NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
River Bend Station	0 5 0 0 0 4 5 8	1 OF 0 3

TITLE (4)
Feedwater Testable Check Valve Returned to Service Without Operability Test

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 1	1 0	8 6	8 6	0 0 6	0 0	0 2	0 9	8 6		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	4	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(a)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(a)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(a)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER AREA CODE
D. E. Zemel	5 0 4 6 3 5 - 6 0 9 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 01/10/86 at 1400 with the unit in operational condition 4 (cold shutdown), it was discovered that maintenance activities had previously been performed on feedwater testable check valve 1B21*AOVF032A without the performance of an operability test prior to returning the valve to service. This error was caused by procedure requirements being misinterpreted by Mechanical Maintenance personnel. As a result of the error, Technical Specification requirements for the valve were not satisfied. The feedwater valve was demonstrated operable on 01/10/86 by the successful completion of Surveillance Test Procedure STP-107-3301. There were no safety consequences to the public as a result of this event.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 11/12/85 Maintenance Work Request (MWR) 10672 was issued requesting maintenance to repair a steam leak on feedwater testable check valve 1B21*AOVF032A. The MWR was planned and processed to be worked in accordance with the procedural requirements of Administrative Procedure ADM-0028 "Maintenance Work Request."

On 11/14/85, prior to implementing the MWR, Mechanical Maintenance personnel inspected the valve leak for more detail. While performing the field inspection it was determined that the valve only required a stem packing adjustment to correct the leak. The necessary packing adjustments were made during the field inspection without the use of the MWR.

During a review of MWR 10672, it was noted that the Mechanical Maintenance personnel misinterpreted the requirements of ADM-0028 when performing the valve packing adjustments without the use of an MWR. The instructions provided in ADM-0028 for adjusting valve packing without the use of an MWR applies only to manual valves. Upon discovering this condition, Mechanical Maintenance foremen were immediately advised of the event and the correct interpretation of the requirements delineated in ADM-0028. MWR 10672 was revised and resubmitted to include applicable testing requirements for verifying the valve operable.

On 01/10/86 Condition Report 86-042 was issued noting that an operability test had not been performed on 1B21*AOVF032A following the

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

packing adjustment that was made on 11/14/86. Discovery of this condition also identified that Technical Specifications require this valve to be demonstrated operable prior to being returned to service after the performance of maintenance. As a result of the operability test not being performed, Technical Specification Sections 3.6.4 and 3.0.4 were not satisfied.

The subject valve was demonstrated operable on 01/10/86 by the successful performance of Surveillance Test Procedure STP-107-3301 "Condensate Makeup and Drawoff Valve Operability Test." In an effort to prevent recurrence, documented required reading was conducted for all Mechanical Maintenance foremen on 01/21/86 on the requirements of ADM-0028.

There were no safety consequences to the public as a result of this event.

GULF STATES UTILITIES COMPANY

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775

AREA CODE 504 635-6094 346-8651

February 9, 1986
RBG- 23153
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

River Bend Station - Unit 1
Docket No. 50-458

Please find enclosed Licensee Event Report No. 86-006 for River Bend Station - Unit 1. This report is submitted pursuant to 10CFR50.73.

Sincerely,

Eddie R Grant
for J. E. Booker
Manager-Engineering,
Nuclear Fuels & Licensing
River Bend Nuclear Group

JEB/TFP/DRG/BEH/ebm

cc: U.S. Nuclear Regulatory Commission
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Arlington, TX 76011

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1100 Circle 75 Parkway
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