OPERATING DATA REPORT

Docket No.50-272
Date February 10, 1986
Telephone 935-6000
Extension 4455

Completed by Pell White

Operating Status

1. 2. 3. 4. 5. 6. 7. 8.	Unit Name Reporting Period Licensed Thermal Power (MWt) Nameplate Rating (Gross MWe) Design Electrical Rating (Net MW Maximum Dependable Capacity (Gros Maximum Dependable Capacity (Net If Changes Occur in Capacity Rat Report, Give Reason	a) 3338 1170 1090 s MWe) 1124 MWe) 1079	Notes 3 through 7) si	nce Last
9.	Power Level to Which Restricted,	if any (Net	MWe) N/A	
10.	Reasons for Restrictions, if any		N/A	
		This Month	Year to Date	Cumulative
11.	Hours in Reporting Period	744	744	75313
	No. of Hrs. Reactor was Critical		711.2	44896.6
	Reactor Reserve Shutdown Hrs.	0	0	3088.4
	Hours Generator On-Line	686.8	682.0	43191.6
	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	2462446	2262446	132746475
17.	Gross Elec. Energy Generated	430-4		
10	(MWH)	753120	753120	44049130
	Net Elec. Energy Generated (MWH)	722106	722106	41827598
	Unit Service Factor	92.3	92.3	57.3
	Unit Availability Factor Unit Capacity Factor	92.3	92.3	57.3
21.	(using MDC Net)	90.1	90.0	51.5
22	Unit Capacity Factor	20.1	70.0	22.3
	(using DER Net)	89.0	89.0	51.0
	Unit Forced Outage Rate	7.7	7.7	28.4

25. If shutdown at end of Report Period, Estimated Date of Startup:

Refueling, March 20, 1986, approximately 60 days

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

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7004

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272

Date February 10,1986

Salem # 1

Unit Name

Completed by Pell White Telephone609-935-6000 Extension 4455 Month January 1986 Day Average Daily Power Level Day Average Daily Power Level (MWe-NET) (MWe-NET)

UNIT SHUTDOWN AND POWER REDUCTIONS REPORT MONTH January 1986

Docket No.50-272 Unit Name Salem No.1 February 10,1986 Date Telephone 609-935-6000 Extension 4455

Completed by Pell White

No.	Date	Type	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
068	0112	F	2.2	A	5	-	WA	НТЕХСН	Condenser Tube & Water Box Claning
100	0116	F	26.0	A	3	-	EB	CKTBRK	4160 Volt Protection Devices
102	0117	F	18.1	A	4	-	НА	VALVEX	Main Stop Valves Turbine
144	0131	F	13.1	A	3	-	СН	VALVEX	Feedwater Regulating Boiler level control Valve

F: Forced S: Scheduled 2 Reason A-Equipment Failure-explain B-Maintenance or Test C-Refueling D-Regulatory Restriction E-Operator Training & Licensing Exam F-Administrative G-Operational Error-explain H-Other-explain

3 Method 1-Manual 2-Manual Scram. 3-Automatic Scram. tion of Data 4-Continuation of Entry Sheets Previous Outage 5-Load Reduction 9-Other

4 Exhibit G Instructions Salem as for Preparafor Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Source

MAJOR PLANT MODIFICATIONS REPORT MONTH JANUARY 1986

DOCKET NO.: 50-272

UNIT NAME: Salem 1
DATE: February 10, 1986

COMPLETED BY: J. Ronafalvy TELEPHONE: 609/339-4455

*DCR NO.

PRINCIPLE SYSTEM

SUBJECT

No major modifications completed this month.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICAT	ION
0099074	1630	1	BAT FILTER	
			FAILURE DESCRIPTION:	U/1 BORIC ACID TRANSFER FILTER IS CREATING RADIOLOGICAL PROBLEM AT THE ENTRANCE TO U/1 DIESELS. DOSE RATES SHOW A HIGH RADIATION.
			CORRECTIVE ACTION:	REMOVED OLD FILTER. INSTALLED NEW FILTER
841028	0400	1	11CS11 VALVE	
	1	the	FAILURE DESCRIPTION:	THE VALVE IS LEAKING BORATED WATER. LEAK IS LOCATED ABOVE THE SCHROUD ON THE DISCHARGE SIDE.
			CORRECTIVE ACTION:	REPACKED VALVE.
850918	0261	1	12 WASTE GAS COMPRESS	SOR
			FAILURE DESCRIPTION:	MECHANICAL SEAL LEAK.
			CORRECTIVE ACTION:	REPLACES SEAL

