HOUSTON LIGHTING AND POWER COMPANY SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION UNIT ONE LICENSE NO. NPF-76

SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT MARCH 8, THROUGH JUNE 30, 1988

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1.0 Introduction

This Semiannual Radioactive Effluent Release Report, for the period March 8, 1988, (initial criticality-Unit 1) to June 30, 1988 is submitted in accordance with Appendix A of License NPF-76. This report will not include data for Unit 2 which is under construction. Separate tables of releases and release totals are included where separate processing systems exist.

In accordance with Technical Specifications 6.9.1.4 the annual summary of hourly meteorological data collected over the previous year shall be included in the Semiannual Radioactive Effluent Release Report to be submitted within 60 days after January 1 of each year. This same report shall include an assessment of the radiation doses due to the radioactive liquid and gaseous effluents released from the unit or station and an assessment of the radiation doses from radioactive liquid and gaseous effluents to MEMBERS OF THE PUBLIC due to their activities inside the SITE BOUNDARY during the reporting period for the previous calendar year. Also included in this report shall be an assessment of radiation doses to the likely most exposed MEMBER OF THE PUBLIC from reactor releases and other nearby uranium fuel cycle sources, including doses from primary effluent pathways and direct radiation for the previous calendar year.

All assessments of radiation coses are performed in accordance with the STPEGS Offsite Dose Calculation Manual (ODCM).

2.0 Supplemental Information for Effluent and Waste Disposal

South Texas Project Electric Generating Station, Houston Lighting and Power Co.

Unit Number 1 Type: PWR Docket No. 50-498 Cooling Water Source: Main Cooling Reservoir Power (MWT)- 3817

Initial Criticality-(March 8, 1988)

2.1 Regulatory Limits

2.1.1 Fission and activation gases

The air dose due to noble gases released in gaseous effluents, from each unit, to areas at and beyond the Site Boundary shall be limited to the following:

- a. During any calendar quarter: Less than or equal to 5 mrads for gamma radiation and less than or equal to 10 mrads for beta radiation, and
- b. During any calendar year: Less than or equal to 10 mrads for gamma radiation and less than or equal to 20 mrads for beta radiation.

2.1.2 Fission and activation gases, iodines, particulates, halflives > 8 days.

The dose rate due to radioactive materials released in gaseous effluents from the site to areas at and beyond the Site Boundary shall be limited to the following:

- a. For noble gases: Less than or equal to 500 mrems/yr to the whole body and less than or equal to 3000 mrems/yr to the skin and
- b. For Iodine-131, for Iodine-133, for tritium, and for all radionuclides in particulate form with half-lives greater than 8 days: Less than or equal to 1500 mrems/yr to any organ.
- 2.1.3 Iodines and Particulates, half-lives > 8 days

The dose to a Member of the Public from Iodine-131, Iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released, from each unit, to areas at and beyond the Site Boundary shall be limited to the following:

- During any calendar quarter: Less than or equal to 7.5 mrems to any organ and,
- b. During any calendar year: Less than or equal to 15 mrems to any organ.

2.1.4 Liquid Effluents

The dose or dose commitment to a Member of the Public from radioactive materials in liquid effluents released, from each unit, to Unrestricted Areas shall be limited to:

- a. During any calendar quarter to less than or equal to 1.5 mrems to the whole body and to less than or equal to 5 mrems to any organ, and
- b. During any calendar year to less than or equal to 3 mrems to the whole body and to less than or equal to 10 mrems to any organ.

2.2 Maximum Permissible Concentrations

2.2.1 Liquid Effluents

The concentration of radioactive material released in liquid effluents to Unrestricted Areas shall be limited to the concentrations specified in 10CFR Fart 20, Appendix B, Table II, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases the concentration shall be limited to 2.0E-04 micro curie/ml total activity.

2.3 Average Energy (MeV/Disintegration)

The Average Energy (or E-bar) is not used at STPEGS to calculate release limits.

2.4 Measurement and Approximations of Total Activity

The following discussions detail the methods used to measure and approximate total activity for the following:

- a. Fission and Activation Gases
- b. Iodines
- c. Particulates
- d. Liquid Effluents

Tables A3-1 and A4-1 of the STPEGS Offsite Dose Calculation Manual (ODCM) give sampling frequencies and minimum detectable concentration requirements for the analysis of liquid and gaseous effluent streams.

2.4.1 Gaseous Effluents

2.4.1.1 Fission and Activation Gases

The following noble gases are considered in evaluating gaseous airborne discharges:

| Ar-41 | Xe-131m |
|--------|---------|
| Kr-83m | Xe-133m |
| Kr-85m | Xe-133 |
| Kr-85 | Xe-135m |
| Kr-87 | Xe-135 |
| Kr-88 | Xe-137 |
| Kr-89 | Xe-138 |
| Kr-90 | |

2.4.1.2 Iodines and Particulates

The radioiodines and radioactive materials in particulate forms to be considered are:

| | Cr-51 | Sb-124 | H-3 |
|---|-----------------|--------|-------|
| | Mn-54 | I-131 | C-14 |
| | Fe-59 | I-133 | Mo-99 |
| | Co-58 | Cs-134 | |
| i | Co-60 | Cs-136 | |
| | Zn-65 | Cs-137 | |
| | Sr-89 | Ba-140 | |
| | Sr-90 | Ce-141 | |
| | Zr-95 | Ce-144 | |
| Ì | Other nuclides | with | |
| ı | half-lives grea | ater | |
| | than 8 days | | |
| | | | |

2.4.1.3 Analytic Methods

a. Batch Gaseous Releases

Pre-release grab samples from the plant containment atmosphere and the RCS Vacuum Degassing System are analyzed on a Gamma Spectroscopy System utilizing high purity germanium detectors (HPGe) for noble gas, iodine and particulate activity. The particulate sample filters are also analyzed for gross alpha on a gas-flow proportional counter. The filter samples are chemically processed to separate strontium for analysis using a gas-flow proportional counter.

The radionuclide values obtained are used in conjunction with the gross noble gas release rate monitoring data collected by the radiation monitoring system to estimate the release rate of each radionuclide in the effluent streams.

b. Continuous Gaseous Releases

Periodic noble gas grab samples are taken from the continuous release points (i.e. the Unit vent and the condenser vacuum pump exhaust). Continuous sampling for particulates and iodine is also performed on the effluent streams. They are analyzed for gross alpha, for gamma radionuclides, and strontium as described above for batch releases.

2.4 2 Liquid Effluents

The radionuclides listed below are considered when evaluating liquid effluents:

| Н3 | Y-90 | I-133 |
|-------|---------|-----------------------------|
| C14 | Y-91m | I-134 |
| Na-24 | Y-91 | I-135 |
| P-32 | Y-92 | Cs-134 |
| Cr-51 | Y-93 | Cs-136 |
| Mn-54 | Zr-95 | Cs-137 |
| Mn-56 | Zr-97 | Cs-138 |
| Fe-55 | Nb-95 | Ba-139 |
| Fe-59 | Mo-99 | Ba-140 |
| Co-58 | Tc-99m | Ba-141 |
| Co-60 | Tc-101 | Ba-142 |
| N1-63 | Ru-103 | La-140 |
| N1-65 | Ru-105 | La-142 |
| Cu-64 | Ru-106 | Ce-141 |
| Zn-65 | Ag-110m | Ce-143 |
| Zn-69 | Te-125m | Ce-144 |
| Br-83 | Te-127m | Pr-143 |
| Br-84 | Te-127 | Pr-144 |
| Br-85 | Te-129m | Nd-147 |
| Rb-86 | Te-129 | W-187 |
| Rb-88 | Te-131m | Np-239 |
| Rm-89 | Te-131 | .LIQ(unidentitied)* |
| Sr-89 | Te-132 | .ALPHA (Gross Alpha) |
| Sr-90 | 1-130 | .Xe-133 |
| Sr-91 | 1-131 | .Xe-135 |
| Sr-92 | I-132 | *includes other gamma peaks |
| | | that are identified |
| | | |

2.4.2.1 Analytic Methods

2.4.2.1 a. Batch Liquid Releases

All liquid effluents are released as batches. Representative pre-release grab samples are taken and analyzed in accordance with Table A3-1 of the ODCM. Radionuclide analyses are performed using the Gamma Spectroscopy System. Aliquots of each pre-release sample are composited in accordance with the requirements in Table A3-1 of the ODCM. Strontium determinations are made by performing a chemical separation and counting he separated radionuclides using the was-flow Proportional Counting System. Gross alpha determinations are made using the Gas-Flow Proportional Counting

System. Tritium concentrations are determined using Liquid Scintillation Counting techniques. Iron determinations are made by performing a chemical separation and counting the separated radionuclide using the Liquid Scintillation Counting System. Dissolved and entrained gases are determined by counting grab samples on the Gamma Spectroscopy System.

The radionuclide concentrations obtained are used with the flow total for each batch release. The error associated with the flow total is small in relation to the counting uncertainty of the radionuclide concentration analysis. The counting uncertainty associated with these measurements is accurate to the 5% significance level for the principle radion, lides released.

2.5 Batch Releases

| 2.5.1 | Liquid | | Quarter 1 | Quarter 2 |
|-------|--------|---|-----------|-----------|
| | a. N | umber of releases: | 0 | 73 |
| | | otal time period for eleases (min): | 0 | 3980 |
| | | faximum time period or a release (min): | | 68 |
| | | verage time period or a release (min): | 0 | 55 |
| | | inimum time period or a release (min): | | 7 |
| .5.2 | Gaseou | s | | |
| | a. Nu | mber of releases: | 0 | 26 |
| | | tal time period r releases (min): | 0 | 31479 |
| | | ximum time period r a release (min): | | 10761 |
| | | erage time period r a release (min): | | 1211 |
| | | nimum time period r a release (min): - 11 - | | 6 |

2.6 Abnormal (Unplanned) Releases

| 2.6.1 | Liquid | Quarter 1 | Quarter 2 |
|-------|---|-----------|-----------|
| | a. Number of releases: | 0 | 2 |
| | b. Total activity released (curies): | 0.000E+00 | 5.126E-02 |
| 2.6.2 | Gaseous | | |
| | a. Number of releases: | 0 | 0 |
| | b. Total activity released (curies): | 0.000E+00 | 0.000E+00 |

2.7 Estimate of Total Error

2.7.1 Liquid

- a. The maximum error associated with volume and flow measurements, based upon plant calibration practice is estimated to be +/-0.08%.
- b. The average error associated with counting uncertainty is accurate to the 5% significance level.

2.7.2 Gaseous

a. The maximum error associated with monitor readings, sample flow, vent flow, sample collection, monitor calibration and laboratory procedures are collectively estimated to be:

| Fission and | | | |
|------------------|--------------|--------------|---------|
| Activation Gases | Iodines | Particulates | Tritium |
| <u>+</u> 25% | <u>+</u> 25% | +25% | +25% |

b. The average error associated with counting uncertainty is accurate to the 5% significance level for fission and activation gases, iodines, particulates and tritium.

2.7.3 Solid Radioactive Waste

To date there have been no radioactive waste shipments to offsite burial facilities. Therefore percent error is not applicable.

2.8 Solid Waste Shipments

No radioactive waste shipments were made during this reporting period.

2.9 Radiological Impact On Man

In accordance with Technical Specifications 6.9.1.4, this data shall be included in the Semiannual Radioactive Effluent Release Report to be submitted within 60 days after January 1 of each year.

2.10 Meteorological Data

In accordance with Technical Specifications 6.9.1.4, this data shall be included in the Semiannual Radioactive Effluent Release Report to be submitted within 60 days after January 1 of each year.

2.11 Lower Limit of Detection (LLD)

The LLD (an a priori limit) is defined as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95% probability, and only a 5% probability of falsely concluding that a blank observation represents a "real" signal.

2.12 Dose to MEMBERS OF THE PUBLIC On Site

In accordance with Technical Specifications 6.9.1.4, this data shall be submitted within 60 days after January 1 of each year.

- 3.0 Technical Specifications Reporting Requir ments
 - 3.1 Radioactive Waste Treatment System Design M ification Description

The following is a list of Unit 1 liquid and solid radioactive waste treatment system design modifications initiated during this reporting period. No modifications were made to the gaseous radioactive waste treatment system. These descriptions are submitted to comply with Section 6.15 of the Technical Specifications.

- 3.1.1 Optional Use of Filter Elements in Influent to Waste Monitor Tanks (Appendix A)
- 3.1.2 Increase in Waste Evaporator Concentrates Operational Levels (Appendix B)

- 3.1.3 Addition of Local Sampling for Waste Monitor Tanks D, E, F (Appendix C)
- 3.1.4 Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks (Appendix D)
- 3.1.5 Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid (Appendix E)
- 3.1.6 Addition of an Isolok Sampler for Spent Resin to Mobile Solidification Unit (Appendix F)
- 3.2 Inoperable Effluent Monitoring Instrumentation Explanation

There were no inoperable liquid or gaseous effluent monitoring instrumentation not corrected within the time specified in Sections 3.3.3.10 or 3.3.3.11 of Technical Specifications. This explanation is submitted to comply with reporting requirements of Section 6.9.1.4 of the Technical Specifications for Unit 1.

3.3 Gas Storage Tank Curie Limit Violation Description

The quantity of radioactive material in the RCS Vacuum Degassing System Storage Tank(s) did not exceed the limits set forth in Section 3.11.2.6 of Technical Specifications during this reporting period. This description is submitted to comply with Section 6.9.1.4 of the Technical Specifications for Unit 1.

3.4 Unprotected Outdoor Tank Curie Limit Violation Description

The quantity of radioactive material in the Unprotected Outdoor Tank(s) did not exceed the limit set forth in Section 3.11.1.4 of Technical Specifications during this reporting period. This description is submitted to comply with Section 6.9.1.4 of the Technical Specifications for Unit 1.

3.5 Abnormal (Unplanned) Release Description

The following is a list of abnormal releases of liquid waste from STPEGS Unit 1 to UNRESTRICTED ARFAS that occurred during this reporting period.

Waste Monitor Tank 1E 05/30/88 23:00 - 05/30/88 23:08

Waste Monitor Tank 1D 06/07/88 18:28 - 06/07/88 18:35

A description of the abnormal release for Waste Monitor Tank 1E is provided in the attached STPEGS Problem Report, Appendix G. The supportive documentation for the problem report, which consisted of copies of logs, a copy of the surveillance package for the release and documentation of training, is not attached but shall be furnished on request. A description of the abnormal release for Waste Monitor Tank 1D is provided in the attached Licensee Event Report, Appendix H. These reports are submitted to comply with Section 6.9.1.4 of the Technical Specifications for Unit 1.

3.6 Radioactive Waste Process Control Program Changes

A HL&P initiated editorial change to the Radioactive Waste Process Control Program (PCP) procedure (HL&P procedure OPGP03-ZO-0017) was completed on February 26, 1988. The change substituted the word "noncompressible" for "noncompactible" in all references to "noncompactible" receptacles. This change was to reflect the wording used in the Dry Active Waste Collection, Sorting and Segregation Procedure (OPGP14-WS-0010) and on the DAW receptacles. Refer to attached copies of the Field Change Request and the Radioactive Waste Process Control Program Procedure pages 5 and 26 in Appendix I. The change did not reduce overall conformance of the solidified waste product to existing criteria for solid waste and because it was editorial in nature, it did not require PORC approval.

This notification of change is submitted to comply with Section 6.13.2 of the Technical Specifications for STPEGS Unit 1.

3.7 Offsite Dose Calculation Manual Change

No changes were made to the Offsite Dose Calculation Manual (ODCM) during this reporting period. This description is submitted to comply with Section 6.14.2 of the Technical Specifications.

3.8 New Land Use Census Location(s) Identification

No location(s) have been identified by the Land Use Census that yields a calculated dose or dose commitment greater than the values currently being calculated in accordance with Section 4.11.2.3 of the Technical Specification. This notification is submitted to comply with Section 3.12.2.a of the Technical Specification.

GASEOUS EFFLUENTS -- SUMMATION OF ALL RELEASES

| | | UNIT | QUARTER 1 | QUARTER 2 | EST. TOTAL ERROR, % |
|----|--|---------|--------------|-----------|------------------------|
| F | ISSION AND ACTIVATION | GASES | | | |
| 1. | TOTAL RELEASE | CI | 0.000E+00 | 0.971E+02 | 0.250E+02 |
| 2. | AVERAGE RELEASE RATE FOR PERIOD | UCI/SEC | 0.000E+00 | 0.124E+02 | |
| 3. | PERCENT OF TECHNICAL SPECIFICATION LIMIT | 2 | 0.000E+00 | 0.458E-02 | |
| I | ODINES | | | | |
| 1. | TOTAL IODINE-131 | CI | 0.000E+00 | 0.000E+00 | 0.250E+02 |
| 2. | AVERAGE RELEASE RATE FOR PERIOD | UCI/SEC | 0.000E+00 | 0.000E+00 | 1.230270 |
| 3. | PERCENT OF TECHNICAL SPECIFICATION LIMIT | I | 0.000E+00 | 0.000E+00 | |
| P | PARTICULATES | | | | |
| 1. | PARTICULATES WITH HALF-LIVES >8 DAYS | CI | 0.000E+00 | 0.892E-01 | 0.250E+02 |
| 2. | AVERAGE RELEASE RATE FOR PERIOD | UCI/SEC | 0.629E+00 | 0.113E-01 | L AVE |
| 3. | PERCENT OF TECHNICAL SPECIFICATION LIMIT | X | 0.000E+06 | 0.125E-01 | |
| 4. | GROSS ALPHA RADIOACTIVITY | CI | 0.000E+00 | 0.658E-01 | |
| T | RITIUM | | | | |
| 1. | TOTAL RELEASE | CI | 0.000E+00 | 0.163E+00 | 0.250E+02 |
| 2. | AVERAGE RELEASE RATE FOR PERIOD | UCI/SEC | 0.000E+00 | 0.207E-01 | 0.2308+02 |
| 3. | PERCENT OF TECHNICAL SPECIFICATION LIMIT | * | 0.000E+00 | 0.115E-04 | |

GASEOUS EFFLUENTS - GROUND LEVEL RELEASE

RELEASE POINT: 1 (UNIT VENT)

(ABOVE)

CONTINUOUS MODE

BATCH MODE

| NUCLIDES RELEASED | UNITS | QUARTER 1 | QUARTER 2 | QUARTER 1 | QUARTEF 2 |
|----------------------|-------|-----------|-----------|-----------|-----------|
| . FISSION G | ASES | | | | |
| £°41 | CI | 0.000E+00 | 0.873E+02 | 0.000E+00 | 0.93 E+01 |
| KI193M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR85M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0./00E+00 |
| KR85 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0 000E+00 |
| KR87 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR88 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR89 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 3.000E+00 |
| KR90 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| AE131M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.0005+00 |
| XE133M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+30 |
| Umino | 0.1 | 0 0000 00 | | | |

0.873E+02

0.000E+00

0.986E+01

0.000E+00

GASEOUS EFFLUENTS - GROUND LEVEL RELEASE

RELEASE POINT: 1 (UNIT VENT)

CONTINUOUS MODE BATCH MODE

| NUCLIDES RELEASED | UNITS | QUARTER 1 | QUARTER 2 | QUARTER 1 | QUARTER 2 |
|--------------------------|-------|-----------|-----------|-----------|--------------|
| . IODINES | | | | | |
| I131 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| I133 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| I135 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

3. PARTICULATES

| C14 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
|--------------------------------|----|-----------|-----------|-----------|-----------|
| CR51 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| MN54 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+0 |
| FE59 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+0 |
| C058 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| C060 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ZN65 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR89 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR90 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ZR95 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SB124 | CI | 9.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS134 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS136 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS137 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| BA140 | CI | 0.000E+00 | C 000E+00 | 0.000E+00 | 0.000E+00 |
| CE141 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CE144 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| UNIDENT | CI | 0.000E+00 | 0.336E-01 | 0.000E+00 | 0.557E-01 |
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.336E-01 | 0.000E+00 | 0.557E-01 |

GASEOUS EFFLUENTS - GROUND LEVEL RELEASE

RELEASE POINT: 2 (MAIN STEAM LINE)

CONTINUOUS MODE BATCH MODE

| NUCLIDES | UNITS | QUARTER | QUARTER | QUARTER | QUARTER |
|----------|-------|---------|---------|---------|---------|
| KELEASED | | 1 1 | 2 | 1 | 2 |

1. FISSION GASES

| AR41 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
|--------------------------------|----|-----------|-----------|-----------|-----------|
| KR83M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR85M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR85 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | C.000E+00 |
| KR87 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | U.000E+00 |
| KR88 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR89 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR90 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE131M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE133M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE133 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE135M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XW135 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE137 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE138 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| UNIDENT. | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

GASEOUS EFFLUENTS - GROUND LEVEL RELEASE

UNITS

RELEASE POINT: 2 (MAIN STEAM LINE)

CONTINUOUS MODE BATCH MODE

QUARTER QUARTER QUARTER QUARTER

| RELEASED | | 1 | 2 | 1 | 2 |
|--------------------------------|----|------------|-----------|-----------|-----------|
| . IODINES | | | | | |
| I131 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| I133 | CI | 0.00000000 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| 1135 | CI | 0.000_+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

3. PARTICULATES

NUCLIDES

| C14 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
|--------------------------------|----|------------|-----------|-----------|-----------|
| CR51 | CI | 0.0001 +00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| MN54 | CI | 0.000E+00 | 0.000E+00 | 0.000E+30 | 0.000E+00 |
| FE59 | CI | 0.000E+00 | 0.000F+00 | 0.000E+00 | 0.000E+00 |
| C058 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| C060 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ZN65 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR89 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR90 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ZR95 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SB124 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS134 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS136 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS137 | CI | 0 000E+00 | 0.000E+00 | C.000E+00 | 0.000E+00 |
| BA140 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CE141 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CE144 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| UNIDENT | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | G.000E+00 |
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

GASEOUS EFFLUENTS - GROUND LEVEL RELEASE

RELEASE POINT: 3 (CONDENSER AIR REMOVAL)

CONTINUOUS MODE

BATCH MODE

| NUCLIDES | UNITS | QUARTER | QUARTER | QUARTER | QUARTER |
|----------|-------|---------|---------|---------|---------|
| RELEASED | | 1 | 2 | 1 | 2 |

