT	SEQUENCE	CUTSETS	-	INCLUDES	RCS	LEAK	EVENT	

NUMBER	SEQUENCE PROBABILITY	PERCENT CONTRIB	SEQUENCE DESCRIPTION				SEQUENCE IDENTIFIER
			MANUAL ADS ACTUA	CMTS FOR SMALL FAILS FULL RCS FAILS PARTL RCS	DEPRESSUR. SLOCA	W/PRHR, W/O CMT) W/PRHR W/O CMT)	SYS-CM2SL SYS-ADN SYS-ADZ
63	1.85E-10	.05	MANUAL ADS ACTUA	CMTS FOR SMALL	INITIATING EVENT LOCAS DEPRESSUR. SLOCA DEPRESSUR. (S2		SIS-VZCUAI
64	1.76E-10	. 05			INITIATING EVENT	OCCURS W/PRHR, W/O CMT) W/PRHR W/O CMT)	SYS-IECV2
65	1.73E-10	.05	LOSS OF SERVICE SFW AND PRT AUTOMATIC ADS	SYSTEMS FAIL ACT. FAILS FULL	INITIATING EVENT RCS DEPRESSURIZ.	OCCURS (TRANS WITH CMT)	IEV-TSW SYS-XWP SYS-ADA
66	1.70E-10	.05	FUT AND SEUT	AND PRT SYSTEMS	FATI	OCCURS IS AVAILABLE (TRANS W/O CMT) (TRANS W/O CMT)	DEL-CUND
67	1.47E-10	.04	FAULURE OF 2/2	CMTS FOR SMALL FAILS FULL RCS FAILS PARTL RCS	LOCAS DEPRESSUR. SLOCA DEPRESSUR. (S2	W/PRHR, W/O CMT) W/PRHR W/O CMT)	IEV-SL SYS-CM2SL SYS-ADN SYS-ADZ
68	1.34E-10	.04	FAULURE OF 2/2 MANUAL ADS ACTUA RNS FAILS TO	TOP EVENT NODES CMTS FOR SMALL FAILS FULL RCS OPERATE IN	LOCAS DEPRESSUR. SLOCA INJECTION MODE	(LOCA/TRANSIENT)	SYS-CM2SL SYS-ADN SYS-RNR
69	1.28E-10	.04	FRACTION OF LOSS SFWT AND PRT FAILURE OF 2/2 MANUAL ADS	OF FW WITH BOTH SYSTEMS FAIL CMTS FOR ACT. FAILS FULL	FW PUMPS LOST TRANSIENTS RCS DEPRESSURIZ.		IEV-TF SUC-VAL2 DEL-COND OTH-VAL1 SYS-XSP SYS-CM2AB SYS-ADT SYS-AD1

P	70	1.13E-10	.03	VERY SMALL LOCA ADZ EVENT TREE	INITIATING EVENT NGDE IS	OCCURS SUCCESSFUL	(AUTO DEPRESS.)	IEV-SŽS DEL-ADZ
				FAILURE TO MANUAL ADS ACTUA RNS FAILS TO		RCS PUMPS DEPRESSUR. SLOCA INJECTION MODE	W/PRHR, W/O CMT) (LOCA/TRANSIENT)	SYS-RCSL SYS-ADN SYS-RNR
	71	1.07E-10	.03	SMALL LOCA FAILURE TO MANUAL ADS ACTUA RNS FAILS TO	INITIATING EVENT TRIP ALL FOUR FAILS FULL RCS OPERATE IN	RCS PUMPS DEPRESSUR. SLOCA	W/PRBR, W/O CMT) (LOCA/TRANSIENT)	IEV-S2 SYS-RCSL SYS-ADN SYS-RNR
	72	9.86E-11	.03	PASSIVE RHR TUBE CSLOCA AND PRI1 FAILURE TO MANUAL ADS ACTUA RNS FAILS TO	TOP EVENT NODES TRIP ALL FOUR	INITIATING EVENT FAIL RCS PUMPS DEPRESSUR. SLOCA INJECTION MODE		IEV-S2P SYS-XCSLPRII SYS-RCSL SYS-ADN SYS-RNR