SALEM UNIT 1

WO NO DEPT	UNIT	EQUIPMENT IDENTIFICAT	ION
8511110208	1	1-A D/G	TO COURT DATED DEFECT OF W/O 8506281369
		FAILURE DESCRIPTION:	WRITTEN TO COVER FAILED RETEST OF W/O 8506281369. OIL IS STILL LEAKING FROM R-1 & R-4.
		CORRECTIVE ACTION:	REPLACED PUMP GASKETS & INJECTION FIRE PORT GASKETS; TIGHTENED ALL FITTINGS.
3511190015	1	FEEDER BKR FOR RC19-3	
		FAILURE DESCRIPTION:	WITH 1BDE2AX11 OPEN THE 1RC19-3 CABINET REMAINS ENERGIZED. THIS IS EVIDENCED BY A LOSS OF 1E5D CONSOLE INDICATION WHEN 1RC19-3-9S IS OPEN.
		CORRECTIVE ACTION:	FOUND WIRING ERROR IN CABINET RESOLDERED WIRES TO CORRESPOND TO PRINT FOR CABINET ON 218515.
3601020151			
	1	14 S/G PRESS	
		FAILURE DESCRIPTION:	14 LOOP PRESSURE CHANNEL HAS AC NOISE FOUND DURING CHANNEL CAL. SURVEILLANCE. REWORK AS NECESSARY.
		CORRECTIVE ACTION:	REPLACED CAPS C2, C3,C4, C6, C7, C8, AND C9 IN IN PM-545A. ADJUSTED TO SPEC.

SALEM UNIT 1

WO NO DEPT	UNIT	EQUIPMENT IDENTIFICAT	ION
8601130011	1	12 BAT PUMP	
		FAILURE DESCRIPTION:	PUMP EXPERIENCED EXCESSIVE SEAL LEAKAGE AND THE SEALS WERE SMOKING.
		CORRECTIVE ACTION:	INSTALLED PUMP & SET NEW MECH SEAL.
8601180019	1	STATUS LIGHTS	
		FAILURE DESCRIPTION:	PLEASE TROUBLE SHOOT ERRONEOUS HIGH FLUX TRIP INDICATION.
		CORRECTIVE ACTION:	FOUND DIODE ON STATUS PANEL FOR 1N42 25% TRIP SHORTED. REPLACED DIODE ASSEMBLY. STATUS PANEL INDICATIONS RETURNED TO NORMAL. 1N42 REMOVED FROM OPERATION DURING REPAIR.

SALEM GENERATING STATION MONTHLY OPERATING SUMMARY - UNIT NO. 1 JANUARY 1986

SALEM NO. 1

The Unit began the period operating at full power. On 1/09/86 at 1515 hours, power was reduced to 82% to remove a buildup of river grasses on the Traveling Screens. Power was restored to 100% at 0450 on 1/10/86. On 1/16/86 at 0638 hours, the Unit tripped as a result of "High Negative Flux Rate on the 4/4 NI Channels". This was caused by the insertion of Shutdown Banks "C" and "D" as a result of a faulty backup power supply in the Rod Control Cabinet coincident with the inadvertent loss of 1A Vital Bus. Upon completion of investigations and repairs, the unit was synchronized with the grid on 1/18/86 at 0246 hours and was returned to full power operation. On 1/31/86 at 1053 hours the Unit tripped due to Steam Flow/Feed Flow mismatch in #11 Steam Generator (i.e., Low Steam Generator Level with Low Feed Flow) as a result of partial closure of valve 11BF19 (Steam Generator Feed Water Control Valve). The Unit was returned to service on 2/02/86 at 2157 hours.

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy	UNIT NAME: DATE: TELEPHONE: EXTENSION:	Salem 1 February 10, 1986 609/935-6000 4455
Month January 1986		
1. Refueling information has change YES	ed from last m	onth:
2. Scheduled date for next refueling	ng: March 2	2, 1986
3. Scheduled date for restart follo	owing refuelin	g: May 20, 1986
4. A) Will Technical Specification amendments be required? YES NOT DETERMINED TO	0	other license
B) Has the reload fuel design Operating Review Committee YES NO If no, when is it	? O X	
5. Scheduled date(s) for submitting February 1986		ensing action:
6. Important licensing consideration NONE	ons associated	with refueling:
7. Number of Fuel Assemblies: A) Incore		193
B) In Spent Fuel Storage		296
8. Present licensed spent fuel stor	rage capacity:	1170
Future spent fuel storage capaca	ity:	1170
9. Date of last refueling that can to spent fuel pool assuming the licensed capacity:		



Public Service Electric and Gas Company P.O. Box E. Hancocks Bridge, New Jersey 08038

Salem Generating Station

February 10, 1986

Director, Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT SALEM NO. 1 DOCKET NO. 50-272

In compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of January 1986 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

Juguetho J.

J. M. Zupko, Jr.

General Manager - Salem Operations

JR:kcb

cc: Dr. Thomas E. Murley

Regional Administrator USNRC

Region I

631 Park Avenue

King of Prussia, PA 19406

Director, Office of Management Information and Program Control U.S. Nuclear Regulatory Commission Washington, DC 20555

Enclosures 8-1-7.R4