NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	1 OF 0 2

TITLE (4) Reactor Trip with SI due to Low Steam Line Pressure

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 1	1 9	8 6	8 6	— 0 0 3	— 0 0	0 2	1 4	8 6		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	[X] 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Paul G. Atkinson III, Associate Engineer	AREA CODE 2 0 3 — 4 4 7 — 1 7 9 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	S B	R V	C 5 6 8	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
[ ] YES (If yes, complete EXPECTED SUBMISSION DATE)	[X] NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1724 on 1/19/86, while operating in the Hot Standby mode, the plant received a Safety Injection Signal from the rate compensated steam line low pressure off of Steam Generator "B". Control rod testing was in progress at this time with a slightly elevated T average. Plant operators verified that all rods were fully inserted after the trip and that the reactor trip breakers were open. All Engineered Safety Feature Systems actuated properly. The restoration from the SI was performed in accordance with plant procedures.

Subsequent investigation revealed the rate compensated steam line low pressure SI resulted when up to two Steam Generator "B" safety relief valves opened below the relief setpoints. The rate compensated steam line low pressure SI is an anticipatory trip and safety injection to mitigate the consequences of a steam line break. This caused a sudden drop in steam line pressure which caused the rate compensated steam line pressure circuit to generate a Safety Injection.

This report is being submitted in accordance with 10CFR50.75 (a) (2) (iv).

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	8 6	— 0 0 3	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1724 on 1/19/86, while operating in the Hot Standby mode, the plant received a Safety Injection Signal from the rate compensated steam line low pressure off of Steam Generator "B". Control rod testing was in progress at this time with a slightly elevated T average. Plant operators verified that all rods were fully inserted after the trip and that the reactor trip breakers were open. All Engineered Safety Feature Systems actuated properly. The restoration from the SI was performed in accordance with plant procedures.

An investigation revealed the rate compensated steam line pressure SI resulted when steam line pressure experienced a step decrease of approximately 60 psi when up to two safety valves on Steam Generator "B" relieved at 1165 psig. 1165 psig is 8 psi & 20 psi below the lowest and next lowest safety valve relief setpoints, respectively. The Maintenance Department checked and reset the relief setpoints on the valves that relieve at the lowest and next lowest pressures for Steam Generator "B". Both valves required minor readjustment. The lowest and next lowest pressure safety valves were checked on the "A" Steam Generator and required no adjustment.

LER 86-001 reports a similar SI when the Atmospheric Steam Dump Valve on Steam Generator "A" opened too quickly. As a result of LER 86-001, the rate compensated steam line low pressure circuitry was checked and has been verified to be functioning correctly.

There were no safety implications to the public as all equipment performed its intended safety function.

This report is being submitted as required by 10CFR50.73 (a) (2) (iv).

NORTHEAST UTILITIES

THE CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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February 14, 1986
MP-8721

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49
Docket No. 50-423
Licensee Event Report 50-423/86-003-00

Gentlemen:

This letter forwards Licensee Event Report 86-003-00 required to be submitted within thirty days pursuant to 10CFR50.73 (a) (2) (iv); any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS).

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

BY: James J. Kelley
Station Services Superintendent
Millstone Nuclear Power Station

WDR/PGA:se

Attachment: LER 86-003-00

cc: Dr. T. E. Murley, Region I

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