NEW YORK POWER AUTHORITY

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

EFFLUENT AND WASTE DISPOSAL

SEMIANNUAL REPORT

JANUARY 1988 - JUNE 1988

DOCKET NO.: 50-333

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SUPPLEMENTAL INFORMATION

FACILITY: JAFNPP LICENSEE: NEW YORK POWER AUTHORITY

1. Regulatory Limits

a. Fission and activation gases:

(1) The dose rate at or beyond the site boundary due to radioactive materials released from the plant in gaseous effluents shall be limited as follows:

(a) Less than or equal to 500 mrem/year to the whole body and less than or equal to 3000 mrem/year to the skin from noble gases.

(2) The air dose to areas at or beyond the site boundary from noble gases released from the plant in gaseous effluents shall be limited:

(a) During any calendar quarter, to less than or equal to 5 mrad from gamma radiation, and less than or equal to 10 mrad from beta radiation; and,

(b) During any calendar year, to less than or equal to 10 mrad from gamma radiation and less than or equal to 20 mrad from beta radiation.

b. Tritium, iodines and particulates, half lives > 8 days:

(1) The dose to a member of the public at or beyond the site boundary from Iodine-131, Iodine-133, Tritium, and radionuclides in particulate form with half-lives greater than 8 days released from the plant in gaseous effluents shall be limited:

(a) During any calendar quarter to less than or equal to 7.5 mrem to any organ; and,

(b) During any calendar year to less than or equal to 15 mrem to any organ.

(c) Less than 0.1% of the limits of Specification 3.4.a.1 and 3.4.a.2 as a result of burning contaminated oil.

(2) The dose rate at or beyond the site boundary due to radioactive materials released from the plant in gaseous effluents shall be limited as follows:

(a) Less than or equal to 1500 mrem/year to any organ from Iodine-131, Iodine-133, Tritium and for radioactive materials in particulate form with half-lives greater than 8 days (inhalation pathway only).

c. Liquid effluents:

(1) The concentration of radioactive materials released to the unrestricted areas shall not exceed the values specified in 10 CFR 20, Appendix B, Table II, Column 2. For dissolved or entrained noble gases the concentration shall be limited to 2.00E-04 µCi/ml.

(2) The dose to a member of the public from radioactive materials released from the plant in liquid effluents to unrestricted areas shall be limited as follows:

(a) During any calendar quarter, limited to less than or equal to 1.5 mrem to the whole body and to less than or equal to 5 mrem to any organ; and,

(b) During any calendar year, limited to less than or equal to 3 mrem to the whole body and to less than or equal to 10 mrem to any organ.

2. Maximum Permissible Concentrations

a.	Fission and activation gases:	(None specified)	
b.	Iodines:	(None specified)	
c.	Particulates, half-lives >8 days:	(None specified)	
d.	Liquid effluents:	Quarter 1	Quarter 2
	(1) Fission and Activation Products (Mixture MPC)	1.54E-05	8.43E-07
	(2) Tritium	3.00E-03	3.00E-03
	(3) Dissolved and Entrained Gases	2.00E-04	2.00E-04

3. Average Energy, $\bar{E}$: (None specified)

4. Measurements and Approximations of Total Radioactivity

a. Fission and activation gases: Continuous monitor on each release path calibrated to Marinelli grab sample analyzed by gamma spectrospcopy; bubbler grab sample analyzed for tritium.

b. Iodines: Gamma spectral analysis of charcoal cartridge and particulate filter on each release path.

c. Particulates: Gamma spectral analysis of particulate filter on each release path.

d. Liquid effluents: Gamma spectral analysis of each batch discharged, except composite analysis for Sr-89, Sr-90, Fe-55, and tritium.

5. Batch Releases

Liquid:

(1) Number of batch releases:	3.00E+01
(2) Total time period for batch release:	2.37E+03 (min)
(3) Maximum time period for batch release:	1.06E+02 (min)
(4) Average time period for batch release:	7.90E+01 (min)
(5) Minimum time period for batch release:	4.60E+01 (min)

6. Abnormal Releases

	Quarter 3	Quarter 4
a. Liquid:		
(1) Number of releases:	None	None
(2) Total activity released:	None	None
b. Gaseous:		
(1) Number of releases:	None	None
(2) Total activity released:	None	None