1. FISSION GASES

| AR41 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
|--------------------------------|----|-----------|-----------|-----------|-----------|
| KR83M | CI | 0.000E+00 | 0.000E- 0 | 0.000E+00 | 0.000E+00 |
| KR85M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR85 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR87 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR88 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR89 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| KR90 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE131M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE133M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE133 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE135M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XW135 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE137 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| XE138 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| UNIDENT. | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

GASEOUS EFFLUENTS - GROUND LEVEL RELEASE

RELEASE POINT: 3 (CONDENSER AIR REMOVAL)

CONTINUOUS MODE BATCH MODE

| NUCLIDES | UNITS | QUARTER | QUARTER | QUARTER | QUARTER |
|----------|-------|---------|---------|---------|---------|
| RELEASED | | 1 | 2 | 1 | 2 |

2. IODINES

| I131 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
|--------------------------------|----|-----------|-----------|-----------|-----------|
| I133 | CI | G.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| I135 | CI | 0.000E+00 | 0.000E+00 | 0.000E+62 | 0.000E+00 |
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

3. PARTICULATES

| C14 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
|--------------------------------|----|-----------|-----------|-----------|-----------|
| CR51 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| MN54 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| FE59 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| C058 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| C060 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ZN65 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR89 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR90 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ZR95 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SB124 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS134 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS136 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS137 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| BA140 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CE141 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CE144 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| UNIDENT | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

LIQUID EFFLUENTS -- SUMMATION OF ALL RELEASES

| UNITS | QUARTER | QUARTER | EST. TOTAL |
|-------|---------|---------|------------|
| | 1 | 2 | ERROR. % |

A. FISSION AND ACTIVATION PRODUCTS

| 1. | TOTAL RELEASE (EXCL. TRIT., GASES, ALPHA) | CI | 0.000E+00 | 0.122E+00 | 0.500E+01 |
|----|--|--------|-----------|-----------|-----------|
| 2. | AVERAGE DILUTED CONC. DURING PERIOD | UCI/ML | 0.000E+00 | 0.122E-07 | |
| 3. | PERCENT OF APPLICABLE LIMIT | X | 0.000E+00 | 0.609E+00 | |

B. TRITIUM

| 1. | TOTAL RELEASE | CI | 0.000E+00 | 0.117+02 | 0.500E+01 |
|----|--|---------|-----------|-----------|-----------|
| 2. | AVERAGE DILUTED CONC. DURING PERIOD | UCI/SEC | 0.000E+00 | 0.117E-05 | |
| 3. | PERCENT OF APPLICABLE LIMIT | X | 0.000E+00 | 0.389E-01 | |

C. DISSOLVED AND ENTRAINED GASES

| 1. | TOTAL RELEASE | CI | 0.000E+00 | 0.857E-03 | 0.500E+01 |
|----|--|---------|-----------|-----------|-----------|
| 2. | AVERAGE DILUTED CONC. DURING PERIOD | UCI/SEC | 0.000E+00 | 0.857E-10 | |
| 3. | PERCENT OF APPLICABLE LIMIT | ž. | 0.000E+00 | 0.429E-04 | 1 + 1 |

D. GROSS ALPHA RADIOACTIVITY

| 1. | TOTAL RELEASE | CI | 0.000E+00 | 0.000E+C0 | 0.500E+01 |
|----|---|--------|-----------|-----------|-----------|
| E. | VOLUME WASTE RELEASED (PRIOR TO DILUTION) | LITERS | 0.163E+07 | 0.370E+07 | 0.800E-01 |
| F. | VOLUME DILUTION WATER USED DURING PERIOD | Liters | 0.100E+11 | 0.100E+11 | 0.100E+02 |

I IQUID EFFLUENTS FOR RELEASE POINT: 2 (LIQUID WASTE)

| NUCLIDES RELEASED | UNITS | QUARTER 1 | QUARTER 2 | QUARTER 1 | QUARTER 2 |
|----------------------|-------|-------------|-----------|--|-----------|
| | | | | | |
| H3 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.117E+02 |
| C14 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| NA24 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.575E-05 |
| P32 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CR51 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.175E-01 |
| MN54 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.245E-02 |
| MN56 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| FE55 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| FE59 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.811E-03 |
| C058 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.918E-01 |
| C060 | 10 | 0.000E+C0 | 0.000E+00 | 0.000E+00 | 0.200E-02 |
| NI63 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| NI65 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | |
| CU64 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ZN65 | CI | 0.000E+00 | 0.000E+00 | THE RESERVE AND PERSONS NAMED IN COLUMN 2 IS NOT THE OWNER, THE OW | 0.000E+00 |
| | | 1 0.000E+00 | 0.000.0 | 0.000E+00 | 0.000E+00 |
| ZN69 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.0000 |
| BR83 | CI | 0.000E+00 | 0.000E+00 | THE RESERVE AND ADDRESS OF THE PARTY OF THE | 0.000E+00 |
| BR84 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| BR85 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| | | 0.00081700 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

LIQUID EFFLUENTS FOR RELEASE POINT: 2 (LIQUID WASTE)

| NUCLIDES RELEASED | UNITS | QUARTER 1 | QUARTER 2 | QUARTER 1 | QUARTER 2 |
|----------------------|-------|-----------|-----------|--------------|-----------|
| | | | | | |
| RB86 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| RB88 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| RB89 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR89 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR90 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| SR91 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| Y90 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| Y91M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| Y91 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| Y92 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| Y93 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ZR95 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.353E-02 |
| ZR97 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| NB95 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.333E-02 |
| M099 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TC99M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.194E-04 |
| TC101 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

LIQUID EFFLUENTS FOR RELEASE POINT: 2 (LIQUID WASTE)

| NUCLIDES RELEASED | UNITS | QUARTER 1 | QUARTER 2 | QUARTER 1 | QUARTER 2 |
|----------------------|-------|-----------|-----------|--------------|-----------|
| | | | | | |
| RU103 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| RU105 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| RU106 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| AG110M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TE125M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TE127M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TE127 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TE.29M | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TE129 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TE131M | CI | 0.000E+00 | 0.000E+00 | 0.000E >00 | 0.000E+00 |
| TE131 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| TE132 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| 1130 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| 1131 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.696E-04 |
| 1132 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| 1133 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.214E-04 |
| 1134 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| I135 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

LIQUID EFFLUENTS FOR RELEASE POINT: 2 (LIQUID WASTE)

| NUCLIDES RELEASED | UNITS | QUARTER 1 | QUARTER 2 | QUARTER 1 | QUARTER 2 |
|----------------------|-------|--------------|-----------|-----------|-----------|
| QUID EFFLUE | NTS | | | | |
| CS134 | CI | 0.300E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS136 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS137 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CS138 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| BA139 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| BA140 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.0000.00 |
| BA141 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| BA142 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| LA140 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| LA142 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CE141 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CE143 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| CE144 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| PR143 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| PR144 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| ND147 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| W187 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |
| NP239 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

LIQUID EFFLUENTS FOR RELEASE POINT: 2 (LIQUID WASTE)

CONTINUOUS MODE BATCH MODE

| NUCLIDES | UNITS | QUARTER | QUARTER | QUARTER | QUARTER |
|----------|-------|---------|---------|---------|---------|
| RELEASED | | 1 | 2 | 1 | 2 |

LIQUID EFFLUENTS

| .LIQ (unident.) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.236E-03 |
|--------------------------------|----|-----------|-----------|-----------|-----------|
| TOTAL FOR PERIOD (ABOVE) | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.118E+02 |
| XE-133 | CI | 0.000E+00 | 0.100E-25 | 0.000E+0G | 0.857E-03 |
| XE-135 | CI | 0.000E+00 | 0.000E+00 | 0.000E+00 | 0.000E+00 |

EFFLUENT AND WASTE DISPOSAL REPORT FROM 3/8/88 0:00 TO 6/30/88 23:00 SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

No solid waste or irradiated fuel shipments were made during this reporting period.

A. SOLID WASTE SHIPPED OFF SITE FOR BURIAL OR DISPOSAL (NOT IRRADIATED FUEL)

| 1. | TYPE OF WASTE | UNIT | 6 MONTH PERIOD | EST. TOTAL ERROR % |
|----|--|----------------------|-------------------|-----------------------|
| Α. | SPENT RESINS, FILTER SLUDGES, EVAPORATOR BOTTOMS, ETC. | m ³ | 0 0 | N/A |
| В. | DRY COMPRESSIBLE WASTE, CONTAMINATED EQUIP., ETC. | M ³ | 0 0 | N/A |
| C. | IRRADIATED COMPONENTS, CONTROL RODS, ETC. | M ³ | 0 | N/A |
| D. | OTHER | M ³ CI | 0 | N/A |

2. ESTIMATE OF MAJOR NUCLIDE COMPOSITION (BY TYPE OF WASTE)

| A. 1 | | |
|------|-----|--|
| 2 | 1 | |
| 3 | ž l | |
| 4 | Ŷ | |
| 5 | 7 | |
| 6 | 7 | |
| B. 1 | Ŷ | |
| 2 | 7 | |
| 3 | Ŷ | |
| 4 | : | |
| 5 | : | |
| 6 | : | |
| C. 1 | : | |
| 2 | : | |
| 3 | | |
| | : | |
| | 2 | |
| 6 | | |
| D 1 | | |
| D. 1 | 7 | |
| 2 | X. | |
| 3 | X | |
| 4 | χ. | |
| 5 | X . | |
| 6 | 2 | |

EFFLUENT AND WASTE DISPOSAL REPORT FROM 3/8/88 0:00 TO 6/30/88 23:00 SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

3. SOLID WASTE DISPOSITION (NOT IRRADIATED FUEL)

| NUMBER OF SHIPMENTS | MODE OF TRANSPORTATION | DESTINATION |
|---------------------|------------------------|-------------|
| 0 | SHIP | N/A |
| 0 | AIR | |

- 4. CLASS OF SOLID WASTE N/A
- 5. TYPE OF CONTAINERS USED FOR SHIPMENT N/A
- 6. SOLIDIFICATION AGENT N/A
- B. IRRADIATED FUEL SHIPMENTS (DISPOSITION)

| NUMBER | OF | SHIPMENTS | MODE | OF | TRANSPORTATION | DESTINATION |
|--------|----|-----------|------|----|----------------|-------------|
| | 0 | | SHIP | | | N/A |
| | 0 | | AIR | | | |

APPENDIX A

Optional Use of Filter Elements in Influent to Waste Monitor Tanks

- a. Attachment 1 A summary of the evaluation that led to the determination that the change could be made in accordance with 10CFR50.59.
- b. Attachment 2 A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems.
- c. Attachment 3 Documentation of the fact that the change was reviewed and approved by the Plant Operations Review Committee (PORC).
- d. Attachment 4 Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information.
- e. Attachment 5 An estimate of the exposure to plant personnel as a result of the change.
- f. Attachment 6 An evaluation of the change, which shows the predicted release of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments.
- g. Attachment 7 An evaluation of the change, which shows the expected maximum exposure to a member of the public in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments.
- h. Attachment 8 A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and solid waste, to the actual releases for the period prior to when the changes are to be made.

APPENDIX A, ATTACHMENT 1

Optional Use of Filter Elements in Influent to Waste Monitor Tanks

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION 10CFR50.59 DETERMINATION REVIEW FORM

| THE LWPS FILTERS | |
|--|---|
| Check one : Procedure (Plant Modification Check one) Quality Related Nonquality Re | |
| Service of Goodly Related Syl tongoally Re | rateo |
| Does the subject of this review involve a change t | to the facility as described in the FSAR? |
| Sellan | 2/19/88 WYes 11 |
| BEC Cognizant Engr Signature | Oate |
| Does the subject of this review involve a change to | the procedures as described in the FSAR? |
| - solo ouras | 22 Feb 1988 WYes |
| Does the subject of this review involve a change to | Oate |
| Does the subject of this review conduct tests and o | experiments not described in the FSAR? |
| - Sllen | 2/19/88 1 1 Yes W |
| BEC Cognizant Engr Signature | Oate |
| Coes the publicit of this region require a change to | the Technical Specifications? |
| Jan Louida | 22 Feb 1988 Yes IV |
| | |

. USTIT CATION TO PICKOSEO CHANGE
THIS CHANGE IS REQUIRED TO PROVIDE FLEXIBILITY IN THE OPERATION
OF THE LIQUID WASTE PROCESSING SYSTEM, THEREBY IMPROVING SYSTEM
PERFORMANCE. FSAR SECTION 11.2 IS AFFECTED. SEE FSAR CHANGE
REQUEST NO. 1226.

HLAP 5689 12-851

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION INTERDEPARTMENTAL PROCEDURES

10CFR50.59 EVALUATIONS

| IP-3.20 | × | NO. |
|-------------------|--------|-----|
| PAGE 36 | OF | 41 |
| EFFECTIVE DATE | 11-20- | -87 |

ATTACHMENT IP-3.200-3

10CFR50.59 EVALUATION FORM - TYPICAL (Page 1 of 2)

| Unit # | | |
|--|-----------------------|----------|
| PROCEDURE PLANT MODIFICATION | | |
| ORIGINATING DOCUMENT NO. FCR 16. HBM \$1541 REV | .NO. | 0 |
| TITLE LWPS FILTERS | | |
| TITLE LOTS FILIERS | | |
| | | |
| DESCRIPTION Add notes to PSID TR349F9\$98 #1 & Vend | or Do | cun |
| 4652-01001-ATY & 5EHD 16702-195EC48 Rev. C. "7 | 70 1 | isto |
| | | |
| of filter elements inside the filter housings of the i | FDI, | 47 |
| & CPRWCT filters is optional." | | |
| ectional fillers is optional. | | |
| e critaci fillers is optional. | Yes | No |
| Does the subject of this review involve a change to the facility as | Yes | |
| | Yes Ø | ¥0 _ |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as | Ø | 1_ |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? | | |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as described in the safety analysis report? | Ø Ø | 1 |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as described in the safety analysis report? Does the subject of this review propose the conduct of tests or | Ø | 1 |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as described in the safety analysis report? Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? | × × | 1-12 |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as described in the safety analysis report? Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? Does the subject of this review require a change to the Plant | Ø Ø | 12 |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as described in the safety analysis report? Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? Does the subject of this review require a change to the Plant Technical Specifications? | × × | 12 |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as described in the safety analysis report? Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? Does the subject of this review require a change to the Plant | N N I I I | I N |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as described in the safety analysis report? Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? Does the subject of this review require a change to the Plant Technical Specifications? Does the proposed change, although not described in the safety | × × | I Z |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? Does the subject of this review involve a change to the procedures as described in the safety analysis report? Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? Does the subject of this review require a change to the Plant Rechnical Specifications? Does the proposed change, although not described in the safety analysis report, affect items or activities that are described | Ø | |

HLAP 168912-851

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION INTERDEPARTMENTAL PROCEDURES

10CFR50.59 EVALUATIONS

| NUMBER IP-3.20 | NO. | |
|-------------------|--------|----|
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| DATE 11 | -20-87 | |

ATTACHMENT IP-3.200-3

10CFR50.59 EVALUATION FORM - TYPICAL (Page 2 of 2)

If any answer is affirmative, perform an Unreviewed Safety Question evaluation.

If all answers are negative, no Unreviewed Safety Question evaluation is required.

Documentation of this review must be retained with the review package for the duration of the station license.

Note: "Safety analysis report" includes the FSAR, safety analyses submitted to the NRC in support of their review of the application for an operating license and subsequent amendments to the operating license and other license commitments made to the NRC.

Prepared by:

| Mile | Description | 22 Feb 1988 |
| Concurrence: | Mile Description | 22 Feb 1988 |
| Department Manager | Date

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION INTERDEPARTMENTAL PROCEDURES

10CFR50.59 EVALUATIONS

| IP-3.20 | Q REV. |
|---------|--------|
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| DATE 11 | -20-87 |

ATTACHMENT IP-3.200-4

UNREVIEWED SAFETY QUESTION EVALUATION FORM (Page 1 of 4)

| Unreviewed Safety Question Evaluation # 88-9029 Date Assigned 2/23/88 |
|--|
| Procedure Change Plant Modification Other BEC FSAR Change Reg. Leg Mo 1226 |
| Originating Document: FCR No. HBM-\$1541 Rev. # \$ |
| TITLE: LWPS FILTERS |
| DESCRIPTION: Add notes to PEID 7R3\$9F9000 #1 & Vendor Documents |
| 4\$52-\$1\$\$1-ATY & SEHO 1\$7\$2-195EC48 Rev C "The installatu |
| of filter clements inside the filter housings of the FOT, LHST |
| CPRINCT filters is optional." Delete FSAR references to FOT, |
| LHST & CPRWLT filters |
| A. 1. Does the subject of this evaluation increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis YES NO report? |
| Bases: There are no accidents evaluated in the |
| FSAR which require the Liquid Wast Processing System |
| FDT, LHST OF CPRWCT filters to be operable. |
| There fore this does not increase the probability of |
| occurance or the consequences of an accident or malfunction |
| of equipment important to safety previously evaluated in the |
| ESAR. |
| |
| (This form, when completed, shall be retained for the duration of the license.) |

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION INTERDEPARTMENTAL PROCEDURES

10CFR50.59 EVALUATIONS

| IP-3.20 | ×0 | REV. |
|-----------------------|-------|------|
| PAGE 39 OF | | 41 |
| EFFECTIVE DATE 11- | 20-87 | |

ATTACHMENT IP-3.200-4

Evaluation to. 88-4429

(Page 2 of 4)

2. Does the subject of this evaluation create the possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report?

I YES NO

do not impact the safe operability of the reactor nor does it provide a boundry to prevent an uncontrolled release of radioactivity to the environment. Therefore these items do not create the possibility for an accident of a different type than any previously evaluated in the FSAR.

3. Does the subject of this evaluation reduce the margin of safety as defined in the basis for any technical specification?

I YES X NO

masses: The LUPS FDT, LHST or CPRINCT Filters do not impact the safe operability of the reactor nor do they provide a boundry to prevent uncontrolled release of radioactivity to the environment. Procedures require that effluents from the FDT, LHST and CPRINCT be routed to a monitored tank and surveillance performed prior to release to the environment. Therefore this item does not reduce the margin of safety as defined in the basis for any Technical Specification

Note: "Safety analysis report" includes the FSAR, safety analyses submitted to the NRC in support of their review of the application for an operating license and subsequent amendments to the operating license, and other license commitments made to the NRC.

HLAP 668912-851

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION INTERDEPARTMENTAL PROCEDURES

10CFR50.59 EVALUATIONS

| IP-3.20Q | | NO. |
|------------|-------|-----|
| PAGE 40 OF | | 41 |
| DATE 11- | 20-87 | |

ATTACHMENT IP-3.200-4

Evaluation so. 88-4429

UNREVIEWED SAFETY QUESTION EVALUATION FORM (Page 3 of 4)

| 1. | All of the above questions were as originating document does not invoquestion. | |
|--------------|---|--|
| 2. | One or more of the above questions the originating document involves question. The originating document be implemented without prior appropriate output of the disposition of question below. | an unreviewed safety nt, as presented, shall No oval of the NRC. Provide |
| RECOMMEND | ED DISPOSITION: | |
| | | |
| | | |
| | 10.1 A AAA A | |
| PREPARED BY: | originator | 1 22 Feb 1988 DATE |
| REVIEWED BY: | mu Burne for Ac. | - , 23 Feb 1988 |
| APPROVED BY: | Warrent King |) 2-23-88 DATE |
| | | |
| AKKS: | | |
| AKKS: | | |
| (ARKS: | | |

Optional Use of Filter Elements in Influent to Waste Monitor Tanks

The Liquid Waste Processing System collects and processes radioactive liquid waste generated during plant operation and reduces radioactivity and chemical content to acceptable levels for discharge or recycling.

The Liquid Waste Processing System consists of the Waste Holdup Tank, which collects radioactive liquid from the Reactor Coolant Drain Tank and the Reactor Containment Building sumps, the Floor Drain Tank, which collects liquids from Floor drains in the Mechanical Auxiliary Building, and the Condensate Polishing Regenerative Waste Collection Tank, which collects radioactive laboratory drains, and serves as an alternate tank for floor drains. Also included in this system is the Laundry and Hot Shower Tank, which collects water from the hot shower facility.

These tanks are all equipped with eductors to ensure proper mixing prior to transfer. The liquid collected in these tanks may be processed in accordance with the Process Control Program. This program selects the optimum processing mode, filtration, demineralization, evaporation, or a combination of the three. After processing by one of these methods, the liquid is then collected in one of six Waste Monitor Tanks and analyzed prior to discharge.

The Liquid Waste Processing System waste evaporator also has the capability to produce reactor grade distillate which is collected in the Waste Evaporator Condensate Tank. This distillate may also be processed as radioactive waste and discharged. Concentrates from the waste evaporator are processed in the Solid Waste Processing System. Any gaseous effluents produced are processed by the Gaseous Waste Processing System.

The filters that were affected by this change were the Floor Drain Tank Filter, the Condensate Polishing Regenerative Waste Collection Tank Filter, and the Laundry and Hot Shower Tank filter. These filters are downstream of their respective tank pumps and allow most of the particulate to be removed prior to processing via demineralization or evaporation. Early in plant life, filtration is frequently not necessary, waste streams can be sent directly to the Waste Monitor Tanks prior to discharge, or processed via demineralization. The ability to process waste streams with or without filtration will allow a much greater flexibility in processing waste early in plant life.

Optional Use of Filter Elements in Influent to Waste Monitor Tanks

Plant Operations Review Committee

OPGP03-ZA-0004 Rev. 6 Page 15 of 15

PORC REVIEW COVER SHEET OPGP03-ZA-0004-1 (Page 1 of 1)

| The PORC ha | s reviewed this item and has determined that (check as |
|-------------|--|
| | does NOT involve an UNREVIEWED SAFETY QUESTION. |
| | |
| | does NOT deviate from the commitments made in the FSAR, SER, icensing documents. |
| It does | does NOT adversely impact plant nuclear safety. |
| t does | does NOT adversely impact the health and safety of plant |
| | |
| | does NOT require further review by the NSRB, the NRC, or other specify below. |
| LEMARKS | does NOT require further review by the NSRB, the NRC, or other specify below. |
| LEMARKS | other, specify below. |
| REMARKS | other, specify below. |

This form, when completed, shall be retained in accordance with the retention requirements of the originating document.

