

OPERATIONS SUMMARY
JANUARY, 1986

The unit entered January operating at 87.5% reactor power (power was limited by steam generator level), with generator output at 750 MWe. On January 3, 1986 the turbine was manually tripped per the final evolution of the Power Escalation Test Program resulting in automatic reactor trip. The reactor coolant pumps were secured and emergency feedwater automatically actuated. Subsequent to acceptable post trip review, the reactor was taken critical. At 22% power during the escalation sequence a turbine trip/reactor trip occurred as a result of high moisture separator level. A failed heater drains valve controller was replaced and later that day the reactor was taken critical and achieved the power level cutoff (92% reactor power) on January 4, 1986 with steam generator levels at 68%.

On January 6, 1986 at 14:05 the unit one reactor achieved 100% power. On January 24, 1986 a partial loss of ICS auto power was experienced as a result of returning power (following maintenance) to the "B" side main feedwater flow transmitter. The plant was stabilized at approximately 97% during this event. Restoration of the ICS auto power was performed following controlled and deliberate trouble shooting by the Lead I&C Engineer and Maintenance Manager. The Unit returned to 100% steady state power.

The reactor was shutdown on January 27, 1986 and a cooldown was complete the following day to repair 8th stage extraction lines internal to the condenser. The unit remained in cold shutdown through the end of January to effect repairs.

MAJOR SAFETY RELATED MAINTENANCE

During the month of January, TMI Unit 1 performed no major maintenance on safety related equipment.

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OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE 1-31-86
 COMPLETED BY C.W. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

1. UNIT NAME: THREE MILE ISLAND UNIT 1
2. REPORTING PERIOD: JANUARY ,1986.
3. LICENSED THERMAL POWER (MWT): 2535.
4. NAMEPLATE RATING (GROSS MWE): 871.
5. DESIGN ELECTRICAL RATING (NET MWE): 819.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 824.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 776.

NOTES

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE) _____
 10. REASONS FOR RESTRICTIONS, IF ANY: _____

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	744.	744.	100081.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	604.3	604.3	34420.9
13. REACTOR RESERVE SHUTDOWN HOURS	105.3	105.3	988.5
14. HOURS GENERATOR ON-LINE	595.9	595.9	33629.8
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	1473545.	1473545.	80823941.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	485365.	485365.	26877518.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	453770.	453770.	25105482.
19. UNIT SERVICE FACTOR	80.1	80.1	33.6
20. UNIT AVAILABILITY FACTOR	80.1	80.1	33.6
21. UNIT CAPACITY FACTOR (USING MDC NET)	78.6	78.6	32.1
22. UNIT CAPACITY FACTOR (USING DER NET)	74.5	74.5	30.6
23. UNIT FORCED OUTAGE RATE	15.6	15.6	63.7

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH
 Eddy Current Outage (Steam Generators); March 22, 1986; 35 days

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE 1-31-86
 COMPLETED BY C.W. Smyth
 TELEPHONE (717) 948-8551

MONTH: JANUARY

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	701.	17	777.
2	334.	18	776.
3	-39.	19	772.
4	200.	20	776.
5	726.	21	777.
6	769.	22	779.
7	806.	23	781.
8	804.	24	780.
9	792.	25	783.
10	724.	26	781.
11	782.	27	682.
12	778.	28	-21.
13	781.	29	-10.
14	779.	30	-7.
15	777.	31	-7.
16	773.		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289
UNIT NAME TMI-I
DATE 1/31/86
COMPLETED BY C.W. Smyth
TELEPHONE (717) 948-8551

REPORT MONTH JANUARY

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴ & 6	Component Code ⁵ & 6	Cause & Corrective Action to Prevent Recurrence
1	1/2/86	S	34.5	B	3	N/A	RC	Instru	Trip from 88% for ICS testing (final test in Power Escalation Program).
2	2/4/86	F	10.5	A	3	86-02	HH	Valvex	Anticipatory Reactor Trip on Turbine Trip caused by high moisture separator drain tank level. Repaired heater drain level controller and associated high level dump valve.
3	2/27/86	S	100	A	1	N/A	HJ	HT Exch	Shutdown to repair bellows on extraction steam line.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

⁶
 Actually used Exhibits F & H NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: Three Mile Island Nuclear Station, Unit 1
2. Scheduled date for next refueling shutdown: December 6, 1986
3. Scheduled date for restart following refueling: May 4, 1987
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

If answer is yes, in general, what will these be?

Conversion to 18 month fuel cycle with associated changes to Power Imbalance, Quadrant Tilt and Rod Insertion Limits.

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? No

If no such review has taken place, when is it scheduled?
To be determined.

5. Scheduled date(s) for submitting proposed licensing action and supporting information: November, 1986
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: None
7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 208
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1991 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).



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February 13, 1986
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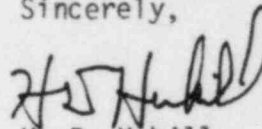
Office of Management Information and Program Control
Attn: W. C. McDonald
c/o Distribution Services Branch DPC, ADM
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. McDonald:

Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report
January, 1986

Enclosed please find two (2) copies of the January, 1986 Monthly Operating Report for Three Mile Island Nuclear Station Unit-1.

Sincerely,



H. D. Hukill
Director, TMI-1

HDH:DVH:spb

Attachments

cc: V. Stello
Dr. T. E. Murley

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