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TABLE 1A
GASEOUS EFFLUENTS--SUMMATION OF ALL RELEASES

	UNIT	QUARTER 1	QUARTER 2	EST TOTAL ERROR %
A. FISSION AND ACTIVATION GASES				
1. Total release	Ci	3.95E+02	5.88E+02	≤25%
2. Average release rate for period	µCi/sec	5.02E+01	7.48E+01	
3. Tech. Spec. Limit	%	*	*	
B. IODINES-131				
1. Total iodine-131	Ci	2.52E-03	8.32E-03	≤25%
2. Average release rate for period	µCi/sec	3.21E-04	1.06E-03	
3. Tech. Spec. Limit	%	*	*	
C. IODINE-133 AND PARTICULATES				
1. Iodine-133 and Particulates with half lives >8 days	Ci	5.92E-03	3.10E-02	≤25%
2. Average release rate for period	µCi/sec	7.53E-04	3.94E-03	
3. Tech. Spec. Limit	%	*	*	
4. Gross alpha radioactivity	Ci	3.29E-05	8.63E-06	≤25%
D. TRITIUM				
1. Total release	Ci	3.74E+00	3.53E+00	≤25%
2. Average release rate for period	µCi/sec	4.76E-01	4.49E-01	
*E. PERCENT OF TECHNICAL SPECIFICATION LIMITS				
FISSION AND ACTIVATION GASES				
1. Quarterly gamma air dose limit	%	2.77E-01	3.34E-01	
2. Quarterly beta air dose limit	%	8.87E-02	7.89E-02	
3. Yearly gamma air dose limit	%	1.38E-01	1.67E-01	
4. Yearly beta air dose limit	%	4.44E-02	3.95E-02	
5. Whole body dose rate limit	%	2.66E-03	3.20E-03	
6. Skin dose rate limit	%	7.03E-04	8.23E-04	
HALOGENS, TRITIUM AND PARTICULATES WITH HALF-LIVES >8 DAYS				
7. Quarterly dose limit (organ)	%	3.72E-01	1.32E+00	
8. Yearly dose limit (organ)	%	1.86E-01	6.61E-01	
9. Organ dose rate limit	%	1.22E-04	5.26E-04	

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TABLE 1B
GASEOUS EFFLUENTS--ELEVATED RELEASE

NUCLIDES RELEASED	UNIT	CONTINUOUS MODE QUARTER 1	QUARTER 2
1. Fission gases			
Argon-41	Ci	2.84E+00	8.00E+00
Krypton-85m	Ci	2.47E+01	1.14E+02
Krypton-87	Ci	1.17E-01	ND
Krypton-88	Ci	1.44E+01	5.22E+01
Xenon-133	Ci	2.48E+01	2.67E+02
Xenon-135	Ci	1.13E+00	4.24E+00
Xenon-135m	Ci	ND	1.93E+00
Xenon-133m	Ci	4.43E-01	ND
Xenon-138	Ci	5.30E-01	8.84E-02
2. Iodines			
Iodine-131	Ci	5.87E-04	1.75E-04
Iodine-133	Ci	4.71E-04	1.52E-04
3. Particulates			
Strontium-89	Ci	2.04E-05	1.64E-05
Strontium-90	Ci	5.90E-08	2.98E-08
Cesium-134	Ci	1.88E-07	ND
Barium/ Lanthanum-140	Ci	1.82E-05	1.43E-05

There are no batch releases for this period.

ND - None Detected.

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TABLE 1C
GASEOUS EFFLUENTS--GROUND LEVEL RELEASES

NUCLIDES RELEASED	UNIT	CONTINUOUS MODE QUARTER 1	CONTINUOUS MODE QUARTER 2
1. Fission gases			
Krypton-85m	Ci	1.05E+01	7.70E+00
Krypton-87	Ci	3.74E-01	5.53E+00
Krypton-88	Ci	1.96E+01	4.45E+00
Xenon-133	Ci	1.84E+02	3.53E+01
Xenon-135	Ci	2.14E+01	8.68E+00
Xenon-133m	Ci	2.97E+01	6.58E+00
Xenon-135m	Ci	3.72E+00	7.83E+00
Xenon-138	Ci	5.60E+01	6.53E+01
2. Iodines			
Iodine-131	Ci	1.93E-03	8.14E-03
Iodine-133	Ci	4.01E-03	2.95E-02
3. Particulates			
Chromium-51	Ci	8.01E-04	5.79E-04
Manganese-54	Ci	2.01E-05	6.18E-05
Cobalt-58	Ci	6.20E-05	1.59E-05
Cobalt-60	Ci	1.17E-04	9.14E-05
Strontium-89	Ci	8.10E-05	2.10E-04
Strontium-90	Ci	8.43E-07	1.21E-06
Cesium-134	Ci	1.57E-05	8.53E-07
Cesium-137	Ci	2.86E-05	1.65E-05
Barium/ Lanthanum-140	Ci	2.73E-04	3.33E-04
Cerium-141	Ci	ND	2.21E-07

There are no batch releases for this period.