Optional Use of Filter Elements in Influent to Waste Monitor Tanks

The Liquid Waste Processing System collects radioactive waste from various sources and processes this waste for discharge or recycle. Several different modes of processing waste streams are available including filtration, demineralization, and waste evaporation, and combinations of these three methods. The optimum processing method is selected in accordance with the Process Control Program. This program chooses methods based on solid content and radiochemistry of the selected waste stream.

In many cases, the waste stream does not require any method of processing prior to discharge. Early in plant life, many of the waste streams have low radioactivity concentrations and may be sent directly to the Waste Monitor Tanks for discharge. In order to extend filter life and provide more flexibility in processing, it is now allowable to process with or without filtration based on the characteristics of the waste stream. Changes were made to the Safety Analysis Report and the applicable Piping and Instrumentation Diagrams in order to allow Chemical Operations the choice of whether or not to use filtration downstream of the Floor Drain Tank, Condensate Polishing Regenerative Waste Collection Tank, and the Laundry and Hot Shower Tank. These changes are attached along with an overall flow diagram of the Liquid Waste Processing System. Also attached is the Engineering Support Request response that explains Support Engineering reasoning for this change.

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION 10CFR50.59 DETERMINATION REVIEW FORM

| 10 100 120 | 6 Persion V: 0 | |
|---|--|------|
| Tile FSAR CHANGE RO | EQUEST NO. 1226 | |
| Check one : Procedure (V) Plant Modification | () Other | - |
| Check one () Quality Related (V) Nonquality Re | Plated | |
| Does the subject of this review involve a change t | o the facility as described in the FSAR? | |
| - Sthan | 2/19/81 WYes | 11 4 |
| BEC Cognizant Engr Signature | Date | |
| | 22 to 1988 VIVes Date | |
| · · | | |
| Does the subject of this review conduct tests and o | experiments not described in the FSAR? | |
| Does the subject of this review conduct tests and o | | 1 40 |
| | 2/19/88 1 1 Yes Date | y 40 |
| Wilher_ | Date Date Technical Specifications? | 1 40 |
| BEC Cognizant Engr Signature | 2/19/88 1 1 Yes | 1 40 |

JHIS EMANGE IS REQUIRED TO PROVIDE FLEXIBILITY IN THE OPERATION OF THE LIQUID WASTE PROCESSING SYSTEM, THEREBY IMPROVING SYSTEM PERFORMANCE. FSAR SECTION 11.2 IS AFFECTED



FSAR CHANGE REQUEST

| 1. DISCIPLINE | MECHANICAL | 2. LOG NO | 1226 |
|--|---|---|---|
| 3 ORIGINATOR | STAN CHAN | 4. DATE | 2/19/88 |
| S. THE LCTS HAS BEEN | REVIEWED | □ YES | BNO |
| 6. PRE-OPERATIONAL | TEST PROGRAM AFFECTED | ☐ YES | D NO |
| 7. SUBMITTED BY (GR | OUPEGS) (DATE) | A. GUIDRY | |
| 11. 2 9. DESCRIPTION OF CHARGE 11. MAB FLOCK 3. CONDENSATE THIS CHARGE 10. JUSTIFICATION BY MAKING TO | IN ALTP CHANGE NO. HANGE SECTION II. ON THE DISCHARGE ORAIG TANK, 2 L POLISIUM REGENERA WILL MAKE THE US. | TOUVERSEDES TICE NO. 1291. 2 OF THE FS. SIDE OF EACH OF AUNDRY AND HOL TICK WASTE COL E OF THESE FI FILTERS OPTICE E CPERATION OF | ALL CHANGES MADE AR STATES THAT THERE OF THE FOLLOWING THIRS: THOWER THIRK, LLCCTION THINK, LLCCTION THINK, LLCCTON THINGS THE LLCCON, WASTE |
| | | | |
| 11 DISCIPLINE REVIE | w | REVIEW DUE DA | ATE |
| 11. DISCIPLINE REVIE | w NA | | WH Extel |
| 11. DISCIPLINE REVIE STARTUP ARCH. | | MECH. X | |
| STARTUP | NA | MECH. X | 14 CESUL |
| ARCH | NA NA | MECH. X | 14 extil |
| ARCH | NA NA NA | MECH. X | 14 CESTILL |
| ARCH | NA NA NA NA | MECH. X NUC Z PLANT DES PSSG | 14 CESULE NA NA NA |

The bulk of the radioactive liquids discharged by the Reactor Coolant System (RCS) can be processed and recycled by the Boron Recycle System (BRS).

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Inputs into the LWFS from the BRS can be processed in the BRS prior to processing in the LWFS. The capability exists to route this waste to the condensate polishing regeneration waste collection tank (CPRWCT) or floor drain tank (FDT) because of either unacceptable water chemistry or for tritium control purposes. This waste is either off-quality refueling water or off-quality recycle holdup tank water. If the input is not processed in the BRS it will be processed in the LWPS and discharged.

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The LWPS is designed to monitor and release sufficient tritiated water to control tritium buildup in the plant. Since the normal mode of the LWPS is process and discharge, there will be no tritium buildup in the LWPS. The majority of other inputs to the LWPS are normally low activity; therefore, the bulk of the processing requirements for the LWPS are based on generally low-activity wastes.

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The capability for handling evaporator concentrates as well as handling and storing spent demineralizer resins is also provided in the system.

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The capability exists to process high and low total dissolved solids (TDS) wastes from condensate polisher regeneration. Both low and high TDS wastes can be discharged directly through the LWPS and the effluent radiation monitor to the CWS if processing is not required. High TDS wastes are routed to the CPRWCT and may be processed through the waste evaporator and monitor tanks prior to discharge to the CWS. Normally no LWPS processing is required. LWPS processing is only required in the case of primary to secondary leakage. Normal CPRW processing is in the neutralization basin.

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Instrumentation and controls necessary for the operation of the LWPS are located on a control board in the radwaste control room which is located within the Mechanical Auxiliary Building (MAB).

Component locations are shown on Figure 1.2-26, Sheets 3 and 4, and Figure 1.2-38. Radioactive equipment is generally isolated in individual cubicles.

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Process flow diagrams and piping and instrumentation diagrams (P&IDs) are shown on Figures 11.2-1 through 11.2-12.

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11.2.2.1.1 Process and Equipment Description: The LWPS is provided to collect and process BRS tritium control volume and non-reactor-grade liquid wastes. These include floor drains, equipment drains containing non-reactor-grade water, laundry and hot shower drains, contaminated wastes from the Condensate Polishing Regeneration System, and other non-reactor-grade sources. Equipment in the LWPS includes the reactor coolant drain tank (RCDT), the RCDT pumps and HX, the waste holdup tank (WHT), WHT filter, WHT pumps, WHT purification demineralizer and filter and two waste evaporator condensate tanks (WECT). A list of equipment is given in Table 11.2-3. The WHT can be processed through either the LWPS waste evaporator or the BRS recycle evaporator. The BRS recycle evaporator is used only in the event the LWPS waste evaporator is not operational. Additional equipment includes a FDT and filter, laundry and hot shower tank (LHST) and filter, CPRWCT and filter, waste evaporator, waste evaporator condensate (WEC) demineralizer and strainer, and six waste monitor tanks (WMT) and pumps.

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The WECTs provide extra storage capacity for the WMTs and can be discharged through the effluent radiation monitor to the CWS. Additional surge capacity is provided by two surge tanks located in the Fuel Handling Building (FHB). Processing consists of any of the following modes:

- Filtration, evaporation, and demineralization
- 2. Filtration and evaporation
- 3. Filtration and demineralization
- 4. Filtration

The processing mode is determined by the quality of the liquid to be processed. The normal processing mode is expected to be one of the following combinations: filtration, filtration and/or evaporation or filtration, evaporation and/or demineralization.

The waste evaporator is a forced circulation crystallizer with a 30-gal/min distillate processing flowrate. In addition this evaporator can also be used as a backup to the BRS recycle evaporator.

The LWPS waste evaporator is designed to operate on a batch basis and the contents are concentrated to approximately 12 we percent boric acid or 25 wt percent sodium sulphate.

The waste evaporator must be operated at a basic pH. Therefore the pH of the feed is adjusted as needed prior to transfer into the evaporator. Feed enters the discharge piping of the recirculation pump, is heated in the heating element and circulated through the vapor body. When the desired concentration is achieved, as indicated by the evaporator density instrumentation, the concentrate is pumped to the concentrate storage tank for eventual solidification, or directly to solidification bypassing the tanks.

Evaporator distillate is condensed and cooled in the distillate condenser and subcooler skid and further treated by the waste evaporator condensate demineralizer or WHT purification demineralizer before transfer to one of the waste monitor tanks.

The two largest sources of liquid volume to the LWPS are the Condensate Polishing System regeneration waste and the equipment floor drains. The equipment floor drains represents a relatively continuous source to the LWPS whereas the condensate polishing system regeneration waste is an infrequent high-volume, short-duration, batch source. The Condensate Polishing System regeneration waste is a source to the LWPS only during periods of off normal operations when there is above normal primary-to-secondary leakage, or excessive fuel defects. At these times the regeneration waste is directed to the CPRWCT as discussed in Section 10.4.6.2; during these periods the equipment and floor drains are directed to the FDT. Processing of this waste in the CPRWCT is identical to the processing specified for waste in the FDT. The system also provides the capability to discharge the condensate polishing regeneration waste directly to the CWS if processing is not required.

Laundry and hot shower drains are normally filtered and released. However, the waste evaporator is capable of processing the laundry and hot shower waste after addition of an antifoaming agent, if treatment of these wastes is required prior to their release.

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The RCDT collects deaerated, tritiated leakoffs from inside the Containment and the contents are transferred by RCDT pumps through the RCDT heat exchanger (for cooling to less than 130°F) to the WHT. For the normal operating mode, a level band is maintained automatically in the RCDT and the system requires no operator action. In addition, the RCDT contents can be transferred directly to the BRS for processing

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11.2.2.3.1.2 Processing Operation - Inputs to the LWPS are segregated according to their chemical makeup: detergent wastes are routed to the LHST, Condensate Polishing System contaminated regeneration wastes are routed to the CPRWCT, and sumps and gravity drains are routed to the FDT.

Three parallel modes of operation are available in the LWPS:

High and low TDS waste streams from the Condensate Polishing System demineralizer regeneration are collected in the CPRWCT or FDT. The regenerants are processed through the CPRWCT or FDT filter, waste evaporator, and/or demineralizers, filters and sent to the WECTs or WMTs. Discharge is to the CWS; the effluent radiation monitor is used to verify acceptable activity levels. If the activity exceeds the setpoint, the flow is diverted back to the discharging tank for re-processing. During operation with no primary-to-secondary leakage, CPRW regenerants can be discharged directly to the CWS using the LWPS discharge header and effluent radiation monitor. The normal flow path for CPRW is to the netralization basin. (See Section 10.4.6.)

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2. MAB floor drains and contents of the RCDT may be processed through the WHT and filter. Capability exists to process the low TDS waste and Mill floor drains in parallel. MAB floor drains are sent to the FDT and if NECESSAR processed through the auxiliary demineralizer, the WECD, and the filter.

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3. The contents of the LHST are normally transferred to a waste monitor tank (WMT) through the LHST filter after sampling. If sampling results show high radioactivity concentrations, the LHST contents can be processed through the waste evaporator after addition of an antifoaming agent. The contents of the FDT and CPRWCT can be transferred directly to the WMT, processed through the waste evaporator and the LWPS AD and WEC demineralizers and WEC filter. The distillate from the waste evaporator is routed to the WMTs or WECTs.

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The contents of the WMTs are discharged to the CWS after sampling. A discharge permit will be issued for each release. The water being discharged from the LWPS to the CWS is monitored for radioactivity level. In the event a higher radioactivity level than set is reached, the three-way valve in the discharge line will divert the flow back to the WMT being discharged. The contents of the WMTs can be recirculated through the auxiliary demineralizer and strainer if additional processing is required.

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TABLE 11.2.3 (Continued)

EQUIPMENT PRINCIPAL DESIGN PARAMETERS

Special features:

Cartridge replacement accomplished by remote handling system. Replacement criteria of either high differential

pressure or time in service.

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Materials:

.. .

Housing: SS

Filter element: EICF

Laundry and Hot Shower Tank Filter

Function: Removal of suspended solids from contents of LHST.

Number: 1

Type: Cartridge

Design pressure, psi: 150

Design temperature, 'F: 200

Design flow, gal/min: 150

-AP at design flow (clean) psi: 5-

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Retention: 98% retention for particles 5 misrons or larger-

Cartridge replacement accomplished by remote handling Special features:

system. Replacement criteria of either high differential

pressure or time in service.

SPECIAL NOTE: THE FILTER CARTRIDGE MAY BE REMOVED FROM THE FILTER HOUSING IF FILTRATION IS NOT REQUIRED FOR THE PRIPER

HOUSING SE PRESSING OF TANK CONTENTS

- Filter element: EICF

Floor Drain Tank Filter

Function: Removal of suspended solids from contents of FDT.

Number: 1

Type: Cartridge

Design pressure, psig: 150

Design temperature, 'F: 200

Design flow, gal/min: 150

TABLE 11.2-3 (Continued)

EQUIPMENT PRINCIPAL DESIGN PARAMETERS

AP at design flow (clean), psi: 5

Rotontion: 98% rotontion for particles 5 microns or larger

Special features: Cartridge replacement accomplished by remote handling

system. Replacement criteria of either high differential

pressure or time in service.

- Materials:

2 4x à

SPECIAL NOIE: THE FILTER CARTRIDGE MAY BE REMOVED

FROM THE FILTER HOUSING IF , ILTRATION IS NOT -HOUSING SS REQUIRED FOR THE PROPER PROCESSING OF TANK

-Filtor element: EICF CONTENTS

Waste Holdup Tank Purification Demineralizer Filter

Function: Collection of resin fines from WHT purification demineralizer.

Number: 1

Type: Cartridge

Design pressure, psig: 150

Design temperature, 'F: 200

Design flow, gal/min: 150

ΔP at design flow (clean), psi: 5

Retention: 98% retention for particles 5 minrons or larger

Special features: Cartridge replacement accomplished by remote handling

system. Replacement criteria of either high differential

pressure time in service.

Materials:

Housing: SS

Filter elements: EICF

Condensate Polishing Regeneration Waste Collection Tank Filter

Function: Removal of suspended solids from contents of CPRWCT.

Number: 1

Type: Cartridge

Design pressure, psi: 150

STP FSAR

TABLE 11.2-3 (Continued)

EQUIPMENT PRINCIPAL DESIGN PARAMETERS

Design temperature, 'F: 200

Design flow, gal/min: 150

AP at design flow (clean), psi: 5

Retention: 98% retention for particles 5 microns or larger

Special Features: Cartridge replacement accomplished by remote handling

system. Replacement criteria of either high differential

pressure or time in service.

SPECIAL NOTE: THE FILTER CARTRIDGE MAY BE REMOVED FROM THE FILTER MATERIALS HOUSING IF FILTRATION IS NOT REQUIRED FOR THE PROPER

PROCESSING OF TANK CONTENTS.

Filter elements -- SIGF-

STRAINERS

Pump Suction Strainers

Function: Duplex basket strainer to trap only large particles that could

cause pump damage and excessive filter loading. Strainers are changed on high differential pressure and are not expected to contain significant concentrations of radioactivity. Are used on

FDT, CPRWCT and LHST pumps.

Number: 3

Type: Basket

Design temperature, 'F: 200

Design flow, gal/min: 100

AP at design flow (clean) psi: 0.5

Nominal rating, microns: 840 (40 mesh)

Materials: SS

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SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

ENGINEERING SUPPORT REQUEST

| 1. PAGE 2 | 2. DATE | 3. ESR NO. | 4 REV. NO. | |
|------------|---------|------------|------------|--|
| : <u>2</u> | 2/19/88 | 87-WL-030 | / | |

THE FOLLOWING PROVIDES REVISED ENGINEERING RESPONSE TO ESR 87-WL-030.

AS DISCUSSED DURING A MEETING BETWEEN HLEP (MILTON REJCEK, L. DUSEK, J. GABRIELSON, ETC.) AND BECHTEL

(A. GUIDRY S. GHAN) ON, 2/18/88, A DESIGN CHANGE WILL

BE MADE TO SELETE THE REQUIREMENT FOR FILTERS DOWNSTREAM OF THE FILOR DRAIN TANK, LAUNDRY AND HOT SHOWER

TANK AND CONDENSATE POLISHING REGENERATIONS WASTE COLLECTIONS

TANK. THE INSTALLATIONS OF FILTER ELEMENTS INSIDE THE EXISTING

HOUSINGS FOR THESE FILTERS WILL BE MADE OPTIONAL IN ORDER

TO PROVIDE FLEXIB LITY IN THE OPERATION OF THE LIQUID WASTE

PROCESSING SYSTED THIS DESIGN CHANGE WILL ALLOW PROCESSING

CF WASTE FROM THE ABOVE TANKS IN THE FOLLOWING MANNEY:

- 1. LIQUIDS WITH A LOW ENDUGH LEVEL OF RADIDACTIVITY WILL BE DISCHARGED DIRECTLY TO THE WASTE MONITOR TANKS WITHOUT FILTRATION OR OTHER PROCESSING.
- 2. CONTAMINATED LIQUIDS REQUIRING PROCESSING AND WHICH AGO
 CONTAIN A HIGH LEVEL OF SOLIDS CAN BE PROCESSED
 BY THE WASTE EVAPORATOR. NO FILTRATION AT THE ABOVE
 FILTERS WOULD BE REQUIRED.
- 3 OTHER LIQUIDS CAN BE PROCESSED BY DEMINERALIZATION WITH OR WITHOUT FILTRATION BEFORE HAND AS DEEMED APPROPRIATE BY CHEM OPS

THE ADDITION OF SCREENS OR BAG FILTERS AT FLOOR DRAINS WILL REWIRE AN - EVALUATION OF FLOODING IMPACT OR TEST TO DETERMINE FLOOR DRAIN BLOCKAGE. THEY ARE NOT EXPECTED TO ALLEVIATE THE FOT FILTER PROBLEM UNLESS VERY SMALL PORE SIZE IS USED.

Sellen 2/22/88

Optional Use of Filter Elements in Influent to Waste Monitor Tanks

This change allows use of the Liquid Waste Processing System with or without the Floor Drain Tank Filter, the Laundry and Hot Shower Tank Filter, and the Condensate Polishing Regenerative Waste Collection Tank Filter. This simply allows flexibility in modes of processing waste streams and does not change the radiochemistry of these waste streams nor does it change the configuration of any plant equipment. Exposure to plant personnel should decrease as a result of this change, due to fewer filter changeouts required.

Optional Use of Filter Elements in Influent to Waste Monitor Tanks

This change allows flexibility in modes of processing radioactive waste streams, it does not change the amount of chemistry of liquid or gaseous waste streams in any way. Fewer filter changeouts will be required in the plant during periods where these three filters are not used in the plant, resulting in less solid waste. However, this reduction of solid waste will be balanced by the increased change of more resin replacements when high solid waste streams are seen by the demineralizers.

Optional Use r . Filter Elements in Influent to Waste Monitor Tanks

This notification simply allows flexibility in processing waste streams generated by the plant. It does not affect the amount or radiochemistry of these waste streams. This modification will not result in a change to the expected maximum exposures to a member of the public in the UNRESTRICTED AREA and to the general population as previously estimated in the License application and amendments.

Optional Use of Filter Elements in Influent to Waste Monitor Tanks

This item is not applicable. This is the initial reporting period for the South Texas Project Electric Generating Station.

APPENDIX B

Increase in Waste Evaporator Concentrates Operational Levels

- a. Attachment 1 A summary of the evaluation that led to the determination that the change could be made in accordance with 10CFR50.59.
- b. Attachment 2 A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems.
- c. Attachment 3 Documentation of the fact that the change was reviewed and approved by the Plant Operations Review Committee (PORC).
- d. Attachment 4 Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information.
- e. Attachment 5 An estimate of the exposure to plant personnel as a result of the change.
- f. Attachment 6 An evaluation of the change, which shows the predicted release of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments.
- g. Attachment 7 An evaluation of the change, which shows the expected maximum exposure to a member of the public in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments.
- h. Attachment 8 A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and solid waste, to the actual release for the period prior to when the changes are to be made.

Increase in Waste Evaporator Concentrates Operational Levels

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION INTERDEPARTMENTAL PROCEDURES 10CFR50.59 EVALUATIONS PAGE 36 OF 41 EFFECTIVE DATE 11-20-87

ATTACHMENT IP-3.200-3

ATTACHMENT 1

10CFR50.59 EVALUATION FORM (Page 1 of 2)

| DESCRIPTION | | |
|--|--------------|------------|
| Unit # | | |
| PROCEDURE FLANT MODIFICATION CTMER. | | |
| ORIGINATING DOCUMENT NO. CHANGE #1355 | .xo. | N/A |
| TITLE SAR 11.2.2.1.1, 11.4.1.2 | | |
| DESCRIPTION CHANGE "APPROXIMATELY 12 WT. PERCENT BORK ACID OR 25 WT. PERCENT SULFATE" TO "OPTIMUM OFERATIONAL LEVELS AS SPECIFIED IN THE PROCESS CONTRO | | |
| DN PG 11.1-3 OF THE SAR, AND CHANGE "UP TO 12 WT. PERCENT BORK ACID OR 25 W | | |
| SOUM SULFATE TO "OPTIMUM OPERATIONAL LEVELS AS SPECIFIED IN THE PROCESS CO ON PG. 11.4-2 OF THE SAR. | | |
| Does the subject of this review involve a change to the facility as described in the safety analysis report? | <u>Yes</u> . | <u>X</u> 0 |
| Does the subject of this review involve a change to the procedures as described in the safety analysis report? | 4 | |
| Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? | 1_1 | 4 |
| Does the subject of this review require a change to the Plant Technical Specifications? | 1_1 | À |
| Does the proposed change, although not described in the safety analysis report, affect items or activities that are described in the safety analysis report? | 121 | Z |
| (This form, when completed, shall be retained for the live of the min | | |

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION 'INTERDEPARTMENTAL PROCEDURES

10CFRSO.59 EVALUATIONS

| IP-3.20Q | | REV. |
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| FFECTIVE | 20-87 | |

ATTACHMENT IP-3.200-3

10CFRSD.59 EVALUATION FORM (Page 2 of 2)

If any answer is affirmative, perform an Unreviewed Safety Question evaluation.

