



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 18
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated September 2, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 18, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 29, 1988

RDI-2/LA
NOL/Braden
8/29/88

GR
PDI-2/PM
GRivenbark:tr
8/1/88

WB
PDI-2/D
WButler
8/29/88

to
BC:RSB
MHodges
8/12/88

EF
BC:RSB
EButcher
8/22/88

OGC
OGC
K. Beckmann
8/24/88

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler

Walter R. Butler, Director
Project Directorate 1-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

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ATTACHMENT TO LICENSE AMENDMENT NO. 18

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages are provided to maintain document completeness.*

Remove

1-9*
1-10

3/4 3-59*
3/4 3-60

3/4 3-61
3/4 3-62*

Insert

1-9*
1-10

3/4 3-59*
3/4 3-60

3/4 3-61
3/4 3-62*

DEFINITIONS

TURBINE BYPASS SYSTEM RESPONSE TIME

- 1.47 The TURBINE BYPASS SYSTEM RESPONSE TIME consists of two separate time intervals: a) time from initial movement of the main turbine stop valve or control valve until 80% of the turbine bypass capacity is established, and b) the time from initial movement of the main turbine stop valve or control valve until initial movement of the turbine bypass valve. Either response time may be measured by any series of sequential, overlapping, or total steps such that the entire response time is measured.

UNIDENTIFIED LEAKAGE

- 1.48 UNIDENTIFIED LEAKAGE shall be all leakage which is not IDENTIFIED LEAKAGE.

UNRESTRICTED AREA

- 1.49 An UNRESTRICTED AREA shall be any area at or beyond the SITE BOUNDARY access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials, or any area within the SITE BOUNDARY used for residential quarters or for industrial, commercial, institutional, and/or recreational purposes.

VENTILATION EXHAUST TREATMENT SYSTEM

- 1.50 A VENTILATION EXHAUST TREATMENT SYSTEM shall be any system designed and installed to reduce gaseous radiiodine or radioactive material in particulate form in effluents by passing ventilation or vent exhaust gases through charcoal adsorbers and/or HEPA filters for the purpose of removing iodines or particulates from the gaseous exhaust stream prior to the release to the environment. Such a system is not considered to have any effect on noble gas effluents. Engineered Safety Feature (ESF) atmospheric cleanup systems are not considered to be VENTILATION EXHAUST TREATMENT SYSTEM components.

VENTING

- 1.51 VENTING shall be the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is not provided or required during VENTING. Vent, used in system names, does not imply a VENTING process.

TABLE 1.1
SURVEILLANCE FREQUENCY NOTATION

<u>NOTATION</u>	<u>FREQUENCY</u>
S	At least once per 12 hours.
D	At least once per 24 hours.
W	At least once per 7 days.
M	At least once per 31 days.
Q	At least once per 92 days.
SA	At least once per 184 days.
A	At least once per 366 days.
R	At least once per 18 months (550 days).
S/U	Prior to each reactor startup.
P	Prior to each radioactive release.
Z	During startup, prior to exceeding 30% of RATED THERMAL POWER, if not performed within the previous 7 days
N.A.	Not applicable.

TABLE 3.3.6-2
CONTROL ROD BLOCK INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. <u>ROD BLOCK MONITOR</u>		
a. Upscale		
i. Flow Biased	$< 0.66(w-\Delta w) + 4.0\%$ ^a	$< 0.66(w-\Delta w) + 4.0\%$ ^a
ii. High Flow Clamped	$< 106\%$	$< 109\%$
b. Inoperative	NA	NA
c. Downscale	$> 5\%$ of RATED THERMAL POWER	$> 3\%$ of RATED THERMAL POWER
2. <u>APRM</u>		
a. Flow Biased Neutron Flux - Upscale	$< 0.66(w-\Delta w) + 4.2\%$ ^a	$< 0.66(w-\Delta w) + 4.5\%$ ^a
b. Inoperative	NA	NA
c. Downscale	$> 4\%$ of RATED THERMAL POWER	$> 3\%$ of RATED THERMAL POWER
d. Neutron Flux - Upscale, Startup	$< 1.2\%$ of RATED THERMAL POWER	$< 1.4\%$ of RATED THERMAL POWER
3. <u>SOURCE RANGE MONITORS</u>		
a. Detector not full in	NA	NA
b. Upscale	$< 1.0 \times 10^5$ cps	$< 1.6 \times 10^5$ cps
c. Inoperative	NA	NA
d. Downscale	> 3 cps	> 1.8 cps
4. <u>INTERMEDIATE RANGE MONITORS</u>		
a. Detector not full in	NA	NA
b. Upscale	$< 100/125$ divisions of Full scale	$< 110/125$ divisions of Full scale
c. Inoperative	NA	NA
d. Downscale	$> 5/125$ divisions of Full scale	$> 3/125$ divisions of Full scale
5. <u>SCRAM DISCHARGE VOLUME</u>		
a. Water Level-High (Float Switch)	109'1" (North Volume) 108'11.5" (South Volume)	109'3" (North Volume) 109'1.5" (South Volume)
6. <u>REACTOR COOLANT SYSTEM RECIRCULATION FLOW</u>		
a. Upscale	$< 100\%$ of rated flow	$< 111\%$ of rated flow
b. Inoperative	NA	NA
c. Comparator	$< 10\%$ flow deviation	$< 11\%$ flow deviation
7. <u>REACTOR MODE SWITCH SHUTDOWN POSITION</u>	NA	NA

