

U.S. NUCLEAR REGULATORY COMMISSION
REGION I

Report No. 50-354/86-04

Docket No. 50-354

License No. CPPR-120

Licensee: Public Service Electric & Gas Company

Post Office Box 236

Hancocks Bridge, New Jersey 08038

Facility Name: Hope Creek Generating Station

Inspection At: Hancocks Bridge, and Salem, New Jersey

Inspection Conducted: January 7-10 and 13-15, 1986

Inspector: *G. Napuda*

G. Napuda, Lead Reactor Engineer

2/13/86
date

Approved by: *P. K. Eapen*

Dr. P. K. Eapen, Chief, Quality Assurance
Section, OB, DRS

2/13/86
date

Inspection Summary: Inspection on January 7-10 and 13-15, 1986
(Report No. 50-354/86-04).

Areas Inspected: Routine announced inspection by a region based inspector to assess the readiness for implementation of the QA Program for Operations and the TS in the areas of design changes/modifications; tests and experiments; onsite operations review committee; onsite independent safety review group; and, offsite safety review group. Also, previously identified items were followed up. The inspection involved 74 inspection hours.

Results: No violations were identified nor were there any concerns that would impact on issuance of an OL.

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DETAILS

1. Persons Contacted

- *E. Butler, Principal Training Supervisor
- R. Burricelli, General Manager Engineering and Plant Betterment
- *A. Giardino, Manager Station QA
- *D. Cooley, Onsite Safety Review Engineer
- *R. Donges, Lead QA Engineer
- *R. Edmonds, Assistant Manager of Nuclear Training
- J. Ellis, System Engineer Training Supervisor
- *R. Griffith, Principal QA Engineer
- D. Hanson, Manager of Nuclear Training
- *E. Liden, Manager Offsite Safety Review
- J. Nichols, Chairman Station Operations Review Committee
- *R. Salvesen, General Manager Operations
- *W. Schultz, Manager QA Programs and Audits
- T. Taylor, Manager Plant Engineering and Betterment Controls

NRC

- *J. Lyash, Resident Inspector
- R. Borchardt, Senior Resident Inspector

Other employees contacted included administrative, engineering, operations, QA/QC and technical personnel.

*attended the January 15, 1986 exit interview.

2. Previously Identified Items

(Closed) Unresolved Item (85-33-01): Verify that the Station Operations Review Committee (SORC) is functioning in accordance with the TS, and administrative procedures describe the manner in which the committee executes its responsibilities. It was verified that the following issued procedures do describe the improved safety and technical review activities in sufficient detail.

- SA-AP.ZZ-004(Q), Station Operations Review Committee, Rev. 4
- SA-AP.ZZ-032(Q), Review and Approval of Station Procedures and Procedure Revisions, Rev. 3

It was also verified that all department managers were involved in a special SORC meeting where the new procedures were discussed and that other staff training was scheduled for January 21, 1986. The General Manager stated that full implementation of the new review methodology is scheduled for February 7, 1986 or in any event prior to core load. This area was determined to be ready to support plant operations. This item is closed.

(Closed) Unresolved Item (85-33-02): Verify that the Offsite Safety Review Group has been staffed and is functioning in accordance with established procedures. The group has been staffed, has begun to function and procedure M40-POP-01, Offsite Review Group Organization and Responsibility, Revision 0 has been issued. Current members exceed position requirements and the one open position is to be occupied by a qualified contracted individual on an interim basis until a permanent employee can be appointed. This area was determined to be ready to support plant operations.

This item is closed.

(Closed) Unresolved Item (85-33-03): Verify that the Onsite Safety Review Group has been staffed and is functioning in accordance with established procedures. The group has been established, is functioning and the following issued procedures describe their activities in sufficient detail.

- M20-POP-01, Safety Review Group, Revision 0
- M20-MSP-01, Onsite Safety Review Group Manual Preparation and Control, Revision 0
- M20-AP-01, Safety Review Group Recommendations, Revision 0
- M20-AP-02, Safety Review Group Independent Review of Reactor Scram/ECCS Actuation Events, Revision 0

Current members education and experience were reviewed to verify they met position qualifications. Two contracted individuals were being considered to occupy the one open position on an interim basis until a permanent employee could be assigned. Both individuals were determined to meet position requirements. This area was determined to be ready to support plant operations.