ND - None detected.

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TABLE 2A
LIQUID EFFLUENTS--SUMMATION OF ALL RELEASES

	UNIT	QUARTER 1	QUARTER 2	EST TOTAL ERROR %
A. FISSION AND ACTIVATION PRODUCTS				
1. Total release (not including tritium and alpha)	Ci	2.40E-02	2.18E-03	≤25%
2. Average diluted concentration during period	µCi/ml	1.37E-10	1.11E-11	
3. Applicable limit	%	8.90E-04	1.32E-03	
B. TRITIUM				
1. Total release	Ci	2.78E+00	3.31E-01	≤25%
2. Average diluted concentration during period	µCi/ml	1.59E-08	1.69E-09	
3. Applicable limit	%	5.29E-04	5.63E-05	
C. DISSOLVED AND ENTRAINED GASES				
1. Total release	Ci	1.09E-04	5.59E-04	≤25%
2. Average diluted concentration during period	µCi/ml	6.23E-13	2.85E-12	
3. Applicable limit	%	3.12E-07	1.43E-06	
D. GROSS ALPHA RADIOACTIVITY				
1. Total release	Ci	3.83E-08	4.25E-08	≤25%
E. VOLUME OF WASTE RELEASED (PRIOR TO DILUTION)	liters	1.32E+06	1.58E+05	
F. VOLUME OF DILUTION WATER USED DURING PERIOD	liters	1.75E+11	1.96E+11	
*G. PERCENT TECHNICAL SPECIFICATION LIMITS				
1. Quarterly whole body dose	%	4.03E-01	1.13E-02	
2. Quarterly organ dose	%	1.77E-01	5.00E-03	
3. Annual whole body dose	%	2.02E-01	5.67E-03	
4. Annual organ dose	%	8.85E-02	2.50E-03	

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TABLE 2B
LIQUID EFFLUENTS

NUCLIDES RELEASED	UNIT	CONTINUOUS MODE QUARTER 1	QUARTER 2
Hydrogen-3	Ci	2.78E+00	3.31E-01
Sodium-24	Ci	4.41E-04	6.80E-05
Chromium-51	Ci	1.34E-04	ND
Manganese-54	Ci	6.45E-04	ND
Cobalt-58	Ci	1.40E-05	ND
Cobalt-60	Ci	9.42E-03	3.17E-04
Zinc-65	Ci	1.69E-04	ND
Iodine-131	Ci	1.07E-04	4.74E-04
Iodine-133	Ci	1.35E-05	2.75E-04
Iodine-135	Ci	ND	5.94E-05
Xenon-133	Ci	2.56E-05	2.62E-04
Xenon-135	Ci	8.34E-05	2.98E-04
Cesium-134	Ci	3.50E-03	1.04E-04
Cesium-137	Ci	9.21E-03	3.24E-04
Neptunium-239	Ci	3.34E-04	ND

There were no continuous mode discharges during this period.

ND - None detected.

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TABLE 3A
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A. SOLID WASTE SHIPPED OFF SITE FOR DISPOSAL (NOT IRRADIATED FUEL)

1. Type of Waste	UNIT	SIX-MONTH PERIOD	EST. TOTAL ERROR %
a. Spent resins, filter sludges evaporator bottoms, etc.	m^3	64.8	<10%
	Ci	168	≤25%
b. Dry compressible waste, contaminated equipment, etc.	m^3	100.8	<10%
	Ci	3.08	≤25%
c. Irradiated components	None		
d. Other	None		

2. Estimate of Major Nuclide Composition

Isotope	% Abundance
Co-60	51.9
Fe-55	26.2
Mn-54	7.0
Cs-137	5.1
Cr-51	2.4
Zn-65	2.3
Cs-134	1.8
Co-58	1.7
Ni-63	1.0

3. Solid Waste Disposition

Shipment	Qty	Transportation Mode	Destination
Type a	14	Truck	Barnwell, SC
Type b	3	Truck	Richland, WA

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TABLE 3B
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A. NRC CLASS "A"

1. Solidified Bead Resin

Container Type:	181.7 Ft^3 Steel Liner (LSA)
Solidification Media:	Cement
Total Volume:	181.7 ft^3 5.1 m^3
No. of Shipments:	1
Type:	Cask
Mode:	Truck

Principal Isotopes	Curies	% Abundance	Quantity Determination
Co-60	1.95	38.8	M
Co-58	1.16	23.1	M
Mn-54	0.85	16.9	M
Cs-134	0.38	7.6	M
Cs-137	0.30	5.9	M
Fe-55	0.12	2.4	E
Zn-65	0.11	2.3	M
C-14	0.05	1.0	E