TABLE 3 DOMINANT ACCIDENT SEQUENCE CUTSETS - INCLUDES RCS LEAK EVENT

No. of Street, or Stre	NUMBER	SEQUENCE PROBABILITY	CONTRIB	DESCRIPTION				SEQUENCE IDENTIFIER
	73	9.35E-11	.03	TURBINE/REACTOR STEAM DUMP AND FWT AND SFWTD AUTOMATIC ADS AUTOMATIC ADS	TRIP /L.RCS FLOW MAIN CONDENSOR AND PRT SYSTEMS ACT. FAILS FULL ACT. FAILS PART'	INITIATING EVENT AVAILABLE FAIL RCS DEPRESSURIZ. RCS DEPRESSURIZ.	OCCURS (TRANS WITH CMT) (TRANS WITH CMT)	IEV-TT DEL-COND SYS-XFSP SYS-ADA SYS-ADIA
	74	9.31E-11	.03	VERY SMALL LOCA FAULURE OF 2/2 FAILURE OF THE MANUAL ADS ACT. MANUAL ADS	INITIATING EVENT CMTS FOR SMALL PRHR SYSTEM TO FAILS FULL RCS ACT FAILS PARTY	OCCURS LOCAS REMOVE DEC. HEAT DEPRESUR. S2 RCS DEPRESCURT?	FROM RPV (TRAMS) W/O PRHR & CMT)	IEV-S2S SYS-CM2SL SYS-PRT SYS-ADC
	75	9.17E-11	.03	LOSS OF OFFSITE GRID IS NOT GRID IS NOT FAILURE OF THE FAULURE OF 2/2 HANUAL ADS	POWER RECOVERED WITHIN RECOVERED WITHIN PRHR SYSTEM TO CMTS FOR TE	INITIATING EVENT 30 MINUTES 24 HOURS FOLLOW- REMOVE DEC. HEAT	OCCURS FOLLOWING LOSP ING LOSP FROM RFV (S.B)	IEV-TE OTH-R05 OTH-R24 SYS-PRB
	76	8.10E-11	.02	LOSS OF COMPONE. COND AND SFW AUTOMATIC ADS	COOLING SYSTEM	INITIATING EVENT	OCCURS	IEV-TCW
	77	7.53E-11	. 02	LOSS OF FW TO COND AND SFW AUTOMATIC ADS AUTOMATIC ADS	STEAM GENERATOR AND PRT SYSTEMS ACT. FAILS FULL ACT. FAILS PART'	INITIATING EVENT FAIL RCS DEPRESSURIZ. RCS DEPRESSURIZ.	OCCURS (TRANS WITH CMT) (TRANS WITH CMT)	IEV-TF SYS-XCSP SYS-ADA SYS-ADIA
	78			MAIN STEAMLINE	BREAR U/MSIV	INITIATING EVENT	OCCURS FROM RPV (TRANS)	IEV-TSLU
	79	7.11E-11	.02				(LOCA/TRANSIENT)	
-	•	6.77E-11	. 02				OCCURS (S1 W/O CMT)	
	81	6.77E-11	.02	VERY SMALL LOCA RNS FAILS TO	INITIATING EVENT OPERATE IN	OCCURS		IEV-S2S SYS-RNR SYS-RECIRC