This item is closed.

3. Design Changes/Modifications

3.1 References/Requirements

1. FSAR Section 17.2, Quality Assurance During the Operations Phase
2. ANSI N18.7-1976, Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants
3. ANSI N45.2.11-1974, Quality Assurance Requirement for the Design of Nuclear Power Plants
4. GM8-EMP-009, Operational Design Change Control, Revision 0
5. GM 8-1, Engineering and Plant Betterment Department Manual

3.2 Program Review

The intent of this review was to ascertain the readiness of the licensee's Design, Design Change and Modification, and Tests and Experiments Programs for the operational phase of the station. Procedures were reviewed to verify that they provided sufficient detail and were consistent with the licensee's commitments. Employees were interviewed to determine that they were aware of their authorities and responsibilities, and were knowledgeable in applicable procedures. Training and personnel records of selected employees were also reviewed to verify that incumbents had adequate education and experience, or proper supplemental training for their positions. Records of completed activities were reviewed to determine the effectiveness of the established program. When possible, ongoing activities were observed to assure they were accomplished in accordance with established procedures.

The licensee has recently completed a restructuring of those engineering organizations that support plant operations. Department procedures and instructions have been completely rewritten and issued. GM8-EM-009 and the Engineering Department Manual were reviewed and it was determined that the following were established:

- A formal method for initiating design changes
- Measures to control modifications, design reviews and safety evaluations
- A method to assure that a change does not constitute an unreviewed safety question as defined in 10 CFR 50.59
- Measures to control changes to previously approved documents, recall obsolete documents, and release and distribute approved documents
- Measures to include those temporary changes, tests and experiments, that were not described in the FSAR, into the modification control program
- Responsibilities to assure the implementation of the above have been delineated in writing.

3.3 Implementation and Findings

The engineering groups have been restructured and the engineering manual describes the organization, responsibilities and methods of implementation. Supplementary training for Systems Engineers is scheduled for the latter part of this year and will be similar to that currently being given to the Salem Station System Engineers.

The course was reviewed and discussed with Training Department managers and instructors. It is the SRO course without the requirements of memorization of operating procedures and other details determined to be unnecessary for engineers. The Licensed Operator Instructors will also present the Hope Creek course including simulator training. The training uses a Qualification Card system and the "objectives" are tasks extracted from the published System Engineer Job Analysis.

It was determined that this area is ready to support plant operations.

No violations were identified.

4. QA/QC Interfaces

4.1 Audits

Audit S-NM-85-13, Engineering and Plant Betterment, was completed during this inspection. It was conducted by five auditors and a technical specialist (design engineer) using approximately 80 man days. The auditors determined that the engineering program had been improved but identified some implementation problems. The audit results were discussed with the team leader and it was concluded that the identified deficiencies would not significantly impact the support provided for plant operations. The licensee acknowledged the statement that this area would be periodically reviewed during subsequent routine NRC inspections.

No violations were identified.

4.2 Training

The QA/QC Inspector Training program was reviewed and discussed with Training Department managers and instructors. The individual courses were developed along INPO Guideline 84-003 and the relevant Job/Task Analysis. Subjects such as hydraulics, fasteners, valve operators and welding were addressed. It was noted that most of the training courses involved hands-on application. The program was based on sound and valid educational principles. Student and instructor texts had been developed and were to be used for each course.

Exams have been developed by QA to determine the level of knowledge of individuals and are currently being administered to various department personnel. A test failure will indicate that the individual requires additional training and specific courses from the above training program will be given to those individuals.

No violations were identified.

5. Management Meetings

License management was informed of the scope and purpose of the inspection at the entrance interview on January 6, 1986. The findings of the inspection were discussed with licensee representatives during the course of the inspection and presented to licensee management at the January 15, 1986 exit interview (see paragraph 1 for attendees).

At no time during the inspection was written material provided to the licensee by the inspectors.