If all answers are negative, no Unreviewed Safety Question evaluation is required.

Documentation of this review must be retained with the review package for the duration of the station license.

Note: "Safety analysis report" includes the FSAR, safety analyses submitted to the NRC in support of their review of the application for an operating license and subsequent amendments to the operating license and other license commitments made to the NRC.

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| Date |
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HL&P 6689(12-85)

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION INTERDEPARTMENTAL PROCEDURES

10CFRSO.59 EVALUATIONS

| NUMBER IP-3.200 | | REV. NO. |
|---------------------------|------|-------------|
| PAGE 38 | OF | 41 |
| EFFECTIVE DATE 11-2 | 0-87 | |

ATTACHMENT IP-3.200-4

UNREVIEWED SAFETY QUESTION EVALUATION FORM (Page 1 of 4)

| Unreviewed Safety Question Evaluation # |
|---|
| Procedure Change Plant Modification Other |
| Originating Document: |
| 200 |
| DESCRIPTION: CHANGE "APPROXIMATELY 12 NT. PERENT BORK ACID OR 25 WT. PERENT SOME SULFA TO "OPTIMUM CPERATIONAL LEVELS AS SPECIFIED IN THE PROCESS CONTROL PROGRAM" ON PG. 11.2-3 OF TH SAR, AND CHANGE "UP TO 12 WEIGHT PERCENT BORK ACID OR 25 WEIGHT PERCENT SODIUM SULFI TO "OPTIMUM OPERATIONAL LEVELS AS SPECIFIED IN THE PROCESS CONTROL PROGRAM" ON PG. 11.4-2 OF THE SAR. |
| A. 1. Does the subject of this evaluation increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis YES NO report? |
| Bases: JHK CHANGE ALLOWS BATCH CONCENTRATIONS OF UP TO 16% No. By 07.7420 |

Eases: THIS CHANGE ALLOWS BATCH CONCENTRATIONS OF UP TO 16% No. By C. 7H.O IN THE CONCENTRATES STREAM NHICH IS TRANSFERRED FROM THE WASTE TO CRATER, THROUGH THE WASTE EVAPORATOR BOTTOM DISCHARGE PIPING TO THE CONCENTRATES STORAGE TANK, PUMPED BY THE CONCENTRATES TRANSFER PUMPIA, THE CONTENTS OF THE CONCENTRATES STORAGE TANK ARE THEN POMPED OUT BY CONCENTRATES TRANSFER PUMPIB TO BE SOUDIFIED. THERE ARE NO ACCIDENTS EVALUATED IN THE SAR WHICH THIS CHANGE WOULD IMPACT, THUS, THIS CHANGE DOES NOT INCREASE THE PROBABILITY OF OCCURRENCE OR THE CONSEQUENCES OF AN ACCIDENT OR MALFUNCTION OF EQUIPMENT IMPORTANT TO SAFETY PREVIOUSLY EVALUATED IN THE SAR. (This form, when completed, shall be retained for the duration of the license.)

HL&P 6689(12-85)

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION INTERDEPARTMENTAL PROCEDURES

10CFR50.59 EVALUATIONS

| NUMBER | | REV. |
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| IP-3.20 | Q | 1 |
| PAGE 39 | OF | 41 |
| EFFECTIVE | 20-87 | |

ATTACHMENT IP-3.200-4

Evaluation No. 88-0055

UNREVIEWED SAFETY QUESTION EVALUATION FORM (Page 2 of 4)

2. Does the subject of this evaluation create the possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report?

I YES A NO

Na BUOY THO AND THE SUBSEQUENT TRANSFER OF THESE CONCENTRATES FROM THE INASTE EVAPORATION PROTECTION DISCHARGE PIPING, THROUGH CONCENTRATES TRANSFER PUMP IA TO THE CONCENTRATED STORAGE TANK, THROUGH CONCENTRATES TRANSFER PUMP IS TO BE SOLIDIFIED, DOES NOT AFFECT ANY SAFETY RELATED SYSTEM, NOR DOES IT CREATE THE POSSIBILITY FOR AN ACCIDENT OR MALFUNCTION OF A DIFFERENT TUPE THAN ANY EVALUATED PREVIOUSLY IN THE SAFETY ANALYSIS REPORT.

3. Does the subject of this evaluation reduce the margin of safety as defined in the basis for any technical specification?

I YES A NO

Bases: THIS CHANGE ALLOWS UP TO 1696 No. B.D. 7.14.0 IN THE CONCENTRATES

STREAM FROM THE WASTE EVAPORATER, THOUGH THE WASTE EVAPORATOR DISCHARGE PIPING

AND CONCENTRATES TRANSFER PUMP IA, TO THE CONCENTRATES STORAGE TANK, AND PUMPED

BUT OF THE CONCENTRATES STORAGE TANK BY CONCENTRATES TRANSFER PUMP IB TO BE

SOCIOIFIED. THIS CHANGE ALLOWS CHEMICAL OPERATIONS TO OBTAIN OPTIMUM OPERATIONAL

LEVELS AS SPECIFICATION BASES NOR DOES IT REDUCE THE MARGIN OF SAFETY AS

DEFINED IN THE BASIS FOR ANY TECHNICAL SPECIFICATION.

Note: "Safety analysis report" includes the FSAR, safety analyses submitted to the NRC in support of their review of the application for an operating license and subsequent amendments to the operating license, and other license commitments made to the NRC.

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

INTERDEPARTMENTAL PROCEDURES

10CFR50.59 EVALUATIONS

| NUMBER IP-3.200 | NO. |
|--------------------------|------|
| PAGE 40 C | F 41 |
| EFFECTIVE DATE 11-20- | 87 |

ATTACHMENT IP-3.200-4

Evaluation No. 88-0055

UNREVIEWED SAFETY QUESTION EVALUATION FORM (Page 3 of 4)

| 1 | origin questi | | Olive an america |
|--------|--|--|---|
| 2 | the or quest: be imprecon | more of the above question riginating document involves ion. The originating document plemented without prior appropriation for disposition of ion below. | nt. as presented, shall NOT oval of the NRC. Provide a |
| RE | COMMENDED DISP | OSITION: | |
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| | OVED BY: 75 | PLANT MANAGEX | |

Increase in Waste Evaporator Concentrates Operational Levels

The Liquid Waste Processing System collects and processes radioactive liquid waste generated during plant operation and reduces radioactivity and chemical content to acceptable levels for discharge or recycling.

The Liquid Waste Processing System consists of the Waste Holdup Tank, which collects radioactive liquid from the Reactor Coolant Drain Tank and the Reactor Containment Building sumps, the Floor Drain Tank, which collects liquids from floor drains in the Mechanical Auxiliary Building, and the Condensate Polishing Regenerative Waste Collection Tank, which collects radioactive laboratory drains, and serves as an alternate tank for floor drains. Also included in this system is the Laundry and Hot Shower Tank, which collects water from the hot shower facility.

These tanks are all equipped with eductors to ensure proper mixing prior to transfer. The liquid collected in these tanks may be processed in accordance with the Process Control Program. This program selects the optimum processing mode, filtration, demineralization, evaporation, or a combination of the three. After processing by one of these methods, the liquid is then collected in one of six Waste Monitor Tanks and analyzed prior to discharge.

The Liquid Waste Processing System waste evaporator also has the capability of processing waste streams that have a high solid content. Concentrates collect in the bottom of the waste evaporator and are subsequently transferred to the Solid Waste Processing System for solidification. The waste evaporator can process concentrates streams with up to 50% solids, however, based on the solubility of sodium tetraborate and the limits of waste evaporator and associated piping heat tracing, sodium tetraborate concentrations are limited to 16%. This limit was originall; 12%, as documented in the Safety Analysis Report. The new limit is well within the capabilities of the system and will allow more economical processing of solid waste streams in the future.

Also attached are copies of the changes to the Safety Analysis Report and the Process Control Program.

License Compliance Review

OPGP03-ZA-0003 Rev. 7 Page 17 of 17

OFGF03-ZA-0003-1 (Page 1 of 1)

| O: | riginating Document No. Change No. 1355 Revision No. 100 |
|-----------|---|
| Ti | tle 8808 11.2.2.1.1 \$ 11.4.1.2 20 |
| 0 | eck one: () Procedure () Plant Hodification (V) Other 2 eck one: (X) Quality-Related () Non Quality-Related |
| 1. | Does the subject of this review involve a charge to the facility as described in the FSAR? |
| 2. | Does the subject of this review involve a charge to the procedures as described in the FSAR? |
| 3. | Does the subject of this review conduct tests and/or experiments but described in the FSAR? |
| 4. | Does the subject of this review require a change to the Technical Specifications? |
| If Sai | may of the above questions (01 through 64) are marked YES, complete and attach a 10077850.59 |
| 5. | Does the subject of this review represent or create a potential fire hazard, affect fire protection training or administration, emergency lighting or communications, or protection of the methods for achieving and maintaining safe shutdown in the event of a fire? (Ono, () yes - attach a Fire Mazards Evaluation |
| 6. | Does the subject of this review represent or create a potential radiological hazard to the |
| | (N)no, ()yes - attach a Radiological Environmental Evaluation |
| 7. | Does the subject of this review represent or create a potential non-radiological hazard to the environment? (X)no, () yes - attach a Non-radiological Environmental Evaluation |
| 8. | Does the subject of this review represent or create a potential ALAFA concern? (1900, () yes - attach a ALAFA Evaluation |
| 9. | Does the subject of this review represent or create a potential industrial safety hazard? (App., ()ye - attach an Industrial Safety Evaluation |
| 10. | Does the subject of this review represent or create a potential to reduce the commitments of the Nuclear Security Program? (x)no, ()yes - attach a Nuclear Security Evaluation |
| 502 | MED BY (1). 8. South II MIE Morch 19 1933 |
| EVI | END ET TEChara MET 3/17/88 |
| his | FURM, when completed, shall be retained for the life of the plant. |

| OLATH TEXAS PROJECT EL | ECTRIC GENERATING STATI | ON NUMBER | NO. |
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| | | 11-1.174 | 0 |
| INTERDEPART | MENTAL PROCEDURES | PAGE 12 OF | 12 |
| CHANGE NOTICES F | OR LICENSED DOCUMENTS | DATE 07/13/8 | 37 |
| | ATTACHMENT IP-1.19Q-01 | 7 | |
| CHANGE NOTIC | | v | |
| CHANGE NOTIC | ES FOR LICENSED DOCUMENT FOR | | |
| | | Change No. 1355 (1) | - |
| | | Date 3/10/88 (2) | |
| Originator (3) | | Organization | |
| William F. Scott, II | 正、人才の | TS/CO&A | |
| Change Required: (5) The LNPS waste evaporator | is designed to operate on a | batch basis and the | e |
| contents are concentrated | to the optimum operational | levels as specified | |
| opcpol-ZA-0021 (Radioacti | ogram. in FSAR are not consistent w we Waste Processing Guidelin ure in the Process Control P | es) Autou is terere | in |
| Reviewed and Approved by | Supervising Engineer, Licer | nsing (7) | |
| | or | | |
| Reviewed and Approved by | (8) Manager, Environmental Asses Waste Management | (8) ssment/ | |
| Change Contained or Appor | and in Amendment Number | \$ | |
| | LEG TH MEELLOWELLE L'ONDERS | 4.6.4 | |
| Verified by | Date | (9) | |



(6) To protect the integrity of the LWPS evaporator the pH of the concentrates is maintained in the range which produces sodium tetraborate, not boric acid. The WECTs provide extra storage capacity for the WMTs and can be discharged through the effluent radiation monitor to the CWS. Additional surge capacity is provided by two surge tanks located in the Fuel Handling Building (FHB). Processing consists of any of the following modes:

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- 1. Filtration, evaporation, and demineralization
- 2. Filtration and raporation
- 3. Filtration eralization
- 4. Filtration

The processing macromined by he quality of the liquid to be processed. The normal macromined is expected to be one of the following combinations: filtration, filtration and/or evaporation or filtration, evaporation and/or demineralization.

The waste evaporator is a forced circulation crystallizer with a 30-gal/min distillate processing flowrate. In addition this evaporator can also be used as a backup to the BRS recycle evaporator.

The LWPS waste evaporator is designed to operate on a batch basis and the contents are concentrated to approximately 12 wt percent borio and or 25 wt percent sodium sulphate. Operation of the Counter Levels he specially in The Paccess General Paccess.

The waste evaporator must be operated at a basic pH. Therefore the pH of the feed is adjusted as needed prior to transfer into the evaporator. Feed enters the discharge piping of the recirculation pump, is heated in the heating element and circulated through the vapor body. When the desired concentration is achieved, as indicated by the evaporator density instrumentation, the concentrate is pumped to the concentrate storage tank for eventual solidification, or directly to solidification bypassing the tanks.

Evaporator distillate is condensed and cooled in the distillate condenser and subgooler skid and further treated by the waste evaporator condensate demineralizer or WHT purification demineralizer before transfer to one of the waste monitor tanks.

The two largest sources of liquid volume to the LWPS are the Condensate Polishing System regeneration waste and the equipment floor drains. The equipment floor drains represents a relatively continuous source to the LWPS whereas the condensate polishing system regeneration waste is an infrequent high-volume, short-duration, batch source. The Condensate Polishing System regeneration waste is a source to the LWPS only during periods of . I normal operations when there is above normal primary-to-secondary leakage, or excessive fuel defects. At these times the regeneration waste is directed to the CPRWCT as discussed in Section 10.4.6.2; during these periods the equipment and floor drains are directed to the FDT. Processing of this waste in the CPRWCT is identical to the processing specified for waste in the FDT. The system also provides the capability to discharge the condensate polishing regeneration waste directly to the CWS if processing is not required.

Laundry and hot shower drains are normally filtered and released. However, the waste evaporator is capable of processing the laundry and hot shower waste after addition of an antiforming agent, if treatment of these wastes is required prior to their release.

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- e. Valves in contact with waste streams are of a design that minimizes pockets or crud traps. Remote operated valves are used, where necessary, in high radiation areas.
- f. Piping is designed to provide a clean, free-flowing path. Bend radius is a minimum of five pipe diameters for all resin slurry piping. All resin slurry piping is provided with means for backflushing and is sloped, where possible, to prevent settling in the pipes.
- g. Waste tanks have provisions for monitoring level and alarming potential overflow conditions.
- h. Mixing and handling equipment are provided with redundant drives or with means for manual operation, where necessary, from low-radiation areas in the event of drive failures.
- Remote control of the system and process is used to the maximum practicable extent. Control of the system is accomplished from a central control station.
- j. Internal parts in contact with the waste streams are designed to withstand an integrated radiation exposure of 10 rads over the 40-year design life.
- k. Provisions for operator surveillative during movement of waste containers are provided by use of closed-circuit television cameras and monitors.
- 1. In accordance with 10C.R50 App. A, General Design Criterion 63, radiation monitors are provided in the SWPS control area. (See Section 12.3 for further details.)

2. Types of Waste

The SWPS is designed to process these types of waste by solidification and en- β 9

- a. Expended bead-type ion exchanger resins
- b. Evaporator concentrates containing up to 12 weight percent boric acid or 25-weight percent sodium sulfate opicified in the house Countrel largram.
- Miscellaneous liquids resulting from decontamination, laboratory wastes, and system cleaning
- d. Expended liquid and air filter elements
- e. Miscellaneous dry wastes including plant equipment and/or parts thereof
- 3. Quality Group Classification
- The SWPS is classified as Non-Nuclear Safety and meets the requirements of Branch Technical Position (BTP) ETSB11-1, Rev. 1.

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Radioactive Waste Processing Guidelines

OPCP01-ZA-0021 Rev. 1 Page 4 of 4

- . 3.2.4.2 The evaporator concentrates should not be allowed to exceed any of the following criteria prior to transfer to the Waste Evaporator Concentrates Tank:
 - a: Specific Activity: 0.2 uCi/cc as total gamma activity
 - b: Na2SO4: 25% by weight
 - c: Na2B407 . 7H20: 16% by weight
 - d. C1: 2000 ppm

NOTE

Items b and c are guidelines of OPGP03-Z0-0017 (Radioactive Waste Process Control Program), item d is for protection of the evaporator.

3.2.4.3 If any of the above criteria stated in 3.2.4.2 are exceeded, dilution will be used in the Waste Evaporator Concentrates Tank.

4.0 References

- 4.1 1PSP07-WL-0001 Liquid Waste Effluent Releases, Rev. 2
- 4.2 1PCP13-WL-0006 Laundry and Hot Shower Tank Operation, Rev. 1
- 4.3 1PCP13-WL-0002 Condens: e Polishing Regeneration Waste Collection Tank Operation, Rev. 1
- 4.4 1PCF13-WL-0032 Condensate Polishing Regeneration Waste Eatch Release, Rev. 0
- 4.5 1PCP13-WL-0003 Floor Drain Tank Operations, Rev. 1
- 4.6 IPCP13-WL-0001 Waste Holdup Tank Operations, Rev. 1
- 4.7 1PCP13-WL-0009 Waste Surge Tank Operations, Rev. 0
- 4.8 OPGP03-Z0-0017 Radioactive Waste Process Control Program, Rev. 1

5.0 Support Documents

None

OPGP03-Z0-0017 Rev. 1 Page 10 of 38

- b. For resin dewatered in HICs, acceptance criteria shall be ≤ one percent (1%) of the internal volume of the HIC.
- 5.1.3.10 The implementing procedures for the processing and handling of contaminated resin are as follows:
 - a. OPCP01-ZA-0021 (Radioactive Waste Processing Guidelines)
 - OPCP10-ZR-0001 (Determination of Radioactive Waste Package Curie Content)
 - OPCP14-WS-0006 (Radioactive Waste Package Transfer to Storage)

5.1.4 Evaporator Concentrates

- 5.1.4.1 The Liquid Waste Processing System (LWPS) is designed to collect, transport, and process radioactive liquids either for release to the environment or for solidification. Liquid waste shall be concentrated in the LWPS evaporator prior to being transferred to a vendor for solidification. Evaporator influent consists principally of either:
 - a. Boric Acid (H3BO4) containing liquids
 - b. Sodium Sulfate (Na₂SO₄) containing liquids
- 25.1.4.2 Borated waste streams shall normally be processed through demineralizers for release to the environment. (Alternately, borated waste streams may be processed by evaporation in the Sodium Tetraborate (Na B 07 . 7H 0) form and transferred to a vendor for solidification. Borated waste streams may originate from the Floor Drain Tank, Waste Holdup Tank, Recycle Holdup Tank, or Laundry and Hot Shower Tank.
 - 5.1.4.3 Condensate Polishing Demineralizer regeneration chemical wastes may be required to be processed through the LWPS in the event of primary to secondary leakage. These regeneration solutions shall be transferred to the Condensate Polishing Regeneration Waste Collection Tank and subsequently concentrated in the LWPS Evaporator to the Sodium Sulfate form.

- 5.1.4.4 Each batch of evaporator concentrates shall be sampled and analyzed to provide isotopic curie content in accordance with IPCP07-ZS-0003 or 2PCP07-ZS-0005 (Local Sampling in MAB, FHB and RCB). Total activity of each processed container shall be calculated in accordance with OPCP10-ZR-0001 (Determination of Radioactive Waste Package Curie Content).
- 5.1.4.5 In accordance with Technical Specifications
 Requirements, in at least one (1) out of every ten
 (10) batches a test solidification is performed.
 The concentrates are transferred to the Vendor
 Solidification System. If the test solidification
 is acceptable as per Section 5.6 the concentrates
 are processed in accordance with the Vendor Process
 Control Program Manual.
- 5.1.4.6 Following solidification, the container shall be classified and labeled in accordance with OPCPO1-ZA-0026 (Classification and Shipment of Solid Radioactive Waste). Verification that container contents form a free standing monolith, shall be performed in accordance with the Vendor Process Control Program Manual. Following this verification the container shall be weighed and placed in storage per OPCP14-WS-0006 (Radioactive Waste Package Transfer to Storage).
- 5.1.4.7 A flow chart of the evaporator concentrates process path is shown in Addendum 4. A description and diagram of the solidification process system and component process parameters (i.e. flow, temperature, etc.) is provided in the Vendor Process Control Program Manual.
- .5.1.4.8 The implementing procedures for processing evaporator concentrates are as follows:
 - a. OPCF01-ZA-0021 (Radioactive Waste Processing Guidelines)
 - OPCP10-ZR-0001 (Determination of Radioactive Waste Fackage Curie Content)
 - OPCP14-WS-0006 (Radioactive Waste Package Transfer to Storage)
 - d. OPCP14-WS-0007 (Waste Transfer to the Portable Solidification System)

Increase in Waste Evaporator Concentrates Operational Levels

Plant Operations Review Committee

OPGP03-ZA-0004 . Rev. 8 Page 15 of 15

PORC REVIEW COVER SHEET OPGP03-ZA-0004-1 (Page 1 of 1)

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| appropriate | s reviewed this item and has determined that (check as |
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This form, when completed, shall be retained in accordance with the retention requirements of the originating document.

Increase in Waste Evaporator Concentrates Operational Levels

- 44 -

Houston Lighting & Power Company

ATTACHMENT 4

OFFICE MEMORANDUM

To

W.H. Humble

April 14, 1988

Fram

W.D. Markham WWY

Subject

SAR Change #1355: Sodium Tetraborate Concentrations SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

A SAR Change Request has been made involving a change in allowable sodium tetraborate concentrations in the waste evaporator concentrate stream from to 12% to 16%. The waste evaporator concentrates flowpath is as follows:

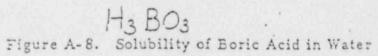
The concentrates in the bottom of the waste evaporator are discharged by the operator using the automatic Horton Process sequencer. This allows the operator to first, send the water remaining in the discharge piping from the last flush to the floor drain tank, second, the bottom concentrates are transferred to the concentrates storage tank by the concentrates transfer pump, third, the discharge piping is flushed with demin water to the floor drain tank, lastly, the discharge piping is refilled with water and left on standby.