7	82	6.75E-11	.02	LOSS OF FW TO FRACTION OF LOSS	STEAM GENERATOR OF FW EVENTS IN	INITIATING EVENT WHICH CONDENSOR	OCCURS IS ALSO LOST	IEV-TF OTH-VAL2
				SFW AND PRT AUTOMATIC ADS AUTOMATIC ADS		RCS DEPRESSURIZ. RCS DEPRESSURIZ.		SYS-XWP SYS-ADA SYS-ADIA
	83	6.40E-11	. 02	SMALL LOCA RNS FAILS TO FAILURE OF	INITIATING EVENT OPERATE IN RECIRCULATION	OCCURS INJECTION MODE	(LOCA/TRANSIENT)	IEV-S2 SYS-RNR SYS-RECIRC
	84	5.88E-11	.02	MAIN STEAMLINE FAILURE OF THE AUTOMATIC ADS AUTOMATIC ADS		INITIATING EVENT REMOVE DEC. HEAT RCS DEPRESSURIZ. RCS DEPRESSURIZ.	FROM RPV (TRANS) (TRANS WITH CMT)	IEV-TSOV SYS-PRT SYS-ADA SYS-ADIA
	85	5.12E-11	.01	SECONDARY TO STEAM DUMP AND	PRIMARY SIDE MAIN CONDENSOR	POWER MISHATCH AVAILABLE	IN. EVENT OCCURS	IEV-TM DEL-COND

			TABLE 3						
DOMINANT	ACCIDENT	SEQUENCE	CUTSETS	-	INCLUDES	RCS	LEAK	EVENT	

NUMBER	SEQUENCE PROBABILITY		SEQUENCE DESCRIPTION				SEQUENCE IDENTIFIER
			FAILURE OF 2/2 MANUAL ADS	CMTS FOR ACT. FAILS FULL	TRANSIENTS RCS DEPRESSURIZ		SYS-XSP SYS-CM2AB SYS-ADT SYS-AD1
86	5.05E-11		CONSEQUENTIAL DELETE NON- AUTOMATIC ADS RNS FAILS TO	SG TUBE RUPTURE SENSICAL CUTSETS ACT. FAILS FULL OPERATE IN	INITIATING EVENT FROM V2 RCS DEPRESSURIZ. INJECTION MODE	SEQUENCES (S2 W/PRHR & CMT	SYS-IECV2 DEL-V2DEL SYS-ADS SYS-RNR
87	4.24E-11	.01	VERY SMALL LOCA FAILURE TO FAILURE OF 2 OF	INITIATING EVENT TRIP ALL FOUR 2 ACCUMULATORS	OCCURS RCS PUMPS	(LOCA/TRANSIENT)	IEV-S2S SYS-RCSL AC2AB-FAILS
88	4.02E-11	.01	SMALL LOCA FAILURE TO FAILURE OF 2 OF	INITIATING EVENT TRIP ALL FOUR 2 ACCUMULATORS	OCCURS RCS PUMPS		IEV-S2 SYS-RCSL AC2AB-FAILS
39	3.82E-11	.01	SYS-XCM OTH-M	WITHOUT SCRAM	INITIATING EVENT	OCCURS	IEV-TFA SYS-XCM OTH-M
90	3.78E-11	. 01					
91	3.47E-11	.01	AUTOMATIC ADS	ACT. FAILS FULL ACT. FAILS PART'	RCS DEPRESSURIZ. RCS DEPRESSURIZ.	FROM RPV (S2) (TRANS WITH CMT) (TRANS WITH CMT)	IEV-S2 SYS-PRL SYS-ADA SYS-ADIA
92	3.38E-11	.01	LOSS OF FW TO FRACTION OF LOSS STEAM DUMP AND FWT AND SFWT AUTOMATIC ADS AUTOMATIC ADS	STEAM GENERATOR OF FW EVENTS IN MAIN CONDENSOR AND PET SYSTEMS	INITIATING EVENT WHICH CONDENSOR AVAILABLE FAIL	OCCURS IS AVAILABLE (TRANS WITH CMT) (TRANS WITH CMT)	IEV-TF SUC-VAL2 DEL-COND
93	3.34E-11	.01	SAFETY INJECTION FAILURE OF 1/1 FAILURE OF 1 OF	CMTS	INITIATING EVENT	OCCURS	IEV-SIS SYS-CMIA ACIA-FAILS
94	3.32E-11	. 01	LOSS OF OFFSITE GRID IS NOT FAILURE OF THE FAILURE OF SFW RNS FAILS TO	RECOVERED WITHIN	BERMOLEE DEC SERIES	OCCURS FOLLOWING LOSP FROM RPV (LOSP) NO DST (TE)	IEV-TE OTH-R05 SYS-PRP SYS-SFWP SYS-RNP

