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DMB

February 11, 1986

Mr. James G. Keppler
Regional Administrator
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL. 60137

Subject: Braidwood Station Unit 1
10 CFR50.55(e) Report No. 86-02
Rejectable Indications During Preservice
Inspection on Loop 1 Steam Generator and
the Pressurizer
NRC Docket No. 50-456

Dear Mr. Keppler:

On January 15, 1986, the Commonwealth Edison Company Nuclear Licensing Department notified Mr. W.S. Little of your office of a potential deficiency reportable pursuant to 10 CFR50.55(e) regarding two (2) rejectable weld indications during Unit 1 preservice inspection at Braidwood Station. This letter fulfills the thirty (30) day reporting requirement and is considered to be an interim report. For tracking purposes, this potential deficiency was assigned number 86-02.

Description of Nonconformance

Preservice inspection examinations under ASME Section XI 1977 Edition with addenda through the Summer, 1978 Addenda has resulted in two (2) rejectable ultrasonic indications in weld seams of the Loop 1 steam generator and the pressurizer at Braidwood Unit 1. One indication is in the upper shell to transition cone circumferential weld of the Loop 1 steam generator. The second indication is in the upper middle shell-to-lower middle shell circumferential weld of the pressurizer.

Analysis of Safety Implications

Stresses due to system pressure and repeated heat-ups and cooldowns have the potential for causing Code rejectable weld indications to propagate into through-wall cracks. There is a possibility that such propagation might occur before it could be detected by nondestructive testing during the first inservice inspection interval.

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For the indications in the Pressurizer, a through-wall crack would result in Reactor Coolant System (RCS) leakage. In such an event, the loss of RCS inventory is expected to be small and would be detected by the various available Reactor Coolant Inventory Monitoring Systems.

For the indications in the steam Generators, a through-wall crack would result in a secondary system steam leak. If such a leak occurred, the leak would be expected to be small and it would be detected by a change in the containment environmental parameters as well as a change in the containment sump inventory.

In both cases of weld indication propagation, the resultant leaks are expected to be promptly detected in order to prevent any adverse safety consequences.

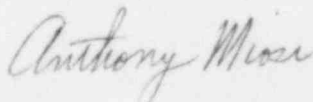
Corrective Action

The two (2) rejectable indications have been characterized as very small innocuous slag inclusions based on the Braidwood preservice examination data and experience with similar indications in the components at Byron Station. Since the small slag inclusions have little or no effect on vessel weld integrity, the present condition of the components will not reduce the level of plant safety. The integrity of the vessels can be best preserved by not removing the indications. Fracture mechanics analysis have been performed which accepts the indications of the vessels for the forty (40) year plant life. This analysis will be submitted to NRR for concurrence.

A Final Report will be sent when resolution of this issue is reached with NRR.

Please address any questions that you or your staff may have concerning this matter to this office.

Very truly yours,



Anthony D. Miosi
Nuclear Licensing Administrator

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cc: NRC Resident Inspector Braidwood

Director of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC. 20555

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