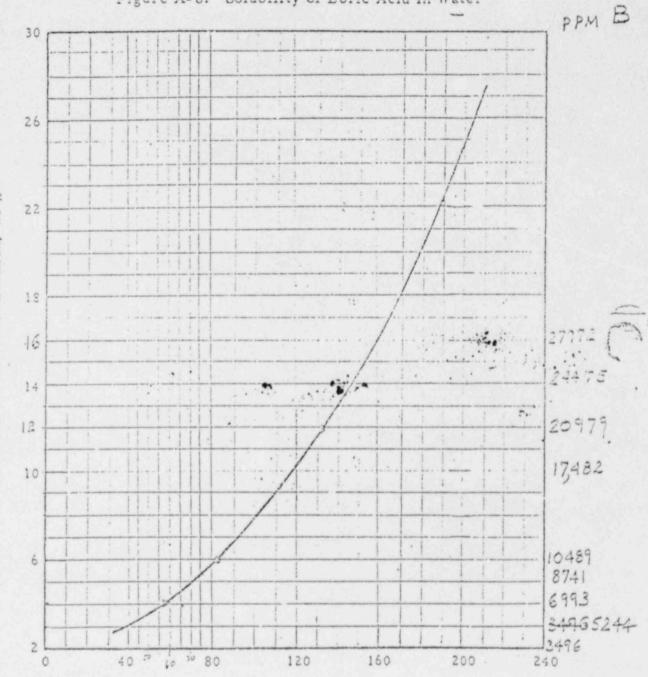
During this process, the following factors must be considered in order to evaluate the proper operation of the system with up to 16% sodium tetraborate:

The waste evaporator is not affected by the concentration of sodium tetraborate. The elements necessary for safe operation are, pH maintained at 7-9, and chlorides kept within specification. At this pH the equilibrium equation for boron shifts to the sodium tetraborate side of the equation, which is desired in the waste evaporator. The waste evaporator is capable of processing slurries of up to 50% concentrate. The discharge piping from the waste evaporator and including the concentrates storage tank is heat traced to 165 degrees F. This temperature is capable of supporting up to 16% sodium tetraborate with no solubility problems (see attached solubility curve). Chem-Nuclear can solidify concentrates up to 20% sodium tetraborate with existing equipment.

The original 12% sodium tetraborate limit in the SAR appears to be taken from the system description manual and is one of several examples of over-description in the SAR. The 16% limit is a safe one, and will also allow us to process solid waste more economically in the future.







Temperature, F



Increase in Waste Evaporator Concentrates Operational Levels

This change allows batching of concentrates in the waste evaporator as specified by the process control program. This allows more economical processing of waste streams that are processed through the waste evaporator. This change will allow batching of up to 16% sodium tetraborate vice 12% originally allowed by the safety analysis report. This change will not affect operation of any plant equipment. Solidification of these waste streams should not result in any additional exposure to plant personnel. The radiation levels in the concentrates storage tank may increase due to the increased concentration of sodium tetraborate but the decreased number of transfers will offset any additional exposure that might result from this. No additional exposure to plant personnel will result due to this modification.

Increase in Waste Evaporator Concentrates Operational Levels

During processing of radioactive waste by waste evaporation, the ability to process concentrates of up to 16% sodium tetraborate vice 12% as previously required by the SAR will allow more economical processing and may reduce the total amount of solid waste produced over the life of the plant.

Increase in Waste Evaporator Concentrates Operational Levels

This change allows up to 16% sodium tetraborate in the concentrates stream produced by the waste evaporator. This allows more economical processing of solid waste over the life of the plant. This change does not change expected maximum exposures to individuals in the UNRESTRICTED AREA and the general population from those previously estimated in the License application and amendments.

Increase in Waste Evaporator Concentrates Operational Levels

This item is not applicable. This is the initial reporting period for the South Texas Project Electric Generating Station.

APPENDIX C

Addition of Local Sampling for Waste Monitor Tanks D, E, F

- a. Attachment 1 A summary of the evaluation that led to the determination that the change could be made in accordance with 10CFR50.59.
- b. Attachment 2 A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems.
- c. Attachment 3 Documentation of the fact that the change was reviewed and approved by the Plant Operations Review Committee (PORC).
- d. Attachment 4 Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information.
- e. Attachment 5 An estimate of the exposure to plant personnel as a result of the change.
- f. Attachment 6 An evaluation of the change, which shows the predicted release of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments.
- g. Attachment 7 An evaluation of the change, which shows the expected maximum exposure to a member of the public in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments.
- h. Attachment 8 A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and solid waste, to the actual releases for the period prior to when the changes are to be made.

Addition of Local Sampling for Waste Monitor Tanks D, E, F

A 10CFR50.59 Determination Review Form was completed by Support Engineering. It was determined that there were no unreviewed safety questions and no further action was required.

100FR50,59 EVALUATION FORM - TYPICAL

(Page 1 of 2)

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10CFR50.59 EVALUATION FORM - TYPICAL

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If any answer is affirmative, perform an Unreviewed Safety Question evaluation.

If all answers are negative, no Unreviewed Safety Question evaluation is required.

Documentation of this review must be retained with the review package for the duration of the station license.

Note:

"Safety analysis report" includes the FSAR, safety analyses submitted to the NRC in support of their review of the application for an operating license and subsequent amendments to the operating license and other license commitments made to the NRC.

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10CFR50.59 EVALUATION FORM - TYPICAL

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100FR50.59 EVALUATION FORM - TYPICAL

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Addition of Local Sampling for Waste Monitor Tanks D, E, F

The Liquid Waste Processing System collects and processes radioactive liquid waste generated during plant operation and reduces radioactivity and chemical content to acceptable levels for discharge or recycling.

The Liquid Waste Processing System consists of the Waste Holdup Tank, which collects radioactive liquid from the Reactor Coolant Drain Tank and the Reactor Containment Building sumps, the Floor Drain Tank, which collects liquids from floor drains in the Mechanical Auxiliary Building, and the Condensate Polishing Regenerative Waste Collection Tank, which collects radioactive laboratory drains, and serves as an alternate tank for floor drains. Also included in this system is the Laundry and Hot Shower Tank, which collects water from the hot shower facility.

These tanks are all equipped with eductors to ensure proper mixing prior to transfer. The liquid collected in these tanks may be processed in accordance with the Process Control Program. This program selects the optimum processing mode, filtration, demineralization, evaporation, or a combination of the three. After processing by one of these methods, the liquid is then collected in one of six Waste Monitor Tanks and analyzed prior to discharge.

These tanks are all equipped with eductors to ensure proper mixing prior to transfer. The liquid collected in these tanks may be processed in accordance with the Process Control Program. This program selects the optimum mode for processing, filtration, demineralization, evaporation, or a combination of the three. After processing by one of these methods, the liquid is then collected in one of six Waste Monitor Tanks and analyzed prior to discharge.

This change involved Waste Monitor Tanks D, E, and F. These tanks are located outdoors in the plant yard. Sampling these tanks was performed via three one inch drain valves, and contamination of personnel was likely. A sample sink with three common sample valves connected to these drain valves as an aid to sampling and to help reduce the likelihood of personnel contamination. See attached Field Change Requests and 10CFR50.59 Review Determination Forms for detail.

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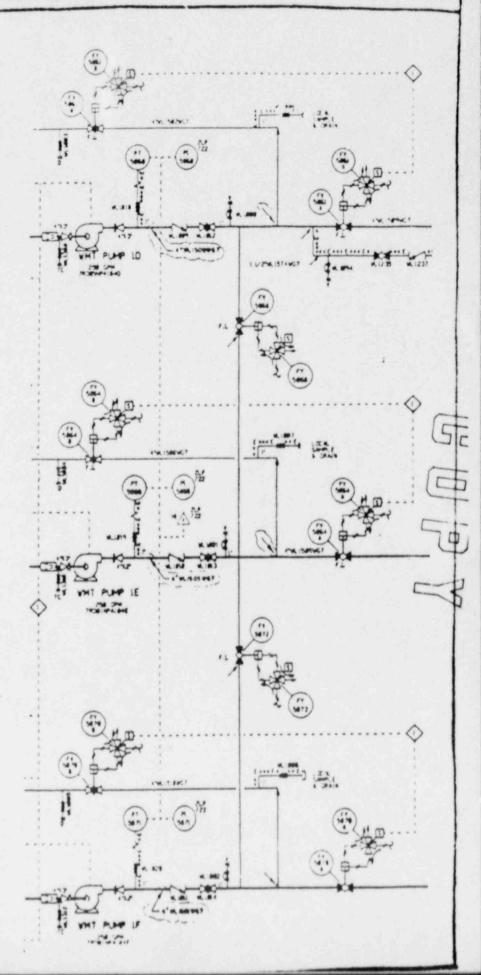


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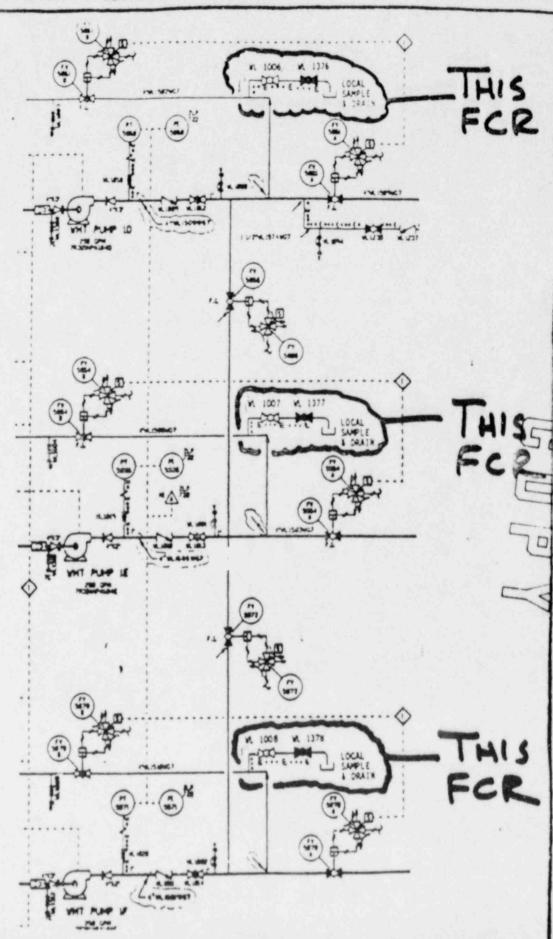


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10CFR50.59 EVALUATION FORM - TYPICAL

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10CFR50.59 EVALUATION FORM - TYPICAL

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If any answer is affirmative, perform an Unreviewed Safety Question evaluation.

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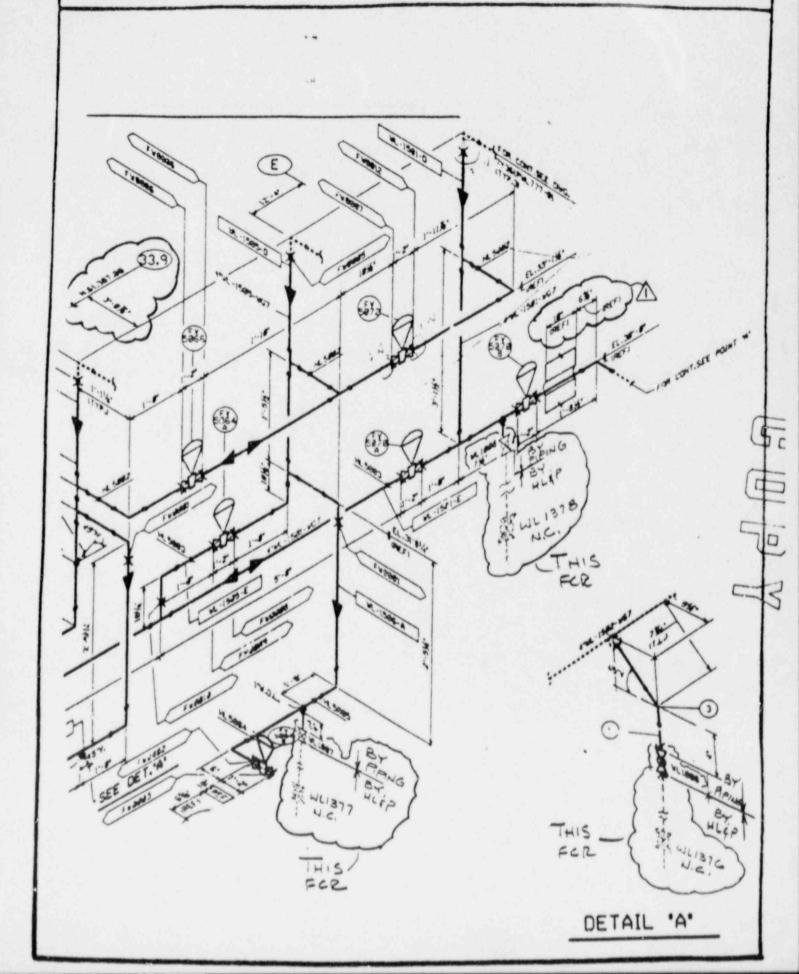
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SOUTH TEXAS ! A CALCT SUECTRIC GENERATING STATION

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FIELD CHANGE REQUEST



10CFR50.59 EVALUATION FORM - TYPICAL

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10CFR50.59 EVALUATION FORM - TYPICAL

(Page 2 of 2)

If any answer is affirmative, perform an Unreviewed Safety Question evaluation.

If all answers are negative, no Unreviewed Safety Question evaluation is required.

Documentation of this review must be retained with the review package for the duration of the station license.

Note:

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Addition of Local Sampling for Waste Monitor Tanks D, E, F
A PORC Review was not required for this design change.

Addition of Local Sampling for Waste Monitor Tanks D, E, F

The Liquid Waste Processing System (LWPS) Waste Monitor Tanks provide a final collection point prior to discharge to insure all applicable limits are met. There are six Waste Monitor Tanks, three are located in the Mechanical Auxiliary Building, three are located in the plant yard. Sampling of the outdoor tanks was difficult to perform due to the sample points consisting of 1" drain valves. Control of flow rate from these valves was difficult and the chances of personnel contamination was increased. A sample sink located on the catwalk above the Waste Monitor Pump Skid was added with three sample valves connected with stainless steel piping to the drain valves. This sink was much easier to access, provided reasonable flow rates for sampling, and greatly decreased the chances of personnel contamination. Each outdoor tank may be sampled from a single point from a much safer location.

Addition of Local Sampling for Waste Monitor Tanks D, E, F

This change added a sample sink at the Waste Monitor Tank D, E, and F Pump Skid which allowed easier sampling and decreased the chances of personnel contamination. This change will not create any additional exposure to plant personnel as a result.

Addition of Local Sampling for Waste Monitor Tanks D, E, F

This change added a sample sink at the Waste Monitor Tank D, E, and F Pump skid and it does not change the amount of chemistry of any waste stream in the Liquid Waste Processing System. This modification does not change the amount of liquid and gaseous effluents, and solid waste prediction in the License application and amendments.

Addition of Local Sampling for Waste Monitor Tanks D, E, F

This modification added a sample sink at the Waste Monitor Tank D, E, F Pump % Xid and does not affect the amount or radiochemistry of any waste stream. This modification will not result in a change to the expected maximum exposures to a member of the public in the UNRESTRICTED AREA and to the general population as previously estimated in the License application and amendments.

Addition of Local Sampling for Waste Monitor Tanks D, E, F

This item is not applicable. This is the initial reporting period for the South Texas Project Electric Generating Station.

APPENDIX D

Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

- a. Attachment 1 A summary of the evaluation that led to the determination that the change could be made in accordance with 10CFR50.59.
- b. Attachment 2 A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems.
- c. Attachment 3 Documentation of the fact that the change was reviewed and approved by the Plant Operations Review Committee (PORC).
- d. Attachment 4 Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information.
- e. Attachment 5 An estimate of the exposure to plant personnel as a result of the change.
- f. Attachment 6 An evaluation of the change, which shows the predicted release of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments.
- g. Attachment 7 An evaluation of the change, which shows the expected maximum exposure to a member of the public in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments.
- h. Attachment 8 A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and solid waste, to the actual release for the period prior to when the changes are to be made.

Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

A 10CFR50.59 Determination Review Form was completed by Support Engineering. It was determined that there were no unreviewed safety questions and no further action was required. See attached forms for detail.

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION 10CFR50.59 DETERMINATION REVIEW FORM

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Justification for Proposed Change:

Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

The Liquid Waste Processing System collects and processes radioactive liquid waste generated during plant operation and reduces radioactivity and chemical content to acceptable levels for discharge or recycling.

The Liquid Waste Processing System consists of the Waste Holdup Tank, which collects radioactive liquid from the Reactor Coolant Drain Tank and the Reactor Containment Building sumps, the Floor Drain Tank, which collects liquids from floor drains in the Mechanical Auxiliary Building, and the Condensate Polishing Regenerative Waste Collection Tank, which collects radioactive laboratory drains, and serves as an alternate tank for floor drains. Also included in this system is the Laundry and Hot Shower Tank, which collects water from the hot shower facility.

These tanks are all equipped with eductors to ensure proper mixing prior to transfer. The liquid collected in these tanks may be processed in accordance with the Process Control Program. This program selects the optimum processing mode, filtration, demineralization, evaporation, or a combination of the three. After processing by one of these methods, the liquid is then collected in one of six Waste Monitor Tanks and analyzed prior to discharge.

These tanks are all equipped with eductors to ensure proper mixing prior to transfer. The liquid collected in these tanks may be processed in accordance with the Process Control Program. This program selects the optimum mode for processing, filtration, demineralization, evaporation, or a combination of the three. After processing by one of these methods, the liquid is then collected in one of six Waste Monitor Tanks and analyzed prior to discharge.

This change involved the level indication equipment for the Waste Evaporator Condensate Tanks. These tanks collect distillate from the waste evaporator and this water may be further processed as reactor makeup water or disposed of as liquid waste. Originally, the instruments were attached to atmosphere on the low pressure side of the transmitter, with the high pressure side connected to the tank bottom. Pressure changes in the vapor space at the top of the tank due to condensation in the plant vent header caused erroneous readings. The arrangement was modified to connect the low pressure side of the level transmitter to the vapor space at the top of the tank, thus eliminating the effect of any pressure variations that might occur.

Attached are copies of the Field Change Requests that authorized this change, along with the respective 10CFR50.59 Determination Review Form.

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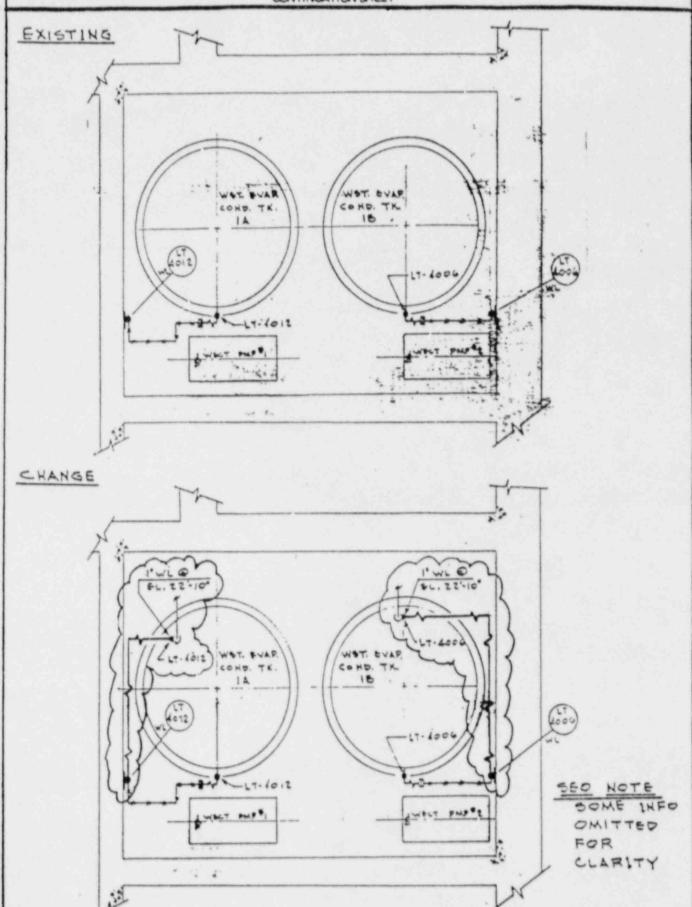
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SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION JOB NO. 14928

1. PAGE 2 OF 2 NO HBZ-01949

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FIELD CHANGE REQUEST CONTINUATION SHEET



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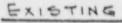
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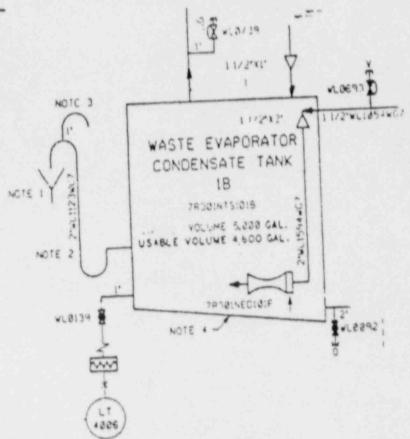


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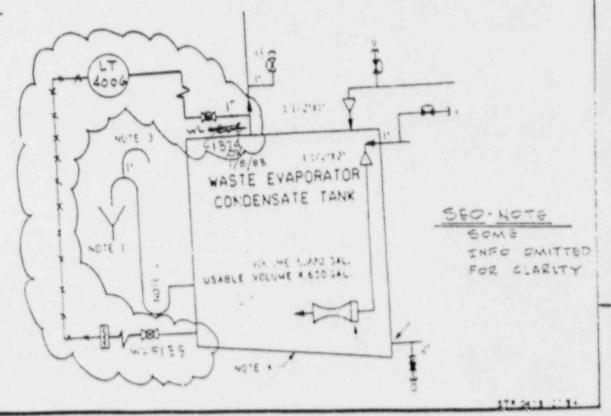
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FIELD CHANGE REQUEST CONTINUATION SHEET





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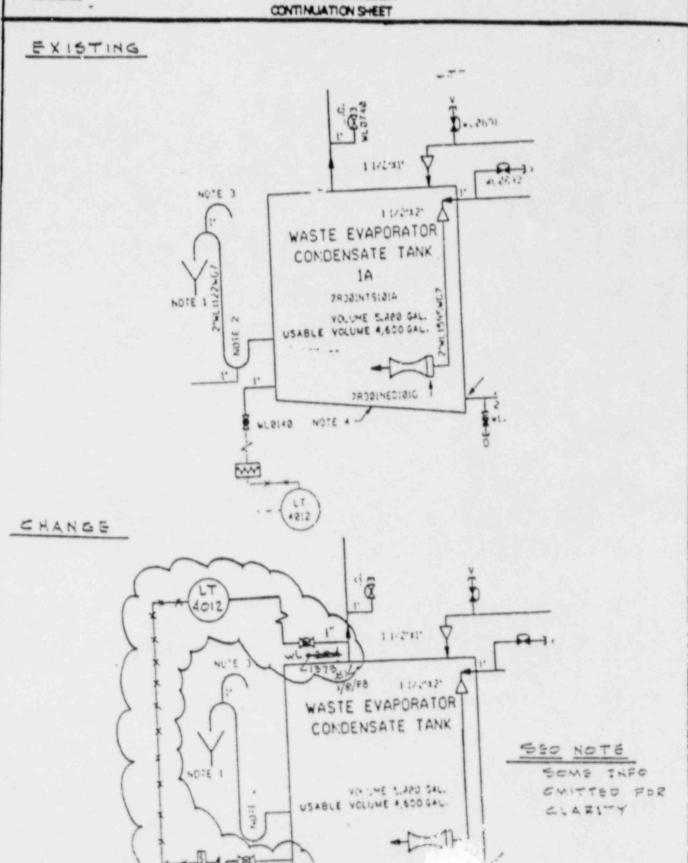




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FIELD CHANGE REQUEST CONTINUATION SHEET



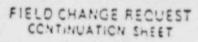
SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION 10CFR50.59 DETERMINATION REVIEW FORM

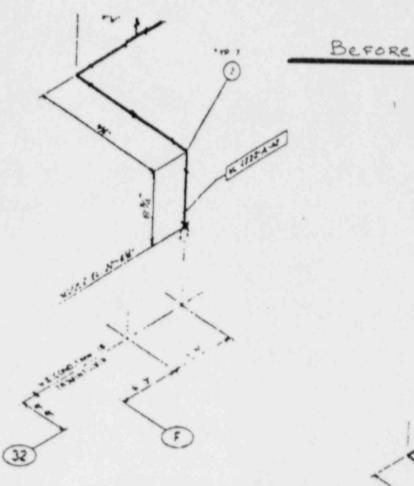
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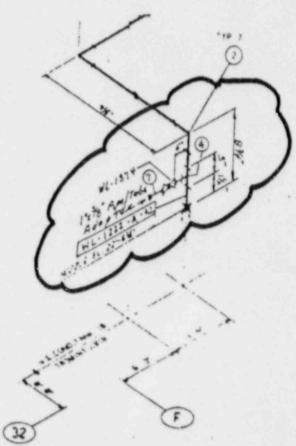
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SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION 10CFR50.59 DETERMINATION REVIEW FORM

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| Note: If response to one (1) or more of questions 1 the provide technical justification for the proposed chanalysis Report and or Plant Technical Specifical | ange(s) and delineate the ! | section(s) of | .L. 6 29.5. |

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Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

A FORC Review was not required for this design change.

Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

The ste Evaporator Condensate Tank Condensate level instrument was or smally installed with the low pressure side of the transmitter connected to atmosphere. This transmitter was calibrated based on atmosphere, however the vapor space at the top of the tank experiences pressure variations due to condensation of liquid in the plant vent header that is connected to this tank. This design change added piping to connect the low pressure side of the level transmitter to the vapor space of the tank, thus eliminating the effect of any pressure variations that might occur in the tank. Two new isolation valves, one for each low pressure side line was also added. This change was documented in the applicable Field Change Request.

Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

This change allows proper level indication in the Waste Evaporator condensate Tank and will cause no additional exposure to plant personnel.

Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

This change allows proper level indication in the Waste Evaporator Condensate Tank and it does not change the amount or chemistry of any waste stream in the Liquid Waste Processing System. This modification does not change the amount of liquid and gaseous effluents, and solid waste predicted in the License application and amendments.

Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

This modification allows proper level indication in the Waste Evaporator Condensate Tanks and does not affect the amount or radiochemistry of any waste stream. This modification will not result in a change to the expected maximum exposures to a member of the public in the UNRESTRICTED AREA and to the general population as previously estimated in the License application and amendments.

Modification to Level Indication Equipment for Waste Evaporator Condensate Tanks

This item is not applicable. This is the initial reporting period for the South Texas Project Electric Generating Station.

APPENDIX E

Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid

- a. Attachment 1 A summary of the evaluation that led to the determination that the change could be made in accordance with 10CFR50.59.
- b. Attachment 2 A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems.
- c. Attachment 3 Documentation of the fact that the charge was reviewed and approved by the Plant Operations Review Committee (PORC).
- d. Attachment 4 Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information.
- e. Attachment 5 An estimate of the exposure to plant personnel as a result of the change.
- f. Attachment 6 An evaluation of the change, which shows the predicted release of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments.
- g. Attachment 7 An evaluation of the change, which shows the expected maximum exposure to a member of the public in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments.
- h. Attachment 8 A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and solid waste, to the actual release for the period prior to when the changes are to be made.

Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid

A 10CFR50.59 Determination Review Form was completed by Support Engineering. It was determined that there were no unreviewed safety questions and no further action was required. See attached forms for detail.

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION 10CFR50.59 DETERMINATION REVIEW FORM

| THE DELETE VELLAND CHESTIES C | Revision No | 00 | |
|---|-----------------------------|----------------|--------|
| Title DELETE VENDOR SUPPLIED FO | OW & PRESSURE | SWITCHES | |
| Check one: () Procedure (Y Plant Modification | I) Other | | |
| theck one: () Quality Related (Nonquality Related | | | |
| | | | |
| | | | |
| Does the subject of this review involve a change to t | he facility as described in | the FSAR? | |
| Peter le de fource 188 BEC Cognizant Engr. Signature | 10/22/07 | () Yes | 145 |
| BEC Cognizant Engr. Signature | Date | | |
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| Does the subject of this review involve a change to the | he procedures as describe | ed in the FSAR | , |
| Viscent C. Jours | 11/17/57 | () Yes | |
| HL&P Responsible Engr. Signature | Date | | |
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| Oces the subject of this review conduct tests and/or | exceniments not described | 10 the E2100 | |
| Poto B dC 11X | | | 1 |
| Peter a de Soura US BEC Cognizant Engr. Signature | 10/22/87 | | 1 1 |
| | Date | | |
| | | | |
| | | | |
| Does the subject of this review require a change to the | e Technical Specifications | 17 | / |
| | e Technical Specifications | () Yes | 14/ No |

Analysis Report and/or Plant Technical Specification(s) affected by the proposed change(s)

Justification for Proposed Change:

DESIGN CHECKLIST (DCL)

| | CETE VENDOR SUPPLIED I | FLOW E PRESSURE STATEM Title: LIQUID WAST | Kev. 00 P | | Of <u>2</u> |
|------------------------|--|--|--|------------|---------------------------|
| This chec process u | klist must be attached to the ini p to the point of issuance by Pro | itiating change document durin | ng the entire review ar | nd approva | el |
| AFFECTED YES NO | POTENTIALLY AFFECTED DOCUMENTS | | WHICH REQUIRE REVISION ecific Reference No.) | | TURNED OVER TO HL&I |
| | Licensing Documents FSAR FHAR ER Technical Specifications (Plant) Deficiency Evaluation Report (DER) Design Criteria System Descriptions Design Calculations Specifications ASME Design Specifications Stress Report N-5 Package (Piping & Supports) Penetration Seals Design Drawings | | | | |
| | | | | | |
| RIG. Fall | Date 10/21/87 EGS | Thush DATE 19/29/4 | 17PECCTILL | DATE | 1/16/87 |

Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid

The Liquid Waste Processing System collects and processes radioactive liquid wastes generated during plant operation and reduces radioactivity and chemical content to acceptable levels for discharge or recycling.

The Liquid Waste Processing System consists of the waste Holdup Tank, which collects radioactive liquid from the Reactor Coolant Drain Tank and the Reactor Containment Building sumps, the Floor Drain Tank, which collects liquids from floor drains in the Mechanical Auxiliary Building, the Condensate Polishing Regenerative Waste Collection Tank, which collects radioactive laboratory drains, and serves as an alternate tank for floor drains. Also included in the system is the Laundry and Hot Shower Tank, which collects water from the hot shower facility.

These tanks are all equipped with eductors to ensure proper mixing prior to transfer. The liquid collected in these tanks may be processed in accordance with the Process Control Program. This program selects the optimum mode for processing, filtration, demineralization, evaporation, or a combination of the three. After processing by one of these methods, the liquid is then collected in one of six Waste Monitor Tanks and analyzed prior to discharge.

This change involved the seal water system which supplies seal water to the pumps in the Liquid Waste Processing System. Vendor supplied flow and pressure switches were supplied with the seal water pump and tank skid. These switches supplied alarm functions that were redundant to pressure indication and flow indication and control provided in the remainder of the system. As a result, nuisance alarms were received at the Radwaste Control Panel whenever the system was operated. In order to solve this problem, the control and alarm functions of these vendor supplied switches were deleted. (See attached work documentation and 10CFR50.59 Review Determination Form)

COMMERCIAL OPERATIONS CONFIGURATION CONTROL PACKAGE

| CCP NO. | 1-J-FST-0485 |
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| DEU NO | TOOIN' DATE |

| IPLEMENTATION SCHEDULE | COVER | SHEET | REV NO. | ISSUE DATE |
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| RESPONSIBLE ENGINEER: A | | | | TE: 4/10/87 |
| SEISMIC CAT: N/A | | BE PERFORMED: | | |
| | BUILDING AREA: | | Carry Chill | |
| | | | | |
| STARTUP SYSTEMS/SUB SYSTEMS AFFECTED: | WL05 | | | |
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| - / 408 | | | | |
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| CLIENTN/A | | THER NA | | |
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CCP NO. 1- J-FST-0485 REV. 00

SOUTH TEXAS PROJECT CONFIGURATION CONTROL PACKAGE CONTENTS

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|---|---------------|------|
| 1. DESIGN CHANGE BASIS | YES 🔀 | NO 🗌 |
| 2. FSAR CHANGE REQUEST PACKAGE | YES 🗌 | № ⊠ |
| 3. MATERIALS SHEET | YES 🗌 | NO 🖂 |
| 4. DOCUMENTATION LIST | YES 🔀 | NO |
| X-FCR'S INCORPORATED INTO THIS PACKAGE: | | |
| X-FCR NOS.: NA | | |



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| CCP REV. | 00 | | | | |
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DESCRIPTION OF CHANGE

Information provided in the Design Change Basis is provided for information only to enhance the definition of the design change as provided in the attached drawings. Installation and QC verification is to be accomplished in accordance with the drawings only. No XFCR's will be accepted against the Design Change Basis.

THIS CCP IS TO DELETE THE CONTROL AND AGARM FUNCTIONS
OF THE FOLLOWING VENDOR SUPPLIED INSTRUMENTS ON
THE LWPS PUMP SEAL WATER SKID:

| NIWL- FIS - 4928 | NIWL - PIS - 4929 |
|------------------|-------------------|
| 4930 | 4931 |
| 4932 | 4933 |
| 4934 | 4935 |
| 4936 | 4937 |
| 4938 | 4939 |
| 4940 | 4941 |
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| 4 944 | 4 4945 |
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THE FLOW SNITCHES ARE TO BE DETERMINATED AT THE
SWITCHES AS EXPLAINED LATER. THE PRESSURE SWITCHES
ARE TO BE DETERMINATED AT THE ANNUNCIATOR WINDOW
98 ON ZLP-189.



| CCP NO. | 1-J-FST-0485 |
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DESCRIPTION OF CHANCE

PACE 2 0 4

ELECTRICAL

- 1. REF. EE 580 INTERIM DATA BASE

 DELGTE CABLES, DETERMINATE BOTH ENDS,

 TAPE AND TAG "SPARE".
- 2. PUST=. I DEN TO DWG 4089 00120 DIC
 - A) DETERMINATE WIRE #964 (DESIGNATED WZA-9)
 FROM TB29-66.
 - B) TERMINATE NEW WIRE #966 (DESIGNATION FUS7-2)
 TO TB29-66.
- 3. PUF. IDEN TO DWG 4089-00121-DIC
 - A) DETERMINATE WIRES # 727, 966, 726, 797 AND 7134.
 FROM RUTHY 20X/FIS4942 CONTACT PTS # 1, 3, 2, 4

 § 6 RESPECTIVELY.
 - B) DETERMINATE WIRDS # 966 9 # 7134 FROM RELAY 20x/FIS \$938 CONTACT PTS #3 3 #6 RESPECTIVELY.
 - FROM RELAY 20x/PAIOS CONTACT PT. #6.
 - D) TERMINATE NEW WIRE # 713A (DESIGNATED SLUG 53.2)
 TO RELAY 20X/PAIDS CONTACT PT. #6.
 - E) DETERMINATE WIRE # 726 (DESIGNATUD ZOX/FIS 49422)
 FROM RELAY ZOX/FV5013 CONTACT PT. #5.
 - FROM RELAY 20x/FV5098 CONTACT PT. #5.
- 4. REF IDEN TO UWG 4089.0122-ETC
 - A) DETERMINATE WIRE # 727 (DESIGNATED 20X/FIS4942-1) FROM FU39-2.
 - B) TERMINATE NEW WIRE # 727 (DESIGNATED 62/PA105-2)



| CCP NO. | 1-J-FST-0485 |
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| | pars 3 or 4 |

DESCRIPTION OF CHANGE

- C) DETERMINATE WIRE # 701 (DESIGNATED 27-1)
 FROM FUS3-2
- O) DETERMINATE WIRE # 713A (DESIGNATED ZOX/FJS4942-6)
 FROM SLUG 53-2.
- E) TERMINATE NEW WIRE# 7134 (OESIGNATED 20X/PA105-6)
- F) DETERMINATE WIRE # 966 (DESIGNATED 20X/FIS 4938.3)
 PROM FUST-2.
- G) TERMINATE NEW WIRE # 966 (DESIGNATED 29-66)
 TO FUST-2
- H) DETERMINATE WIRE# 798 (DESIGNATED ZOXIFIS 4942-3)
 FROM FUS8-2.
- I) TERMINATE NEW WIRE# 798 (OUSIGNATUD 62/PA105-8)

5. REF FOCN TO DWG 4089-00124-51C

- A) DETERMINATE WIRE#727 (DESIGNATED 201/F194942-1)
 FROM RELAY 62/PA105 CENTALT PT. # 2.
- B) TERMINATE NEW WIRE #727 (DESIGNATED FU39-2)
 TO RELAY 62/PAIDS CONTACT PT. #2.
- C) DETERMINATE WIRE # 798 (DESIGNATED 20X/FIS 4942-3)
 FROM RELAY 62/PAIDS CONTACT PT.#8.
- 0) TERMINATE NEW WIRE # 798 (06516NATED FUS8-2)
 TO RELAY 62/ PAIDS CONTACT PT. #8.
- 6. REF IOCN TO DWG 4089 00124 ETC
 - A) DETERMINATE WINE #701 (DESIGNA, JD FUS3-2)
 FROM 27-1.



| CCP NO | 1-J-FST-0485 |
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| CCP REV. | 00 |
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DESCRIPTION OF CHANCE

- 7. REF IOCN TO DWG. 4089.00127-DIC

 A) DETERMINATE WIRES #966 & 967 FROM TAWZA-9

 § WZA-10 RESPECTIVELY.
- 8. REF IDEN TO DWG. 4089-00128-DIC

 A) DETERMINATE WIRE # 8119 (DESIGNATED W2-87) AND

 WIRE # 8120 (DESIGNATED W2-88) FROM TB 28-119

 § TB 28-120 RESPECTIVELY.
- 9. REF IDON TO DWG. 4089-CO129-EIC

 A) DETERMINATE WIRES # 8119 & 8120 FROM
 FROM TBW2-87 & WZ-88 RESPECTIVELY.
- 10. REF IDON TO DWG. 4089 00/3/- DIC
 A) DETURNINATE WIRE #967 (DESIGNATED WZA-10)
 FROM RELAY ZOX/FV-5039 CONTACT PT #5.

NOTE: I FIELD MAY RE-USED DETGRMINATED WIRES IF POSSIBLE, OTHERWISE TAPE BOTH ENDS IF CANNOT BE DISCARDED.

BASIS OF CHANGE

These switches are not interlocked with thier corresponding pump and therefore cause nuisance alarms. The switches are also redundant to other pressure and flow switches in the system.

EQ CHECKLIST IMPACT YES NO XX



SOUTH TEXAS PROJECT CONFIGURATION CONTROL PACKAGE DOCUMENTATION LIST

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| 6R309247251 | - 2 | | WE LOGIC DIAGRAM | > | IDEN (K) | |
| GR 3092 42252 | 2 | | WE COLIC DIAGRAM | | IDCN (K) | |
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| 6R309242257 | 9 | | WE LOUIC DINGRAM | | IDM (K) | |
| 09224260829 | 8 | | WE LOGIC DIAGRAM | 7 | IDEM (K) | |
| GR 309242292 | 2 | | WE LOGIC DIAGRAM | | IDEN (K) | |
| 78311247502. | 5 | | LOCAL ANNUNCIATOR LIST | | IDEM (K) | |
| 52010248001 | 26 | | INSTRUMENT SETPOINT LIST . | | IDEN (K) | |
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CONFIGURATION CONTROL PACKAGE DOCUMENTATION LIST

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| (If Wander-Incl., Sub + AC) | (BC) | (T) NO. | IME | K N F L | RETIMENS | (If none, enter N/A) |
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| 9-E-WL46-01 | 4 | | | 7 | (*) " | |
| 9-E-WL47-01 | W | | | _ | (4) " | |
| 9-E-WL82-01 | M | | | 2 | (4) | |
| 9-6-10184-02 | \ | | | 2 | (4) " | |
| 9-E-WL85-01 | \ | | | 7 | (1) | |
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SOUTH TEXAS PROJECT CONFIGURATION CONTROL PACKAGE DOCUMENTATION LIST

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| COMPLETE DOC. NO. (If Vendor-Incl. Sub - AC) | REV. (BC) | AMEND. | TITLE | YES | | N M | REMARKS | CLOSURE ACTION (If none, enter N/A) |
| 4089-00129-IC 4089-00131-IC | E | | | 1 | | | IDCN | |
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| DRAWING | CHANGE | NOTICE |
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STARTUP SYSTEMS AFFECTED: WLO5

JOB NO. 14926 PAGE / OF 3

FSAR CHANCE REQUIRED? YES __ NO 🛛

REASON FOR CHANGE (Please be specific) NUISANCE ALARMS
GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES.
(REF. CARRS 1358 AND 1595).

DESCRIPTION OF CHANGE

- DELETE NOTE 5.
- @ ABANDON THE FLOW AND PRESSURE SWITCHES AS SHOWN ON PAGE 2.
- 3 ADD THE FOLLOWING NOTE:

NOTE 12. THE SWITCH FUNCTIONS ON THESE INSTRUMENTS HAVE BEEN DELETED.

0 Vincent C. Jans 11/7/87

REV HL&P DATE
-14926-001

BECHTEL ENGINEERING APPROVALS

ORIGINATOR CHECKS ECS PE

A. de Souza The Souza Committy (15) In

11 16 87 DATE



DOP NO. 1-J-FST- 0485

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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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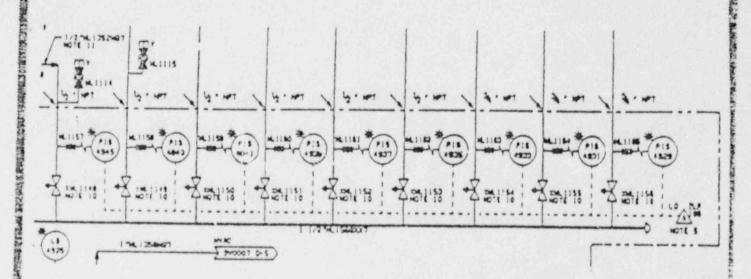
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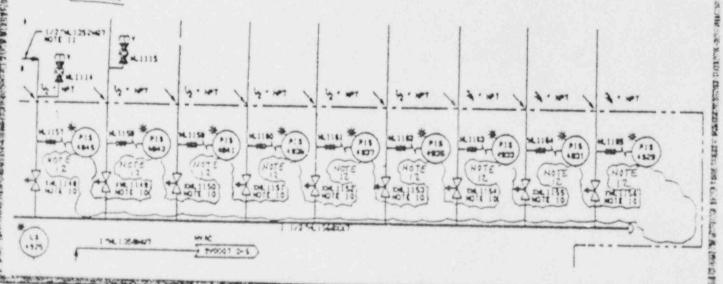
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DESCRIPTION OF CHANGE:

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INTERIH DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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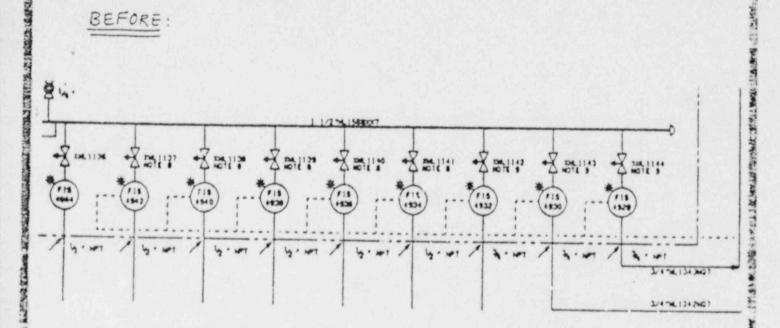
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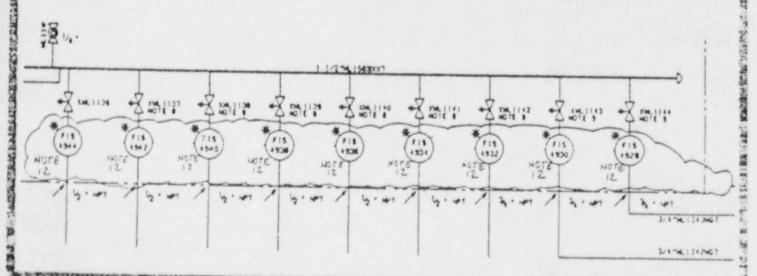
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JOB NO. 14926 PAGE 3 OF 3

DESCRIPTION OF CHANGE:

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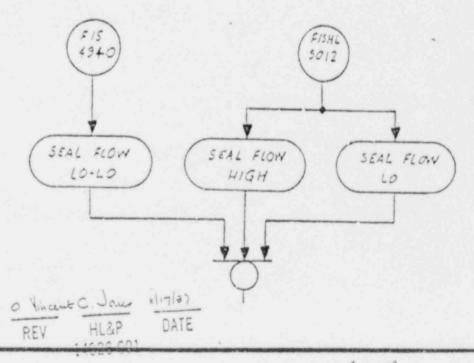
JOB NO. 14926 PAGE / OF 2

STARTUP SYSTEMS AFFECTED: WLOS

FSAR CHANGE REQUIRED? YES TO NO X

REASON FOR CHANGE (Please be specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE & FLOW SWITCHES. (REF. CARRS 1358 & 1595.