-				FAILURE OF	2 OF 2 GRAVITY	INJECTION LINES		SYS-IW2AB
	95	3.10E-11	.01	IEV-SL				IEV-SL
				OTH-CVCSFIX AUTOMATIC ADS RNS FAILS TO	ACT. FAILS FULL OPERATE IN	RCS DEPRESSURIZ. INJECTION MODE	(S2 W/PRHR & CMT (LOCA/TRANSIENT)	GTH-CVCSFIX SYS-ADS SYS-RNR
	96	3.03E-11	01	CONSEQUENTIAL FAILURE OF THE FAULURE OF 2/2 FAILURE OF 2 OF	MSL SV STUCK PRHR SYSTEM TO CMTS FOR SMALL 2 ACCUMULATORS	OPEN INITIATING REMOVE DEC. HEAT LOCAS	EVENT OCCURS FROM RPV (TRANS)	SYS-IECTSOV SYS-PRT SYS-CM2SL AC2AB-FAILS
	97	2.63E-11	.01	LOSS OF FW TO FRACTION OF LOSS STEAM DUMP AND FRACTION OF LOSS SFWT AND PRT	MAIN CONDENSOR	INITIATING EVENT WHICH CONDENSOR AVAILABLE FW PUMPS LOST	OCCURS IS AVAILABLE	IEV-TF SUC-VAL2 DEL-COND OTH-VAL1 SYS-XSP

			TABLE 3						
DOMINANT	ACCIDENT	SEQUENCE	CUTSETS	-	INCLUDES	RCS	LEAK	EVENT	

NUMBER	SEQUENCE PROBABILITY	PERCENT	SEQUENCE DESCRIPTION				SEQUENCE IDENTIFIER
			AUTOMATIC ADS AUTOMATIC ADS	ACT. FAILS FULL ACT. FAILS PART'		(TRANS WITH CMT) (TRANS WITH CMT)	SYS-ADA SYS-ADIA
98	2.51E-11	.01	LOSS OF OFFSITE GRID IS NOT FAILURE OF THE FAILURE OF SFW FAULURE OF 2/2 MANUAL ADS ACTUA RNS FAILS TO	POWER PECOVERED WITHIN PRHR SYSTEM TO TO SUPPLY SC A CMTS FOR TE FAILS FULL RCSL OPERATE IN	REMOVE DEC. HEAT FROM CST - DEPRESSURIZATION	FOLLOWING LOSP	IEV-TE OTH-ROS SYS-PRP SYS-SFWP SYS-CM2P SYS-ADL SYS-RNP
99	2.408-11	.01	LOSS OF OFFSITE GRID IS NOT CRID IS NOT FAILURE OF THE AUTOMATIC ADS		24 HOURS FOLLOW- REMOVE DEC. HEAT	FOLLOWING LOSP	IEV-TE OTH-R05 OTH-R24 SYS-PRB SYS-ADAB
100	2.36E-11	.01	VERY SMALL LOCA FAILURE OF THE AUTOMATIC ADS AUTOMATIC ADS	INITIATING EVENT PRHR SYSTEM TO ACT. FAILS FULL ACT. FAILS PART'	REMOVE DEC. HEAT RCS DEPRESSURIZ.	FROM RPV (TRANS) (TRANS WITH CMT) (TRANS WITH CMT)	IEV-S25 SYS-PRT SYS-ADA SYS-ADIA

