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NRC Farm 386 (9-83)

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Unit Conditions Prior to the Event:

Unit 3 was in cold condition with the reactor mode switch in the 'REFUEL' position.

Reactor Water Cleanup was out-of-service to accommodate modification work.

Description of the Event:

On June 13, 1988 at 1533 hours, the Unit 3 Reactor Water Cleanup (RWCU) outboard isolation valve logic was unexpectedly actuated on a false RWCU high flow signal. The unexpected actuation of an engineered safety feature is reportable to the NRC under 10 CFR 50.73(a)(2)(iv).

Prior to the event, electricians were inspecting cable splices in the area of the RWCU outboard logic differential pressure switch (DPIS-3-12-124B) as part of modification work which required the inspection and resplicing of Class 1E (environmentally qualified) cable. During the inspection, relay 16A-K63 was momentarily deenergized, generating an outboard Group II/III isolation signal.

Consequences of the Event:

There were no adverse consequences of this event. The RWCU isolation logic actuated as designed; however, no valve motion occurred because RWCU was out-of-service, with its associated isolation valves blocked closed. The purpose of the differential pressure switch is to sense high flow in the event of a pipe rupture downstream of the isolation valves. Actuation of the isolation feature with the reactor at power and the RWCU in service would have isolated the RWCU from the primary coolant system. Closure of the isolation valves prevents any potential release of radioactive effluents from the primary coolant system to the RWCU system. Short term isolation of the RWCU system during shutdown or at power has a negligible affect on primary coolant chemistry.

| NAC Form 304A - (9-83) | LICENSEE EVENT REPO | ORT | LE | R) T | EX | TO | SOM. | TIN | ATI | ON | Cove . | er v | 8 NU 40 80 | CLEAR REG | NA NO 21 | 50-010 | 115610N |
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Cause of the Event:

The cause of the event was equipment failure. An investigation revealed that a small piece of the insulation on the lead wire from a differential pressure switch DPIS-3-12-124B was missing. The switch and wire are manufactured by ITT Barton, serial number 289A-3920. When the lead was moved during an approved procedural inspection, the exposed area contacted the metal conduit causing a ground which resulted in the PCIS logic actuation. The reason for the missing insulation could not be determined.

Corrective Actions:

The isolation was reset at 1700 hours. The isolation signal existed for 87 minutes.

Actions Taken to Prevent Recurrence:

Modification No. 2355 has been initiated to inspect and rework Unit 2 cable splices, and rework Unit 3 cable splices for safetyrelated instrumentatio and 480V motors (except for motors for motor operated valves which will be examined under modification nos. 1915, 2231, and 2533 prior to restart). The repair of the insulation will be incorporated into modification number 2355. This modification is scheduled to be completed by October 1, 1988 for Unit 2 and February 15, 1989 for Unit 3.

EIIS Codes for Systems and Components:

The EIIS codes for the systems and components discussed in this LER are as follows: Containment Isolation Control System (PCIS) -JM; Reactor Water Cleanup System (RWCU) - CE; isolation valves -ISV; cable (wire) - CEL; differential pressure indicating switch - PDIS.

| LIGE SEE EVENT REPO | RT (LER) TEXT CONTINU | UATION | 6.1 NOC | ROVED O | MB NO. 31 | 80-0 | 104 |
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Previous Similar Occurrences:

LERs 2-87-13 and 2-87-27 addressed PCIS actuations as a result grounds caused by moving cables. However, neither of these events involved missing cable insulation.

10 CFR 50.73(a)(2)(iv)

PHILADELPHIA ELECTRIC COMPANY

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(215) 841-4000

August 30, 1988

Docket No. 50-278

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555

1.1.1.1

SUBJECT: Licensee Event Report Peach Bottom Atomic Power Station - Unit 3

This report addresses a Reactor Water Cleanup System isolation caused by grounding a cable.

| Reference: | Docket No. 50-278 |
|------------------|-----------------------------------|
| Report Number: | 3-88-05 |
| Revision Number: | 01 |
| Event Date: | June 13, 1988 |
| Report Date: | August 30, 1988 |
| Facility: | Peach Bottom Atomic Power Station |
| | RD 1, Box 208, Delta, PA 17314 |

This revised LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv). The original report stated in "Actions Taken to Prevent Recurrence" that all Unit 2 and Unit 3 Class 1E in-line splices will be inspected and reworked under Modification No. 2355. This revised report is being submitted to more accurately identify the scope of the work to be performed and to state that the insulation will be replaced as part of the modification. The changes have been identified by a bar in the margin adjacent to the changes.

Very truly yours, olai

E. P. Fogarty Manager Nuclear Support Division

CC: W. T. Russell, Administrator, Region I, USNRC T. P. Johnson, NRC Senior Resident Inspector T. E. Magette, State of Maryland INPO Records Center