TOTAL: 5.02 Curies

E = Estimated

M = Measured

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TABLE 3B
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS (CONTINUED)

A. NRC CLASS "A" (CONT'D)

2. Dewatered Powdered Resin/Filter Sludge

Container Type:	158.1 Ft^3 Ploy Hic
Solidification Media:	None
Total Volume:	474.3 Ft^3 13.4 m^3
No. of Shipments:	3
Type:	Cask
Mode:	Truck

Principal Isotopes	Curies	% Abundance	Quantity Determination
Co-60	41.9	50.8	M
Fe-55	19.3	23.4	E
Mn-54	7.4	8.9	M
Cs-137	4.2	5.1	M
Cr-51	2.9	3.5	M
Zn-65	2.1	2.5	M
Cs-134	1.6	2.0	M
Co-58	1.5	1.8	M
Ni-63	0.9	1.0	E

TOTAL: 82.4 Curies

E = Estimated

M = Measured

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TABLE 3B
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS (CONTINUED)

A. NRC CLASS "A" (CONT'D)

3. Solidified Evaporator Bottoms

Container Type:	181.7 Ft^3 Steel Liner (LSA)
Solidification Media:	Cement
Total Volume:	545.1 Ft^3 15.4 m^3
No. of Shipments:	3
Type:	Cask
Mode:	Truck

Principal Isotopes	Curies	% Abundance	Quantity Determination
Co-60	2.04	49.4	M
Fe-55	1.28	31.0	E
Mn-54	0.25	6.1	M
Cs-137	0.24	5.9	M
Cs-134	0.09	2.2	M
Zn-65	0.08	2.0	M
Co-58	0.07	1.8	M
Cr-51	0.05	1.2	M

TOTAL: 4.1 Curies

E = Estimated

M = Measured

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TABLE 3B
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS (CONTINUED)

A. NRC CLASS "A" (CONT'D)

4. Solidified Evaporator Bottoms/Powdered Resin

Container Type:	181.7 Ft^3 Steel Liner (LSA)
Solidification Media:	Cement
Total Volume:	1090.2 Ft^3 30.9 m^3
No. of Shipments:	6
Type:	Cask
Mode:	Truck

Principal Isotopes	Curies	% Abundance	Quantity Determination
Co-60	41.2	54.0	M
Fe-55	23.3	30.5	E
Cs-137	3.7	4.9	M
Mn-54	3.3	4.3	M
Zn-65	1.6	2.1	M
Cr-51	1.1	1.4	M
Cs-134	0.86	1.1	M
Ni-63	0.75	1.0	E

TOTAL: 76.2 Curies

E = Estimated

M = Measured

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TABLE 3B
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS (CONTINUED)

A. NRC CLASS "A" (CONT'D)

5. Compacted/Noncompacted Dry Waste

Container Type:	107 Ft^3 Steel Box (LSA)
Solidification Media:	None
Total Volume:	2996 Ft^3 84.8 m^3
No. of Shipments:	3
Type:	Flatbed
Mode:	Truck

Principal Isotopes	Curies	% Abundance	Quantity Determination
Co-60	0.33	51.1	E
Fe-55	0.15	23.5	E
Mn-54	0.049	7.6	E
Cs-137	0.044	6.9	E
Zn-65	0.021	3.2	E
Cs-134	0.019	2.9	E
Cr-51	0.016	2.5	E
Co-58	0.008	1.2	E
Ni-63	0.006	1.0	E

TOTAL: 0.643 Curies

E = Estimated

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TABLE 3B
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS (CONTINUED)

A. NRC CLASS "A" (CONT'D)

6. Compacted/Noncompacted Dry Waste

Container Type:	92 Ft^3 Steel Box (LSA)
Solidification Media:	None
Total Volume:	460 Ft^3 13.0 m^3
No. of Shipments:	1
Type:	Flatbed
Mode:	Truck

Principal Isotopes	Curies	% Abundance	Quantity Determination
Co-60	0.038	51.0	E
Fe-55	0.018	23.5	E
Mn-54	0.006	7.6	E
Cs-137	0.005	6.9	E
Zn-65	0.002	3.2	E
Cs-134	0.002	2.9	E
Cr-51	0.002	2.6	E
Co-58	0.001	1.2	E
Ni-63	0.001	1.0	E

TOTAL: 0.075 Curies

E = Estimated

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TABLE 3B
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS (CONTINUED)