DESCRIPTION OF CHANGE



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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

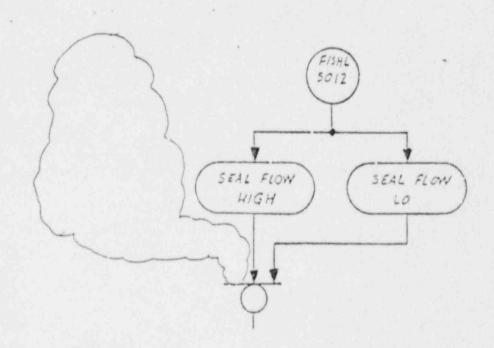
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308 NO. 14926 PAGE Z OF Z

DESCRIPTION OF CHANGE:





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INTERIM DRAVING CHANGE NOTICE (IDCN)

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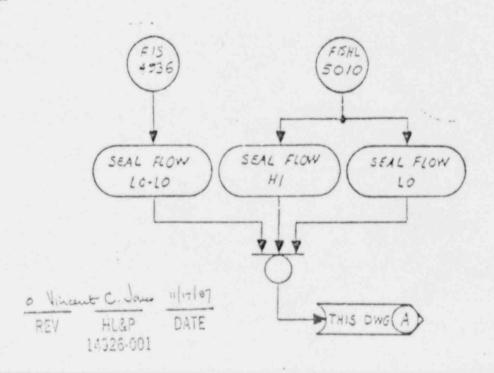
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REASON FOR CHANGE (Please be specific) __NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE & FLOW SWITCHES. (REF. CARRS 1358 8 1595

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INTERIM DRAVING CHANGE NOTICE (IDCN) (Cont'd)

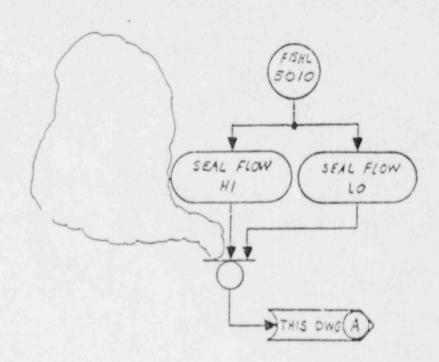
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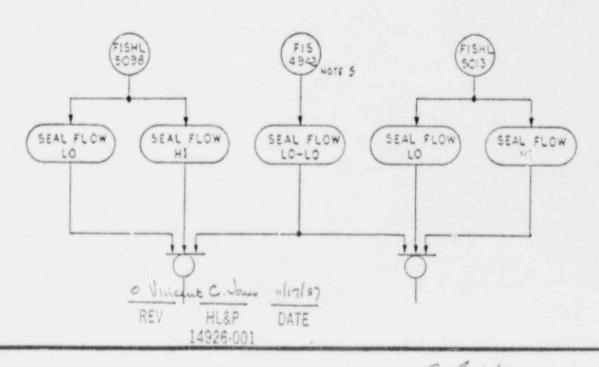
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JOB NO. 14926 PAGE / OF 7

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REASON FOR CHANGE (Please De specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES. (REF. CARRS 1358 & 1595

DESCRIPTION OF CHANGE BEFORE:



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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

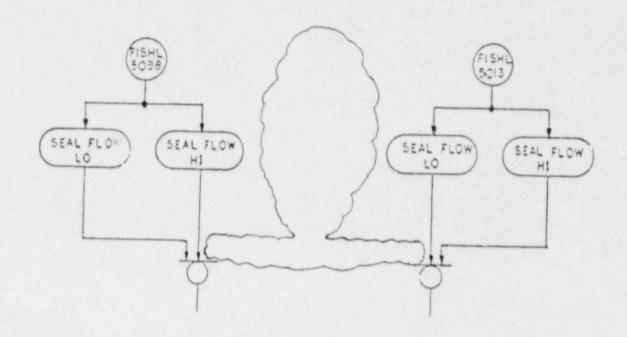
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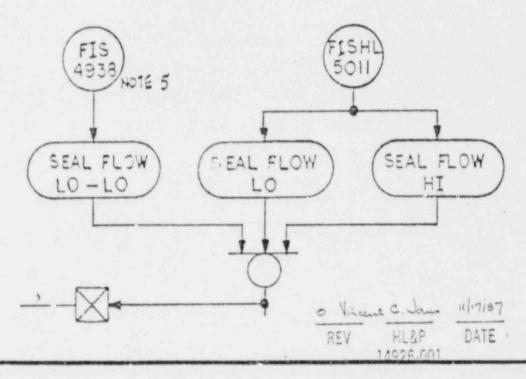
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REASON FOR CHANGE (Please be specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES. (REF. CARRS 1358 & 1595).

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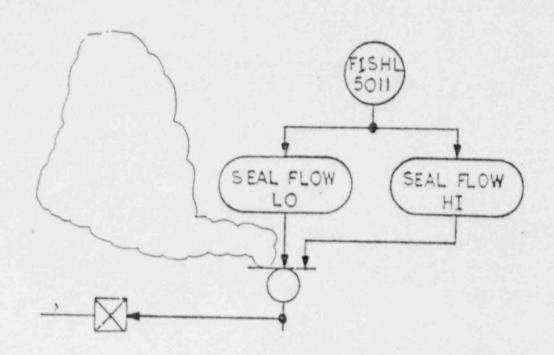
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DESCRIPTION OF CHANGE:





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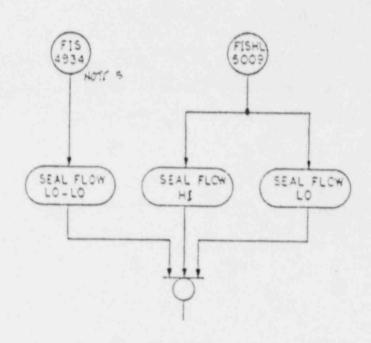
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DESCRIPTION OF CHANGE BEFOX:



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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

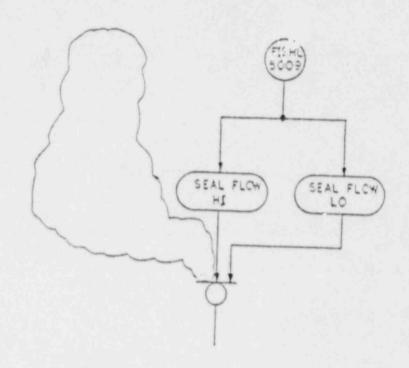
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DESCRIPTION OF CHANGE:





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| GR 309 Z42292 | | 2 | | | |

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| FSAR CHANGE REQUIRED? YES | w⊠ | | |
| REASON FOR CHANGE (F GENERATED BY SA (REF CARRS) | KID MOUNTED | PRESSURE AND | |

DESCRIPTION OF CHANGE SEE PAGES 2,384.

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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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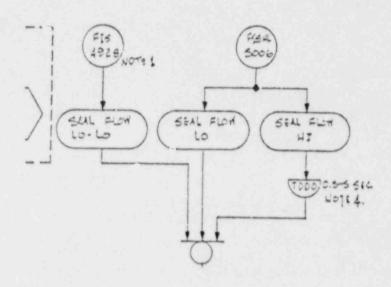
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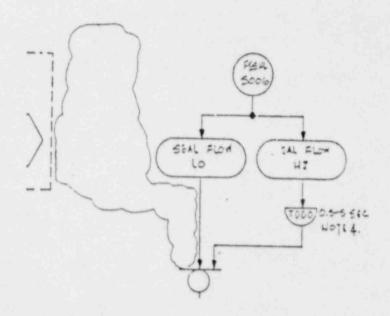
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DESCRIPTION OF CHANCE:

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INTERIH DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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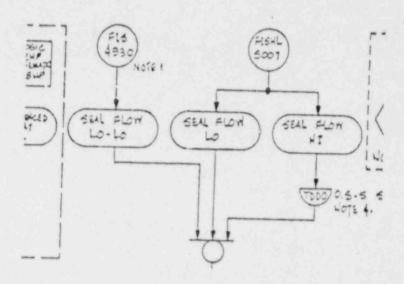
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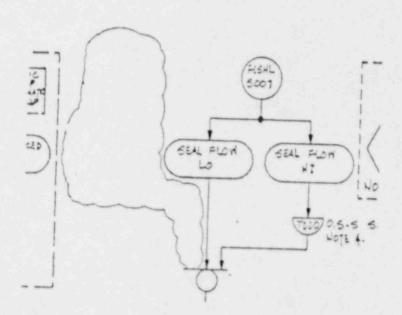
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DESCRIPTION OF CHANGE:

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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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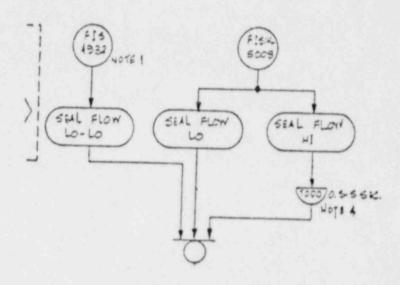
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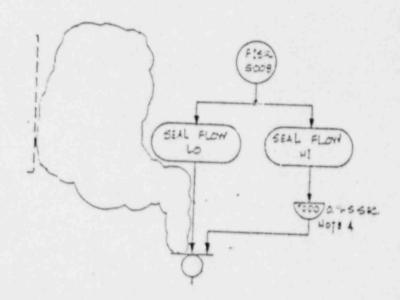
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JOB NO. 14926 PAGE 4 OF 4

DESCRIPTION OF CHANGE:

BEFORE:







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| | SKID MOUNT | fic) <u>NUISANCE ALI</u> ED PRESSURE É FLON | |

DESCRIPTION OF CHANGE SEE PAGE 2.

| BECHTEL | ENGINEER | TIME APP | PROVALS | REV HL&P DATE |
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| ORIGINATOR | DECKER | EGS | PE | 14926-001 |
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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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JOB NO. 14926 PAGE Z OF Z

DESCRIPTION OF CHANGE:

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| | REV | WINDOW DESCRIPTION: | COL | | BOX | TAG NO. ELEC DEG | P&ID LOGIC DIAGRAM | LOCAL CAS TRM LOCAL SOX TRM REMARKS | CCP NO REV NO | YENDOR DYG |
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| | - 1 | STM COND RET CHOCT HI | 09 | 89 | | N1-VL-CIS 4020 | F05027 Z42264 | 22-125,126 1-9F,FC | J-\$T-0207 | 4089-014510 |
| | | LWPS PUMPS SEAL WATER PRESS LO | 09 | | | N1-WL-SEE REMARK 9-E-WL47-01 | F90017 N/A | 22-143,144 2-9F.FC PIS 4928,31,33. | J-\$7-0207 00 35.37.39.4 | 408 1-014510 |
| | | WASTE EVAP STM COMD RET CMOCT HIGH | 00 | 89 | | N1-VL-CIS 4021 9-E-VL23-01 | F05027 Z42264 | ** | J-51-0207 | 4089-014510 |
| - | | LYPS PUMPS DISCH PRESS H' | 09 | 89 | | NI-WL-SEE REMARK | N/A | | J-\$1-0207 00 | 4089-01451C |
| | | | | | | | | *********** | ********* | |

| REV WINDOW DESCRIPTION: | LAMP BOX | TAG NO. ELEC DWG | PAID LOGIC DIAGRAM | LOCAL CAB TRM LOCAL BOX TRM REMARKS | CCP NO REV NO | VENOOR DWG |
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SOUTH TEXAS PROJECT COP NO. 1- J-FFST - 0485 REV 00

INTERIM DRAVING CHANGE NOTICE (IDCN)

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JOS NO. 14926 PAGE | OF 3

FSAR CHANCE REQUIRED? YES NO 🛛

REASON FOR CHANGE (Please be specific) _NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES. ('REF. CARRS 1358 & 1595

DESCRIPTION OF CHANGE

DELETE THE FOLLOWING SWITCHES AS SHOWN ON PAGES Z & 3.

o Vinant C Jame 11/17/87

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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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DESCRIPTION OF CHANGE: BEFORE.

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| į | | | | | T 38 SEAL WIR SUPE ME-FLY | 0-0.7 GPA | 0.625 65* 150 | ******** | 1244720562 |
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308 NO. 14926 PAGE 3 OF 3

DESCRIPTION OF CHANGE:

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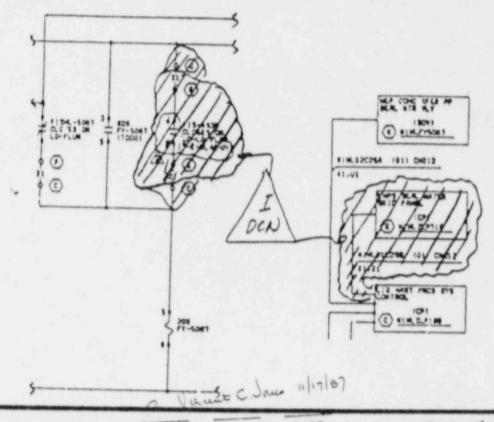
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FSAR CHANGE REQUIRED? YES MO X

REASON FOR CHANGE (Please be specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES.

REF CARR #1358 8 #1595

DESCRIPTION OF CHANGE



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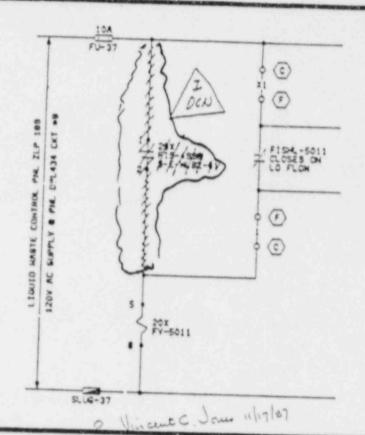
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DESCRIPTION OF CHANGE



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INTERIM DRAWING CHANGE NOTICE (IDCN)

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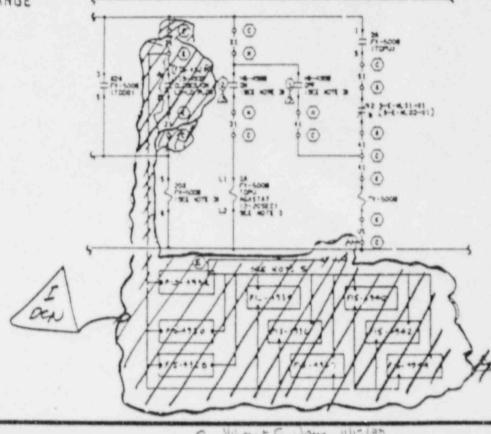
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DESCRIPTION OF CHANGE



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INTERIM DRAVING CHANGE NOTICE (IDCN) (Cont'd)

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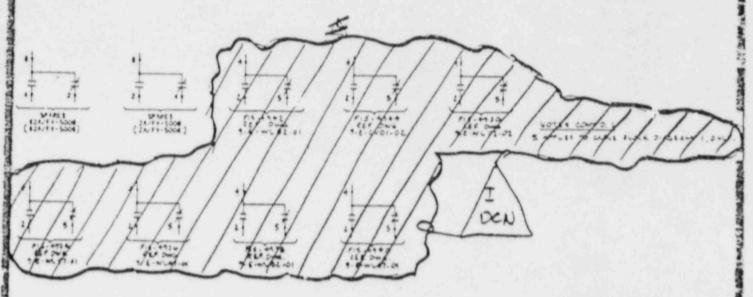
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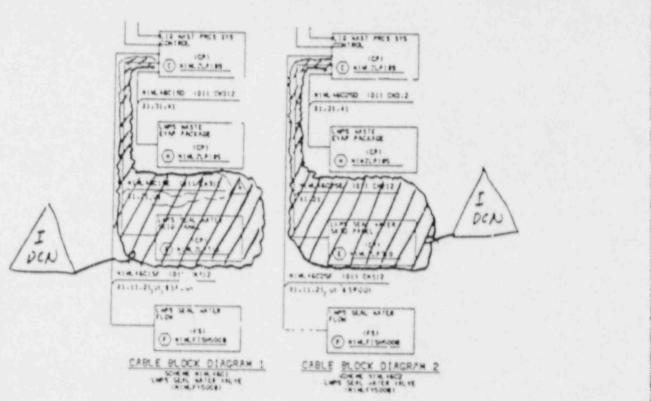
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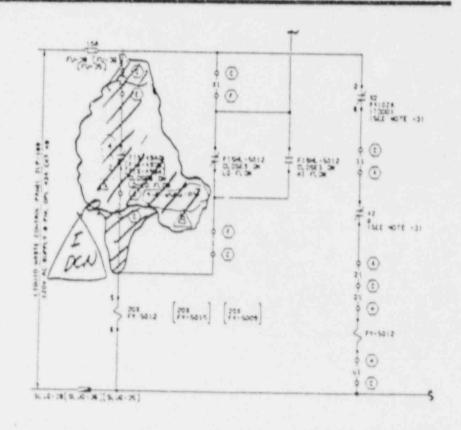
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DESCRIPTION OF CHANGE



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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

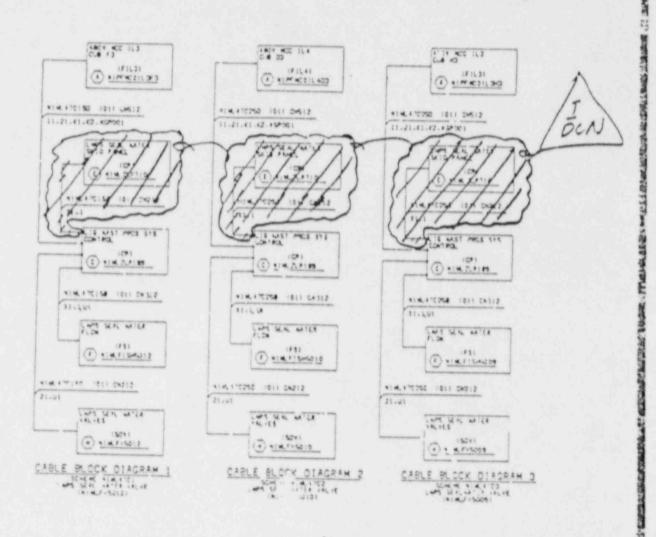
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JOB NO. 14926 PAGE 2 OF 2

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INTERIM DRAWING CHANGE NOTICE (IDCN)

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REASON FOR CHANGE (Please De specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES

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DESCRIPTION OF CHANGE

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INTERIM DRAVING CHANGE NOTICE (IDCN) (Cont'd)

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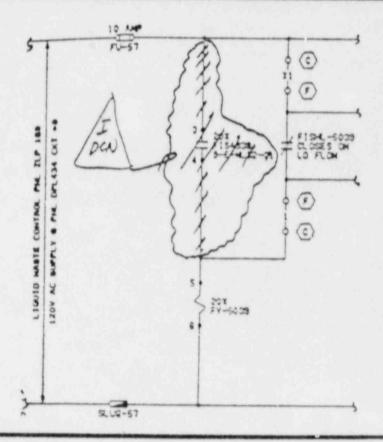
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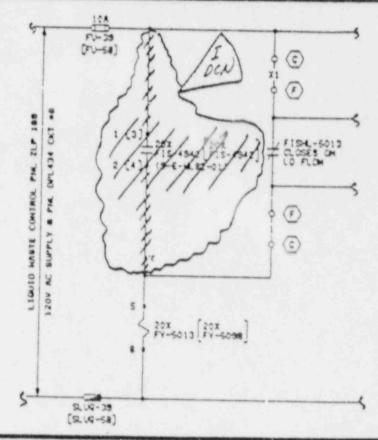
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DESCRIPTION OF CHANGE



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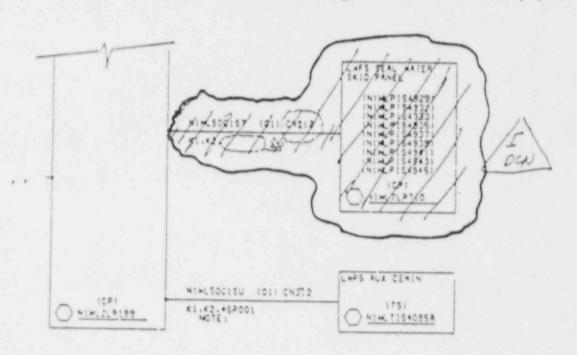
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REASON FOR CHANGE (Please De specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHUS

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FSAR CHANCE REQUIRED? YES ** 🖾

REASON FOR CHANGE (Please De specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES

REF CARR # 1358 9 # 1595

DESCRIPTION OF CHANGE

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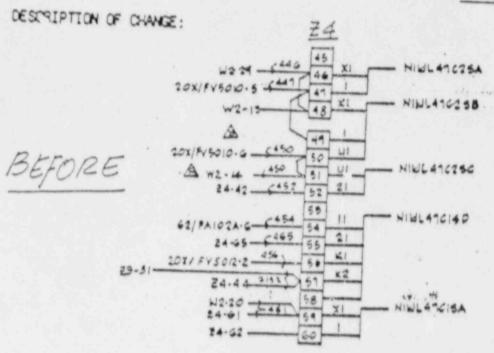
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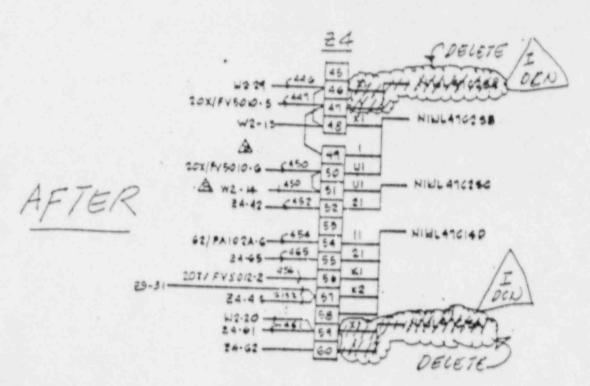
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INTERIM DRAWING CHANGE NOTICE (IDCN)

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FSAR CHANCE REQUIRED? YES . NO 🔀

GENERATED BY SKID MOUNTED PRESSURE

REF CARR #1358 9 #1095

DESCRIPTION OF CHANGE

- DELETED CABLES "NIWLAGGISE" & "NIWAGGESE",
 AS SHOWN ON PAGEE 2 AND "NIWLOZGESB"
 SHOWN ON PAGE 3.
- 2) REVISED WIRE DESIGNATION ON TB Z9-66 FROM'WZA-9' TO FUST-2" AS SHOWN ON Pg. 3.

