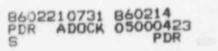
NRC Form 364 (9-83)							APPRO	CLEAR REGULATORY COMMISSION PPROVED OMB NO. 3150-0104 XPIRES 8/31/85								
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			COMPL	ETE ONE LINE FOR	REACH COMPONEN	T FAILUR	DESCRIBE	D IN THIS REPO	ORT (13)							
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Safet Gener eleva the t Syste with Subse resul relie trip cause	ty I rato ated trip pla eque lted ef s and ed a	njecti r "B". T ave and t actuat nt pro nt inv when etpoin safet sudde	ion Signal Control rage. Pl that the r ted proper ocedures. restigatio up to two its. The y injection of the the the the the the the the the the the the the the the the	from the rod test ant opera eactor tr ly. The n revealed Steam Gen rate compo on to mit steam lin	ing in the rate comp ing was in tors verif ip breaker restoration d the rate nerator "B ensated st igate the ne pressur e a Safety	pensat prog fied t rs wer on fro e comp " saf conse re whi	ed st ress hat a e ope om the ensat ety r ine l quence ch ca	eam line at this 11 rods n. All SI was ed steam elief va ow press es of a used the	e low time were Engi perf lin lves ure stea	e low open open SI is m lin	sur y i d S l in ed an e b	re c sli inse iafe ac bel an rea	off c ightl ertec ety F cord ow t tici k.	of St ly laft eatu lance SI .he pato This	er re	
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NRC Form 366A (9-83) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO 3: 9- EXPIRES: 8/31/88								
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER	(6)	PAGE (3)				
Millstone Nuclear Power Station		YEAR SEQUENTI	AL REVISION					
Unit 3	0 15 10 10 10 14 2 3	86 -001	3 _ 010	012 OF 012				

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1724 on 1/19/86, while operating in the Hot Standby mode, the plant received a Safety Injection Signal from the rate compensated steam line low pressure off of Steam Generator "B". Control rod testing was in progress at this time with a slightly elevated T average. Plant operators verified that all rods were fully inserted after the trip and that the reactor trip breakers were open. All Engineered Safety Feature Systems actuated properly. The restoration from the SI was performed in accordance with plant procedures.

An investigation revealed the rate compensated steam line pressure SI resulted when steam line pressure experienced a step decrease of approximately 60 psi when up to two safety valves on Steam Generator "B" relieved at 1165 psig. 1165 psig is 8 psi & 20 psi below the lowest and next lowest safety valve relief setpoints, respectively. The Maintenance Department checked and reset the relief setpoints on the valves that relieve at the lowest and next lowest pressures for Steam Generator "B". Both valves required minor readjustment. The lowest and next lowest pressure safety valves were checked on the "A" Steam Generator and required no adjustment.

LER 86-001 reports a similar SI when the Atmospheric Steam Dump Valve on Steam Generator "A" opened too quickly. As a result of LER 86-001, the rate compensated steam line low pressure circuitry was checked and has been verified to be functioning correctly.

There were no safety implications to the public as all equipment performed its intended safety function.

This report is being submitted as required by 10CFR50.73 (a) (2) (iv).



P.O. BOX 270 HARTFORD, CONNECTICUT 06101 (203) 666-6911

February 14, 1986 MP-8721

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49 Docket No. 50-423 Licensee Event Report 50-423/86-003-00

Gentlemen:

This letter forwards Licensee Event Report 86-003-00 required to be submitted within thirty days pursuant to 10CFR50.73 (a) (2) (iv); any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS).

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: Wayne D. Romberg Station Superintendent Millstone Nuclear Power Station

BY: James J. Kelley Station Services Superintendent Millstone Nuclear Power Station

WDR/PGA:se

Attachment: LER 86-003-00

cc: Dr. T. E. Murley, Region I