TABLE 4 INITIATING EVENT IMPORTANCES FOR RISK DECREASE MEASURE

SYSTEM UNAVAILABILITY (Q) = 3.358E-07 NUMBER OF BASIC EVENTS = 23 NUMBER OF CUTSETS = 9900

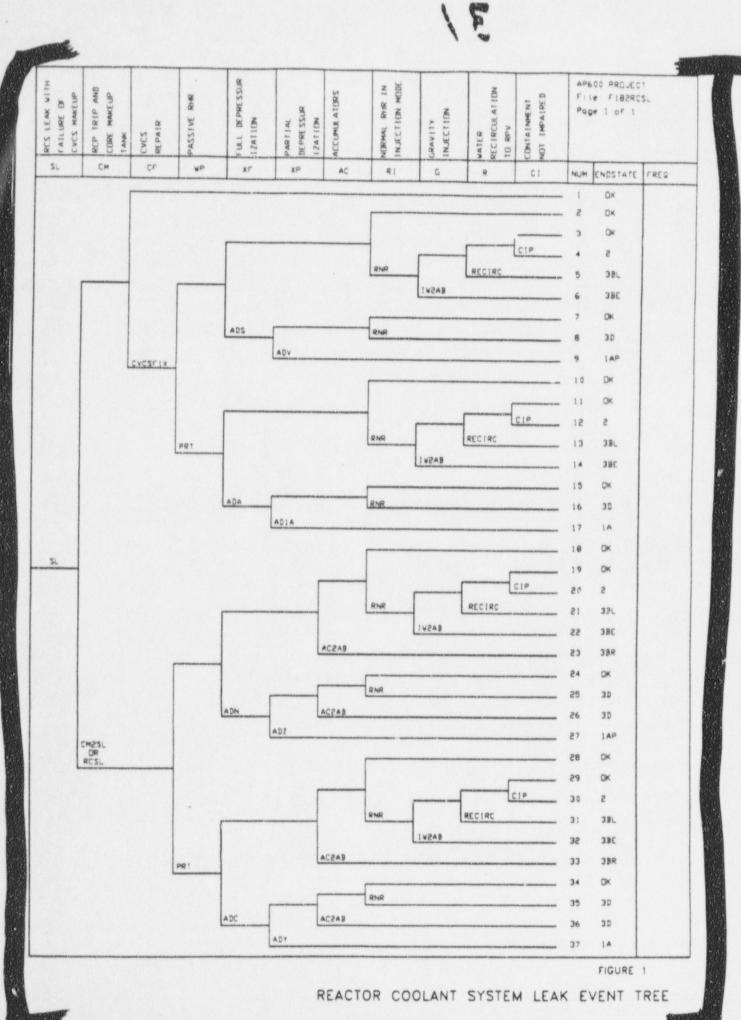
BASIC	EVENT	IMPORTANCE (IDECREASE)	NUMBER OF CUTSETS	DECREASE IN SYSTEM UNAVAILABILITY	BASIC EVENT PROBABILITY
1	IEV-SIS	21.60	284	7.25338-08	1.2000E-04
2	IEV-TFA	13.78	378	4.6266E-08	5.3000E-01
3	IEV-TT	12.75	705	4.2808E-08	1.4000E+00
4	IEV-S2P	12.41	1601	4.1669E-08	5.0000E-03
5	IEV-VR	8.93	1	3.0000E-08	3.0000E-08
6	IEV-S2	6.85	877	2.3013E-08	5.2000E-04
7	IEV-TF	5.42	1225	1.8184E-08	5.0570E-01
8	IEV-A	4.73	114	1.5867E-08	9.7000E-05
9	IEV-S2S	3.54	897	1.1882E-08	5.5000E-04
10	IEV-SI	3.53	960	1.1836E-08	5.6000E-04
11	IEV-TS	1.31	399	4.4149E-09	8.5000E-02
12	IEV-TE	.86 .83	956	2.8769E-09	8.6000E-02
13	IEV-TM	.83	242	2.7726E-09	5.4000E-02
14	IEV-SIC	.81	448	2.7079E-09	1.3000E-04
15	IEV-V2	.78	123	2.6053E-09	5.2000E-01
16	IEV-SL	.47	102	1.5733E-09	6.4800E-05
17	IEV-TCA	.40	145	1.3423E-09	1.4400E-02
18	IEV-TSW	. 35	176	1.1766E-09	2.6200E-02
19	IEV-TSOV	. 26	132	8.6466E-10	1.2000E-03
20	IEV-TP	.21	17	7.0750E-10	4.5000E-03
21	IEV-TCW	.11	4.4	3.7587E-10	1.3500E-02
22	IEV-TSLU	.06	60	2.1309E-10	3.3300E-04
23	IEV-TSLD	. 02	14	8.0270E-11	6.0000E-04

