

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102-2247
402/536-4000

August 31, 1988
LIC-88-610

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

- References:
1. Docket No. 50-285
 2. Generic Letter 88-05, "Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWR Plants," dated March 17, 1988
 3. Letter from OPPD (R. L. Andrews) to NRC (Document Control Desk), dated June 6, 1988 (LIC-88-464)

Gentlemen:

SUBJECT: Additional Information for Response to NRC Generic Letter 88-05

Omaha Public Power District (OPPD) submitted Fort Calhoun Station's current program for detection of boric acid corrosion in the Reactor Coolant System in Reference 3. This submittal transmits additional information and a schedule for implementation of improvements to the program.

Special Procedure SP-CSF-1 is conducted on a refueling outage basis to examine carbon steel in safety related systems for evidence of boric acid corrosion. SP-CSF-1 will be reviewed to ensure that all principal mechanical joints in the RCS are included for inspection. Any changes to SP-CSF-1 as a result of this review will be made prior to its use during the 1988 refueling outage scheduled to start September 23, 1988 (hereafter referred to as "the outage"). Steps will also be added to SP-CSF-1 before its use to ensure proper inspection and engineering evaluation of any corroded areas that may be found. A detailed walkdown of the RCS will be conducted as SP-CSF-1 is performed during the outage to provide assurance that the procedure is complete and adequate. Special emphasis will be placed on identifying potential paths for leakage. Inspection of such paths will be incorporated into SP-CSF-1 along with any other changes resulting from the walkdown within 60 days of the end of the outage.

The RCS leak rate calculation (Procedure ST-RLT-3) has been transferred to the new plant computer and is scheduled to be used following the outage. In order to ensure that results are accurate, the maximum error of the leak rate calculation will be determined prior to its use. This calculation will continue to be done on a daily basis. ST-RLT-3 will be revised prior to the end of the outage

8809070010 880831
PDR ADOCK 05000285
P PDC

Acc 1/10

Document Control Desk
LIC-88-610
Page Two

to require review of leakage values by Station Engineering. This will provide assurance that any increase in leakage rates is adequately addressed. Training for engineers on this procedure change will be provided prior to the end of the outage and will stress the importance of locating small primary system leaks.

OPPD believes that our improved program including engineering review of RCS leakage values and detailed inspections of the RCS on a refueling outage basis will provide adequate assurance that boric acid corrosion is promptly detected and corrected.

Sincerely,



K. J. Morris
Division Manager
Nuclear Operations

KJM/rh

Attachment

c: LeBoeuf, Lamb, Leiby & MacRae
1333 New Hampshire Avenue, N.W.
Washington, DC 20036

R. D. Martin, NRC Regional Administrator, Region IV
P. D. Milano, NRC Project Manager
P. H. Harrell, NRC Senior Resident Inspector