A. NRC CLASS "A" (CONT'D)

7. Compacted/Noncompacted Dry Waste

Container Type:	7.5 Ft^3 55 Gallon Drum
Solidification Media:	None
Total Volume:	105 Ft^3 3.0 m^3
No. of Shipments:	1
Type:	Cask
Mode:	Truck

Principal Isotopes	Curies	% Abundance	Quantity Determination
Co-60	1.21	51.2	M
Fe-55	0.56	23.6	E
Mn-54	0.18	7.5	M
Cs-137	0.16	7.0	M
Zn-65	0.08	3.2	M
Cs-134	0.07	2.9	M
Cr-51	0.06	2.5	M
Co-58	0.03	1.2	M
Ni-63	0.02	1.0	E

TOTAL: 2.36 Curies

E = Estimated

M = Measured

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ATTACHMENT NO. 1

SEMIANNUAL SUMMARY OF HOURLY METEOROLOGICAL DATA

In accordance with Amendment 93 of the James A. Fitzpatrick Nuclear Power Plant Technical Specifications, an annual summary of hourly meteorological data shall be included and submitted in the Semiannual Radioactive Effluent Release Report within 60 days after January 1 of each year. Meteorological data for period of January 1, 1988 through June 30, 1988 will be included in the second half of the 1988 report.

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ATTACHMENT NO. 2

SUMMARY OF CHANGES TO THE OFFSITE DOSE CALCULATION MANUAL AND PROCESS CONTROL PROGRAM

In accordance with Section 7.3.C.3 of Amendment 93 to the James A. Fitzpatrick Nuclear Power Plant Technical Specifications, changes made to the Process Control Program (PCP) and Offsite Dose Calculation Manual (ODCM) during the reporting period are to be included in the Semiannual Radiological Effluent Release Report.

No changes were made to the PCP during this report period.

No changes were made to the ODCM during this report period.

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ATTACHMENT NO. 3

SUMMARY OF CHANGES TO THE OFFSITE DOSE CALCULATION MANUAL AND PROCESS CONTROL PROGRAM

In accordance with Section 7.3.C.3 of Amendment 93 to the James A. FitzPatrick Nuclear Power Plant Technical Specifications, a listing of new locations for dose calculation and/or environmental monitoring identified by the land use census shall be included in the Radioactive Effluent Release Reports.

1) CHANGES IN ENVIRONMENTAL MONITORING LOCATIONS

No change in Environmental Monitoring Locations was required based on the land use census.

2) NEW LOCATIONS FOR DOSE CALCULATIONS

Based on the most recent land use census, a new location has been identified for dose calculation for the consumption of fresh fruits/vegetables and stored fruits/vegetables 0.9 miles East (82°) of the plant.

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ATTACHMENT NO. 4

DEVIATIONS FROM THE REQUIRED ENVIRONMENTAL SAMPLING SCHEDULE

In accordance with Section 7.3.C.7 of Amendment 93 to the James A. FitzPatrick Nuclear Power Plant Technical Specifications, the cause for unavailability of any environmental samples required shall be included in the Radioactive Effluent Release Report.

EXCEPTION TO THE ENVIRONMENTAL SAMPLING PROGRAM

1. The air sampling pump at R-3, off-site Environmental Sampling Station was inoperable from January 01, 1988 (1036 hours) to February 22, 1988 (0833 hours). Inoperability was caused by a pump mechanical failure.

2. The air sampling pump at R-4, off-site Environmental Sampling Station was inoperable from May 08, 1988 (2324 hours) to May 10, 1988 (0900 hours). Inoperability was caused by a pump mechanical failure.

James A. FitzPatrick
Nuclear Power Plant
P.O. Box 41
Lycoming, New York 13093
315 342.3840

New York Power Authority

Radford J. Converse
Resident Manager

August 25, 1988
JAFP-88-0798

United States Nuclear
Regulatory Commission
Region 1
475 Allendale Road
King of Prussia, PA 19406

Attention: Mr. William T. Russell
Regional Administrator

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333, LICENSE NO. DPR-59

Gentlemen:

Attached is the Semiannual Radioactive Effluent Release Report for the period of January 1, 1988 through June 30, 1988. This report is submitted in accordance with the requirements of Amendment 93, Appendix B, Section 7.3.C of the James A. FitzPatrick Nuclear Power Plant Technical Specifications.

The format used for the effluent data is as outlined in Appendix B of Regulatory Guide 1.21, Revision 1. Distribution is in accordance with Regulatory Guide 10.1, Revision 4.

Very Truly Yours,

RADFORD J. CONVERSE

RJC:WH:jmb
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