TABLE 4 INITIATING EVENT IMPORTANCES FOR RISK - INCREASE MEASURE

	NUMBER OF B.		3.358E-0 23 9900	'	
BASIC	EVENT	IMPORTANCE (INCREASE)	NUMBER OF CUTSETS	IN SYSTEM	
1	IEV-VR	2.9782442+08		1.0000E+00	3.0000E-08
2	IEV-SIS	179996.	284		1.2000E-04
3	IEV-A	48712.4	114	1.63 6E-04	9.7000E-05
4	1EV-52	13173.7	877	4.4233E-05	5.2000E-04
5 6 7 8	IEV-SL	7230.46	102	2.4278E-05	6.4800E-05
6	IEV-S2S	6430.30	897	2.1591E-05	5.5000E-04
7	IEV-SI	6291.28			5.6000E-04
	IEY-SIC	6202.89	448	2.0827E-05	1.3000E-04
9	IEV-S2P	2469.57	1601		5.0000E-03
10	IEV-TSOV	214.339	132	7.1968E-07	1.2000E-03
11	IEV-TSLU	190.518	60	6.39702-07	3.3300E-04
12	IEV-V2	148.439	123	4.9841E-07	5.2000E-03
13	IEV-TP	46.6139	17	1.5651E-07	4.5000E-03
14	IEV-TSLD	39.8200	14	1.3370E-07	6.0000E-04
15	IEV-TCA	27.3614	145	9.1871E-08	1.4400E-02
16	IEV-TH	14.4661	242	4.8573E-08	5.4000E-02
17	IEV-TS	14.1541	399	4.7525E-08	8.500CE-02
18	IEV-TSW	13.0248	176	4.3733E-08	2.6200E-02
19	IEV-TFA	12.2192	378	4.1028E-08	5.3000E-01
20	IEV-TE	9.10615	956	3.0576E-08	8.6000E-02
21	IEV-TCW	8.18007	44	2.7466E-08	1.3500E-02
22	IEV-TF	5.29349	1225	1.7774E-08	5.0570E-01

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Westinghouse Electric Corporation

Energy Systems

Box 355 Pittsburgh Pennsylvania 15230-0355

> DCP/NRC1413 NSD-NRC-98-5757 Docket No.: 52-003

> > August 14, 1998

Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555

ATTENTION: T. R. Quay

SUBJECT: RESPONSE TO NRC LETTERS CONCERNING REQUEST FOR WITHHOLDING INFORMATION

Reference:

 Letter, Sebrosky to McIntyre, "Request for withholding information from public disclosure for Westinghouse AP600 design letter of October 20, 1993," dated June 18, 1998.

- Letter, Sebrosky to McIntyre, "Request for withholding information from public disclosure for Westinghouse AP600 design letter of January 17, 1994," dated June 18, 1998.
- Letter, Sebrosky to McIntyre, "Request for withholding information from public disclosure for Westinghouse AP600 letters of September 20, 1993, January 21, 1994, and February 3, 1994," dated July 10, 1998.
- 4. Letter, Sebrosky to McIntyre, "Request for withholding proprietary information for Westinghouse letters dated April 18, 1995," dated July 15, 1998.
- Letter, Huilman to McIntyre, "Request for withholding information from public disclosure of Westinghouse report on AP600 function based task analysis," dated July 17, 1998.

