

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) LaSalle County Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 7 3	PAGE (3) 1 OF 0 3
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TITLE (4) Main Steam Safety Relief Valves E, H, U Fail to Meet Setpoint Tolerance Specified by Technical Specifications

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																					
0 1	2 0	8 6	8 6	0 2	0 0	0 2	1 8	8 6			0 5 0 0 0																																					
<table border="1" style="width:100%"> <tr> <td style="width:15%">OPERATING MODE (9) 5</td> <td colspan="10">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 0 0 0</td> <td>20.402(b)</td> <td>20.408(e)</td> <td>80.73(a)(2)(iv)</td> <td>73.71(b)</td> </tr> <tr> <td>20.408(a)(1)(i)</td> <td>80.38(a)(1)</td> <td>80.73(a)(2)(v)</td> <td>73.71(a)</td> </tr> <tr> <td>20.408(a)(1)(ii)</td> <td>X 80.38(a)(2)</td> <td>80.73(a)(2)(vi)</td> <td>OTHER (Specify in Abstract below and in Text, NRC Form 308A)</td> </tr> <tr> <td>20.408(a)(1)(iii)</td> <td>80.73(a)(2)(ii)</td> <td>80.73(a)(2)(viii)(A)</td> <td></td> </tr> <tr> <td>20.408(a)(1)(iv)</td> <td>80.73(a)(2)(iii)</td> <td>80.73(a)(2)(viii)(B)</td> <td></td> </tr> <tr> <td></td> <td>20.408(a)(1)(v)</td> <td>80.73(a)(2)(iv)</td> <td>80.73(a)(2)(ix)</td> <td></td> </tr> </table>												OPERATING MODE (9) 5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)										POWER LEVEL (10) 0 0 0	20.402(b)	20.408(e)	80.73(a)(2)(iv)	73.71(b)	20.408(a)(1)(i)	80.38(a)(1)	80.73(a)(2)(v)	73.71(a)	20.408(a)(1)(ii)	X 80.38(a)(2)	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)	20.408(a)(1)(iii)	80.73(a)(2)(ii)	80.73(a)(2)(viii)(A)		20.408(a)(1)(iv)	80.73(a)(2)(iii)	80.73(a)(2)(viii)(B)			20.408(a)(1)(v)	80.73(a)(2)(iv)	80.73(a)(2)(ix)	
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LICENSEE CONTACT FOR THIS LER (12)

NAME John Ullrich, Technical Staff Engineer, extension 463	TELEPHONE NUMBER AREA CODE: 8 1 5 3 5 7 - 6 7 6 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	S B	R V	C 7 1 0	Y	X	S B	R V	C 7 1 0	Y
X	S B	R V	C 7 1 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

When the as received set pressure test results, from Wyle Laboratories, were reviewed for the 18 Safety Relief Valves (SRV's) sent for testing, SRV's 1B21-F013E, H and U were determined to exceed their set pressure tolerance set forth in Technical Specification 3/4.4.2.

The cause for SRV's 1B21-F013E and U exceeding their set pressure tolerance was determined to be due to seat leakage which results in a decrease in set pressure. No cause could be determined for SRV 1B21-F013H set point drift.

This event was of minimal significance because the Reactor Vessel had adequate over-pressure protection for all expected transients.

The setpoints for valves 1B21-F013E and H were adjusted to nameplate $\pm 1\%$ and returned to LaSalle for installation. SRV 1B21-F013U will be adjusted to nameplate $\pm 1\%$ and returned to LaSalle as a spare SRV.

No additional corrective actions are anticipated.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT: If more space is required, use additional NRC Form 366A (1) (17)

I. EVENT DESCRIPTION

In accordance with the In-Service Inspection Program, and because of suspected seat leakage, all 18 of the Unit 1 Main Steam (SB) Safety/Relief Valves (SRV) were sent to Wyle Laboratories for set pressure testing. On January 20, 1986, the results of these tests were reviewed and it was determined that valves 1B21-F013E (S/N N63790-00-0074), 1B21-F013H (S/N N63790-00-0069) and 1B21-F013U (S/N N63790-00-0064) did not meet the nameplate set pressure +1%, -3% tolerance as specified in Technical Specification 3/4.4.2. Listed below are the valves' nameplate set pressure, tolerance, and as received test results.

VALVE	NAMEPLATE PRESSURE	NAMEPLATE +1%	NAMEPLATE -3%	AS RECEIVED TEST RESULTS
E	1175	1186.75	1139.75	1137.5
H	1195	1206.95	1159.15	1218.5
U	1150	1161.5	1115.5	1114.8

All pressures are in psig.

II. CAUSE

At the time of this event, Unit 1 was defueled during the first refueling outage. These and other SRV's have seat leakage, due at least in part, to foreign material, dirt and rust, in the main steam lines when the valves were first cycled. Another cause of leakage was due to cycling the valves at low, 150 psig, steam pressure for surveillances. This allows the disk to strike the seat, during closing, extremely hard due to the low pressure on the under side of the disk. Per conversation with the SRV manufacturer, when the SRV's begin to experience seat leakage, this leakage tends to cause the relief set pressure to decrease. This is the reason that SRV's S/N N63790-00-0074 and S/N N63790-00-0064 were low out of tolerance. The valve manufacturer could give no cause for the set pressure of valve S/N N63790-00-0069 increasing to greater than the nameplate set pressure +1%. These SRV's are style 6XRX10 HB-65-BP Safety/Relief Valves manufactured by Crosby Valve and Gage Company.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

This event was of minimal significance because the relief setting, the pressure at which the SRV will open due to a pressure switch, for 14 of the 18 SRV's is lower than safety setting of SRV 1B21-F013U, which had the lowest safety setting. That is, if a pressure transient occurred which required opening numerous SRV's, 14 of the 18 SRV's would be open prior to reaching the safety setting of SRV 1B21-F013U.

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TEXT (If more space is required, use additional NRC Form 388A's) (17)

IV. CORRECTIVE ACTION

The safety setpoints of valves S/N N63790-00-0069 and S/N N63790-00-0074 were reset to the nameplate set pressure + 1%, and reinstalled in Unit 1 as SRV's 1B21-F013M and 1B21-F013K, respectively. Valve N63790-00-0064 is presently at Wyle Laboratories where it will be reset to the nameplate set pressure + 1%. When returned to LaSalle, it will be a spare SRV. No additional corrective actions are anticipated.

V. PREVIOUS OCCURRENCES

No previous occurrences of this type have occurred at LaSalle.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

John Ullrich, Technical Staff Engineer, 815/357-6761, extension 463.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

February 18, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #86-002-00, Docket #050-373 is being submitted to your office in accordance with 10CFR 50.73.

R. J. Diederich
for G. J. Diederich
Station Manager
LaSalle County Station

GJD/DRR/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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