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^aThe rod block function is varied as a function of recirculation loop flow (w) and Δw which is defined as the difference in indicated drive flow (in percent of drive flow which produces rated core flow) between two loop and single loop operation at the same core flow. The trip setting of the Average Power Range Monitor Rod Block function must be maintained in accordance with Specification 3.7.2.

TABLE 4.3.6-1

CONTROL ROD BLOCK INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION ^(a)	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
1. <u>ROD BLOCK MONITOR</u>				
a. Upscale	NA	Z ^{(c)(d)} , M ^(c)	SA	1*
b. Inoperative	NA	Z ^{(c)(d)} , M ^(c)	NA	1*
c. Downscale	NA	Z ^{(c)(d)} , M ^(c)	SA	1*
2. <u>APRM</u>				
a. Flow Biased Neutron Flux - Upscale	NA	S/U ^(b) , M	SA	1
b. Inoperative	NA	S/U ^(b) , M	NA	1, 2, 5
c. Downscale	NA	S/U ^(b) , M	SA	1
d. Neutron Flux - Upscale, Startup	NA	S/U ^(b) , M	SA	2, 5
3. <u>SOURCE RANGE MONITORS</u>				
a. Detector not full in	NA	S/U ^(b) , W	NA	2, 5
b. Upscale	NA	S/U ^(b) , W	SA	2, 5
c. Inoperative	NA	S/U ^(b) , W	NA	2, 5
d. Downscale	NA	S/U ^(b) , W	SA	2, 5
4. <u>INTERMEDIATE RANGE MONITORS</u>				
a. Detector not full in	NA	S/U ^(b) , W	NA	2, 5
b. Upscale	NA	S/U ^(b) , W	SA	2, 5
c. Inoperative	NA	S/U ^(b) , W	NA	2, 5
d. Downscale	NA	S/U ^(b) , W	SA	2, 5
5. <u>SCRAM DISCHARGE VOLUME</u>				
a. Water Level-High (Float Switch)	NA	M	R	1, 2, 5**
6. <u>REACTOR COOLANT SYSTEM RECIRCULATION FLOW</u>				
a. Upscale	NA	S/U ^(b) , M	SA	1
b. Inoperative	NA	S/U ^(b) , M	NA	1
c. Comparator	NA	S/U ^(b) , M	SA	1
7. <u>REACTOR MODE SWITCH SHUTDOWN POSITION</u>	NA	R	NA	3, 4

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TABLE 4.3.6-1 (Continued)

CONTROL ROD BLOCK INSTRUMENTATION SURVEILLANCE REQUIREMENTS

NOTES:

- a. Neutron detectors may be excluded from CHANNEL CALIBRATION.
- b. Within 24 hours prior to startup, if not performed within the previous 7 days.
- c. Includes reactor manual control multiplexing system input.
- d. Within 24 hours prior to exceeding 30% of RATED THERMAL POWER, if not performed within the previous 7 days.
- * With THERMAL POWER \geq 30% of RATED THERMAL POWER.
- ** With more than one control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

INSTRUMENTATION

3/4.3.7 MONITORING INSTRUMENTATION

RADIATION MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.7.1 The radiation monitoring instrumentation channels shown in Table 3.3.7.1-1 shall be OPERABLE with their alarm/trip setpoints within the specified limits.

APPLICABILITY: As shown in Table 3.3.7.1-1.

ACTION:

- a. With a radiation monitoring instrumentation channel alarm/trip setpoint exceeding the value shown in Table 3.3.7.1-1, adjust the setpoint to within the limit within 4 hours or declare the channel inoperable.
- b. With one or more radiation monitoring channels inoperable, take the ACTION required by Table 3.3.7.1-1.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.7.1 Each of the above required radiation monitoring instrumentation channels shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations for the conditions and at the frequencies shown in Table 4.3.7.1-1.