Enclosure 2

Dear Mr. Quay:

Reference 1 provided the NRC assessment of the West inghouse claim that proprietary information was provided in a letter dated October 20, 1993, that contained the response to a staff request for additional information regarding the AP600 probabilistic risk assessment. The NRC assessment was that the material was similar to material that exists in the current (1998) nonproprietary version of the AP600 probabilistic risk assessment (PRA) report. In addition, the staff indicated the material was used so the staff in the development of the AP600 draft safety evaluation report and therefore should remain on the docket. At the time this request for additional information response was provided to the

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NRC technical staff, the information was considered to be proprietary by Westinghouse since it contained information that had commercial value to Westinghouse. If this request for additional information response was indeed used by the staff in development of the AP600 draft final safety evaluation report in November 30, 1994, then at this time, almost five years later this information is no longer considered to be proprietary by Westinghouse.

Reference 2 provided the NRC assessment of the Westinghouse claim that proprietary information was provided in a letter dated January 17, 1994, that contained the response to a staff request for additional information regarding the AP600 instrumentation and control system. The NRC assessment was that the material was similar to material that exists in the current (1998) nonproprietary version of the AP600 standard safety analysis report. In addition, the staff indicated the material was used by the staff in the development of the AP600 draft safety evaluation report and therefore should remain on the docket. At the time this request for additional information response was provided to the NRC technical staff, the information was considered to be proprietary by Westinghouse since it contained information that had commercial value to Westinghouse. If this request for additional information report in November 30, 1994, then at this time, over four years later, this information is no longer considered to be proprietary by Westinghouse.

Reference 3 provided the NRC assessment of the Westinghouse claim that proprietary information was provided in a letter dated September 20, 1993, that contained information related to the AP600 PRA and WCAP-13795, which provided the PRA uncertainty analysis. The NRC assessment was that the material was similar to material that exists in the current (1998) nonproprietary version of the AP600 probabilistic risk assessment (PRA) report. In addition, the staff indicated the material was used by the staff in the development of the AP600 draft safety evaluation report and therefore should remain on the docket. At the time this information was provided to the NRC technical staff, it was considered to be proprietary by Westinghouse since it contained information that had commercial value to Westinghouse. If the information transmitted by the Westinghouse September 20, 1993, letter was indeed used by the staff in development of the AP600 draft final safety evaluation report in November 30, 1994, then at this time, almost five years later, this information is no longer considered to be proprietary by Westinghouse.

Reference 3 also provided the NRC assessment of the Westinghouse c'aim that proprietary information was provided in a letter dated January 21, 1994, that contained WCAF-13913, "Framework for AP600 Severe Accident Management Guidance" (SAMG). The NRC assessment was that the material was similar to material that exists in current (1998) nonproprietary AP600 documents (e.g., WCAP-13914, "Framework for AP600 Severe Accident Management Guidance"). In addition, the staff indicated the material was used by the staff in the development of the AP600 draft safety evaluation report and therefore should remain on the docket. At the time this Framework for SAMG was provided to the NRC technical staff, the information was considered to be proprietary by Westinghouse since it contained information that had commercial value to Westinghouse. At this time, over four years later, this information is no longer considered to be proprietary by Westinghouse.

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Reference 3 also provided the NRC assessment of the Westinghouse claim that proprietary information was provided in a letter dated February 3, 1994, that contained additional copies of WCAP-13913, "Framework for AP600 Severe Accident Management Guidance" (SAMG). The NRC assessment was that the material was similar to material that exists in current (1998) none roprictary AP600 documents (e.g., WCAP-13914, "Framework for AP600 Severe Accident Management Guidance"). In addition, the staff indicated the material was used by the staff in the development of the AP600 draft safety evaluation report and therefore should remain on the docket. At the time this Framework for SAMG was provided to the NRC technical staff, the information was considered to be proprietary by Westinghouse since it contained information that had commercial value to Westinghouse. At this time, over four years later, this information is no longer considered to be proprietary by Westinghouse

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Reference 4 provided the NRC assessment of the Westinghouse claim that proprietary information was provided in a letter dated April 18, 1995, that contained information for a MAAP4/RELAP comparison for the AP600 in response to a staff request for additional information. The NRC assessment was that the Westinghouse cover letter indicated that Enclosure 2 is a non-proprietary version of Enclosure 3, however, the staff could not find any portion of the enclosures marked us proprietary. The staff assessment further states the conventional bracketed-superscript notation also appears to be missing. Finally, the NRC assessment states the staff could not determine which part of the material enclosed with the Westinghouse letter was Enclosure 1, 2, or 3. It should be noted that the Westinghouse April 18, 1995, cover letter states "Enclosures 2 (nonproprietary) and 3 (proprietary) provide the requested information." The letter does not indicate that enclosure 2 was a duplicate of enclosure 3 minus the proprietary information. A cover sheet was provided just prior to each of the enclosures to the Westinghouse letter. The enclosures contained the following: Enclosure 1 provided a copy of the NRC's two-page request for information for the MAAP-RELAP comparison. Enclosure 2 provided the requested information, and was titled "Requested Information for AP600 MAAP4/RELAP Comparison." Under section 4, Initial Conditions, of Enclosure 2 it states the initial conditions information (which was proprietary) is provided in Enclosure 3 of the subject Westinghouse letter. Finally, Enclosure 3 contained the list of initial conditions. The information provided in Enclosure 3 was labeled as Westinghouse Proprietary Class 2 at the top of the page, however, the specific proprietary information was not indicated by the bracketed-superscripted notation. In addition to the initial conditions, a mark-up of AP600 PRA Figure K-1 was provided in Enclosure 3. Again, the information was labeled as Westinghouse Proprietary Class 2 at the top of the page, however, the specific proprietary information was not indicated by the bracketed-superscripted notation. At the time the information provided in Enclosure 3 of the subject Westinghouse letter was provided to the NRC technical staff, the information was considered to be proprietary by Westinghouse since it contained information that had commercial value to Westinghouse. At this time, over three years later, this information is no longer considered to be proprietary by Westinghouse.

Reference 5 provided the NRC assessment of the Westinghouse claim that proprietary information was provided in a letter dated February 8, 1994, provided a copy of WCAP-13957, "AP600 Reactor Coolant System Mass Inventory: Function Based Risk Analysis." The NRC assessment was that the material was not "information that the staff customarily accepts as proprietary." In addition, the staff indicated the material was used by the staff in the development of the AP600 final safety evaluation report and therefore should remain on the docket. At the time this report was prepared, the

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DCP/NRC1413 NSD-NRC-98-5757

August 14, 1998

information was considered to be proprietary by Westinghouse since it contained information that had commercial value to Westinghouse and was of the type of information that was customarily held in confidence by Westinghouse. That the material was not information that the staff customarily accepts as proprietary is not relevant to making the proprietary determination. However, in an effort to expedite the issuance of the AP600 Final Safety Evaluation Report and Final Design Approval, Westinghouse agrees to no longer consider this information to be proprietary.

-4-

In a telephone call on July 8, 1998, the staff informed Westinghouse of a concern related to WCAP-13288 and WCAP-13289, which were associated with the AP600 check valve testing specification. The concern was that the proprietary report had no proprietary information identified and the nonproprietary report had been placed in the public document room. Westinghouse has reviewed these reports and, at this time, considers none of the information to be proprietary.

This response addresses the proprietary issues delineated in the references.

Brian A. McIntyre, Manager Advanced Plant Safety and Licensing

jml

cc: J. W. Roe - NRC/NRR/DRPM
J. M. Sebrosky - NRC/NRR/DRPM
W. C. Huffman - NRC/NRR/DRPM
H. A. Sepp - Westinghouse

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