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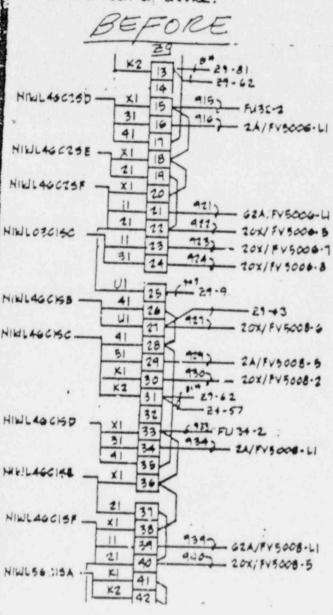
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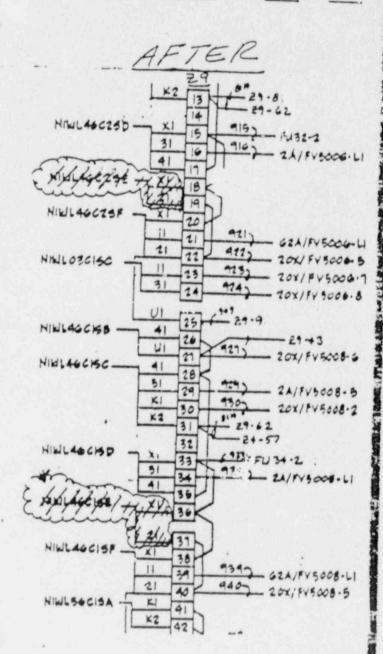
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DESCRIPTION OF CHANGE.





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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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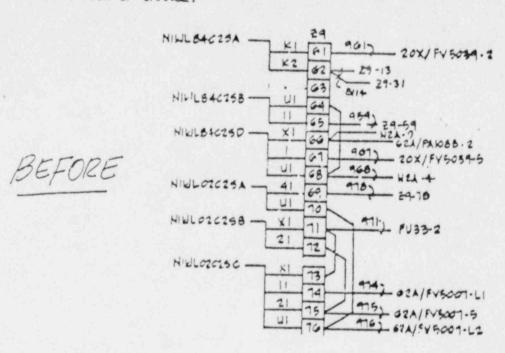
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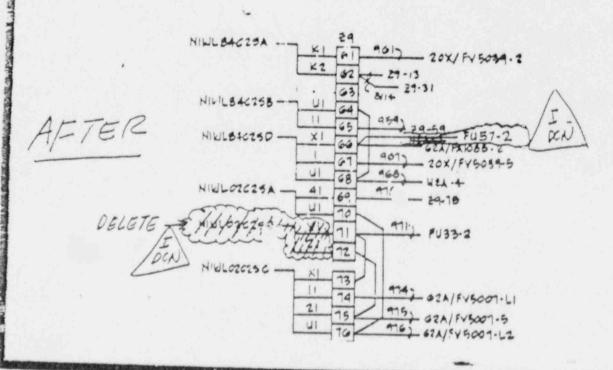
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308 NO. 14926 PAGE 3 OF 3

DESCRIPTION OF CHANGE:





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STARTUP SYSTEMS AFFECTED: 1WL 05

308 NO. 14926 PAGE / OF 3

FSAR CHANGE REQUIRED? YES

REASON FOR CHANGE (Please be specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES

RUF CARR # 1358 9 # 1595

DESCRIPTION OF CHANGE

REVISE TERMINATIONS ON RELAYS AS SHOWN ON PAGE 3.

| BECHTEL | ENGINEE | RING API | PROVALS |
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| ORIGINATOR | CHECKER | EGS | PE |
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o Vincent C. Javes "117/87 DATE HL&P 10/88 N/00 2/10/88 REV 14926-001

DATE



COP NO. 1-J-FST-0485 REV 00

INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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ORAWING MUMBER SHEET REV.
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4089-00121-IC - D

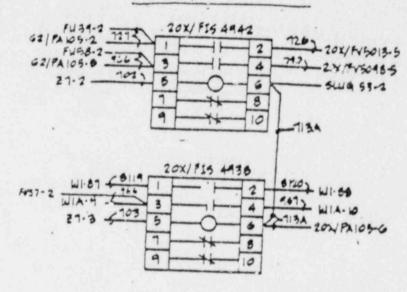
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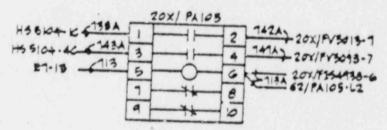
ISSUED DATE

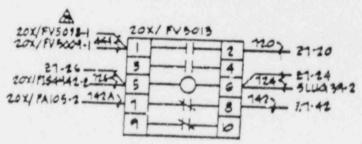
JOB NO. 14926 PAGE 2 OF 3

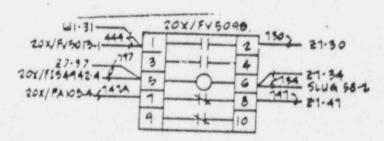
DESCRIPTION OF CHANGE:

BEFORE









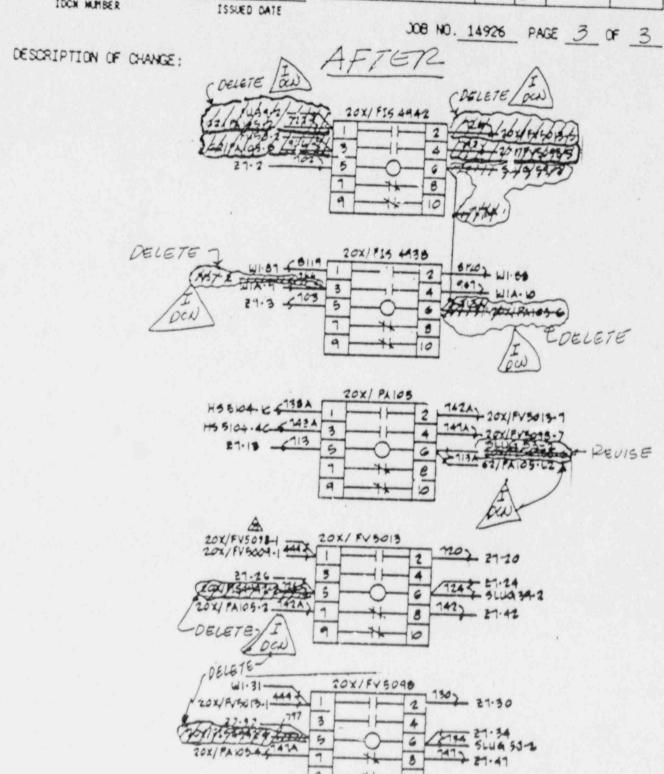


COP NO. 1-J-FST-0485 REV. 00

INTERIN DRAWING CHANGE NOTICE (IDCN) (Cont'd)

DON CONV. DATE: DRAWING MUMBER SHEET REV DUC. REV. DON NO. NO. 4089-00121-IC

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CCP NO. 1-J-FST- 0485 REV. 00

INTERIM DRAWING CHANGE NOTICE (IDCN)

DON CONV. DATE: SHEET REY. DRAWING NUMBER DWG. REV. DCM HO. NO. NO. 4089-00122-IC E

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STARTUP SYSTEMS AFFECTED: 1WL05

JOB NO. 14926 PAGE / OF 3

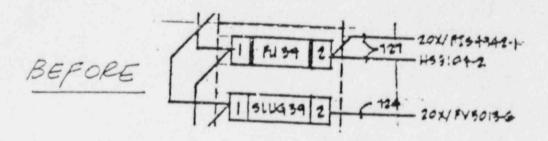
FSAR CHANGE REQUIRED? YES

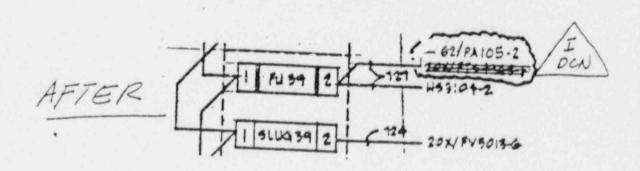
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REASON FOR CHANGE (Please De specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES

125F CARR # 1358 5' #1595

DESCRIPTION OF CHANGE REVISE AS SHOWN:





| BECHTEL | ENGINEER | RING APP | PROVALS | Vincent C. Jour | DATE | 1 0 h 900 |
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| ORIGINATOR | CHECKER | ECS | PE REV. | 14026.001 | | N.V. Chiang 2/10/88 |
| - AYRAN | E Crany 1 | m. bs | 18-3-1 | 14920.00- | 11/11/8 | NEOD 4.700 |



COP NO. 1-J-FST-0485 REV. 00

INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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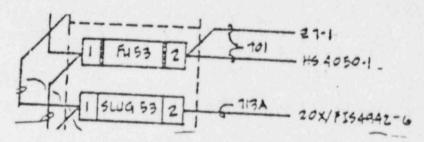
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JOB NO. 14926 PAGE Z OF 3

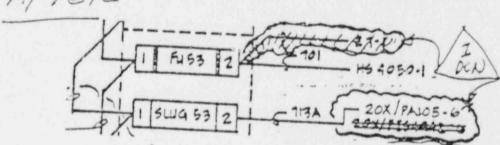
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000 NO. 1-J-FST-0485 REV. 00

INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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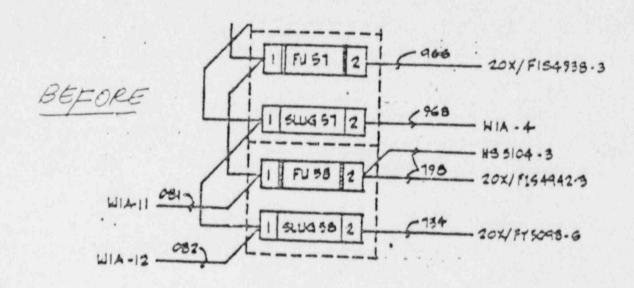
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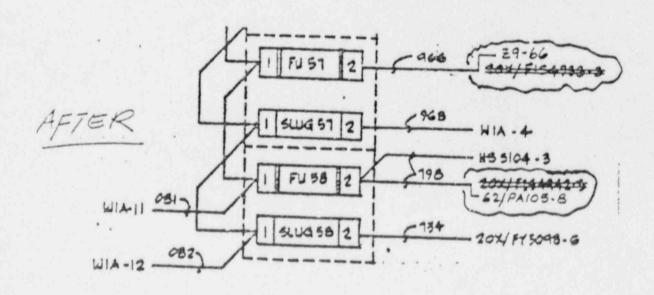
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JOB NO. 14926 PAGE 3 OF 3

DESCRIPTION OF CHANGE:

REVISE AS SHOWN:





SOUTH TEXAS PROJECT CCP NO. 1- J- FST- 0485 REV. DO

INTERIH DRAWING CHANGE NOTICE (IDCN)

DON CON. DATE: SHEET REV. DRAWING NUMBER **WO.** DUG. REV. DON NO. 40. 4089-00123-IC

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STARTUP SYSTEMS AFFECTED: 1WL05

300 NO. 14/26 PAGE / OF Z

FSAR CHANCE REQUIRED? YES MO 🔀

REASON FOR CHANGE (Please De specific) NUISANCE ALARAIS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES

REF CARR # 1358 9 # 1595

DESCRIPTION OF CHANGE

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INTERIM DRAWING CHANGE NOTICE (IDCN) (CONt'd)

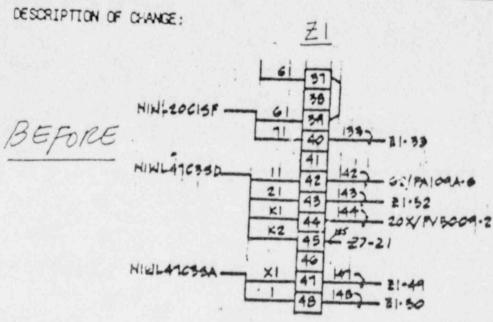
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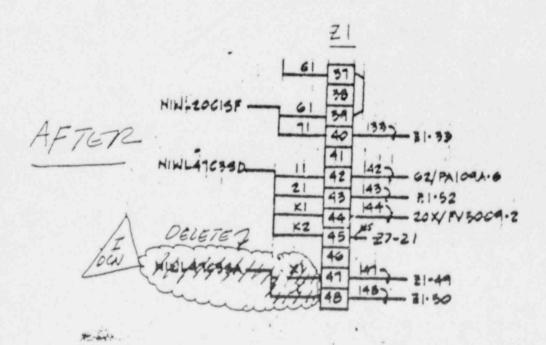
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JOB NO. 14926 PAGE 2 OF 2





CCP. NO. 1-J-FST- 0485 REV. 00

INTERIM DRAWING CHANGE NOTICE (IDCN)

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STARTUP SYSTEMS AFFECTED: 1WL05

JOS NO. 14925 PAGE / OF Z

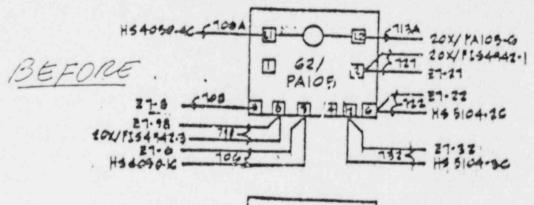
FSAR CHANCE REQUIRED? YES

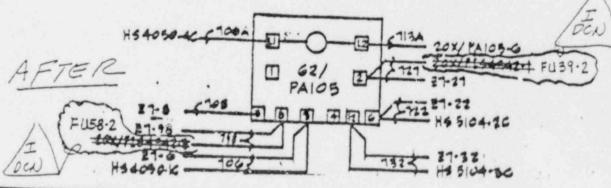
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REASON FOR CHANGE (Please De specific) NUISANCE ALARMS GENERATED SISKID MOUNTED PRESSURE AND FLOW SWITCHES

REF CARR # 1358 \$ #1595

DESCRIPTION OF CHANGE REVISE TERMINATIONS AS SHOWN.





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Q Vincent C. Jan 11/17/87 REV HL&P 14926-001

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INTERIM DRAWING CHANGE NOTICE (Inch) (Cont'd)

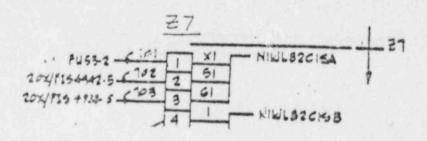
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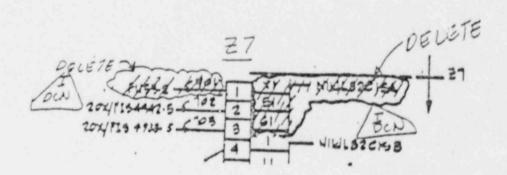
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ISSUED DATE

308 NO. 14926 PAGE 2 OF Z

DESCRIPTION OF CHANCE: REVISE TERMINATION AND DELETE CABLE "NINL 82 CISA" AS SHOWN:





SOUTH TEXAS PROJECT CCP.NO. 1-J. FST-0485 REV. 00

INTERIM DRAVING CHANGE NOTICE (IDCN)

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JOS NO. 14926 PAGE / OF /

FSAM CHANGE REQUIRED? YES

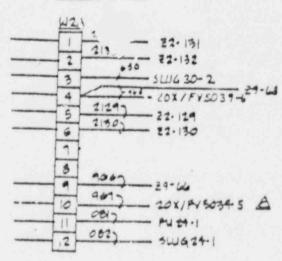
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REASON FOR CHANGE (Please be specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES.

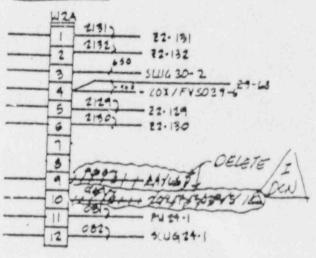
PUF CARR # 1358 ; # 1595

DESCRIPTION OF CHANGE REVISE TERMINATION AS SHOWN

BEFORE



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O Vincent C. Jan 11/17/87 DATE HL&P 14926-001

11/16/87

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NEOD Les. O Coming Hard 210/88 CATE

CCP. NO. 1-J-FST- 0485 REV. 00

INTERIM DRAWING CHANGE NOTICE (IDCN)

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FSAR CHANGE REQUIRED?

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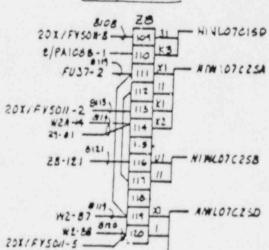
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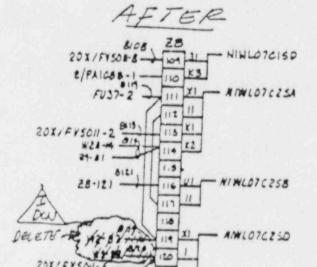
REASON FOR CHANGE (Please be specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES

CARR # 1358 9 1595

DESCRIPTION OF CHANGE ROVISE TERMINATIONS AS SHOWN.

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INTERIM DRAWING CHANGE NOTICE (IDCN)

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IDON NUMBER

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STARTUP SYSTEMS AFFECTED: 1WLOS

308 NO. 14926 PAGE / OF /

FSAR CHANGE REQUIRED? YES

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REASON FOR CHANGE (Please De specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES

CARR # 1358 & #1595

DESCRIPTION OF CHANGE REVISE TERMINATIONS AS SHOWN:

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O Vinaut C. Jans 11/7/87 Res. O Grong Blood REV HL&P DATE 14926-001

DATE

SOUTH TEXAS PROJECT CCP NO. 1-J-FS T- 0485 REV. 00

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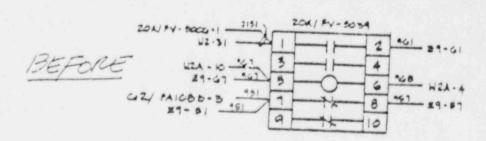
JOB NO. 14926 PAGE / OF /

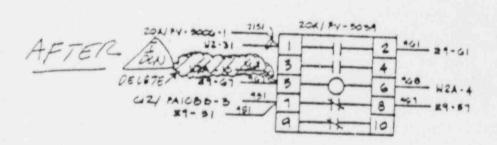
FSAR CHANCE RECVIRED? YES NO 🗵

REASON FOR CHANGE (Please De specific) NUISANCE ALARMS GENERATED BY SKID MOUNTED PRESSURE AND FLOW SWITCHES.

RUF CARR # 1358 8 #1595

DESCRIPTION OF CHANGE REVISE AS SHOWN.





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2 CHANGED PER CCT 3365
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               2 CHANGED PER CCT 3365
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              V DELETED
              V NIXHIDTSAG NIXHIDTSBF NIXHIDTSBG NIXHIDTSAT NIXHIDTSAS NIXHIDTSAV NIXHIDTSAU
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              1 SPARE --- SEE TERMINATION CARD
              2 CHANGED PER CCT 3365, FCR SED953
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              1 SPARE --- SEE TERMINATION CARD
              2 CHANGED PER CCT 3565
END OF THE REPORT
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   1JF5T0435
              TV DR XXXXXX +HLP OPS AC+
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                                  WH
              1 SPARE --- DISCONNECT ALL WIRES
 NTHEOZCZSB Z 03 SR *C N S NL 9ENLOZOZ 00 C* Z 01 CN212 NTHEZEP710 NTHEOZCZSB
  1JEST0435 TY DR XXXXXX *HLP OPS AC*
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14926-001

CCP NO. 1-5-FST-0485 REV. 00

X1 BK 21 WH 1 SPARE --- DISCONNECT ALL WIRES NIME40CISE 1 05 SR +C N S HL 9ENL4601 04 C= 2 01 CN312 NIMEZEP189 NIME46CISE 1JFST0485 TV DR XXXXXX *HLP UPS AC* CD MM NI X1 3K 21 WH U1 RD 1 SPARE --- DISCONNECT ALL WIRES 2 CHANGED PER FCR SECOSSS, SU REQ 86-388 NIME 46CTSE 2 05 SR +C N S HL PENL4601 04 C= 2 01 CN312 NIME ZEP710 NIME 46CTSE 1JFST0485 TV UR XXXXXX *HLP DPS AC* CO NI X1 8K 21 U1 RD 1 SPARE --- DISCONNECT ALL WIRES 2 CHANGED PER FCR SECOPSSISU REQ C6-384 NIWE46C2SE 1 03 SR +C N S HE 9EWE4601 01 C= 2 01 CN212 MIWEZEP189 NIWE46C2SE 1 JEST 0485 TY DR XXXXXX . . THEP DPS AC. CD NI X1 BK 2.1 1 SPARE --- DISCONNECT ALL WIRES NINE 46CZSE 2 03 SR +C N S HE PENE 4001 01 C= 2 01 CN212 NINE ZEP710 NINE 46CZSE 1JFST0435 TW DR XXXXXX *HLP OPS AC* CO MM NI X1 BK 21 WH 1 SPARE --- DISCONNECT ALL WIRES NINL47C1SA 1 06 SR MC N S HL 9ENL4701 01 C* 2 01 CN212 NINLZEP189 NINL47C1SA 1 JFSTO485 TV DR XXXXXX +HLP OPS AC+ CO MM NI X1 8K - 1 1 SPARE --- DISCONNECT ALL WIRES NINE 47C1SA 2 08 SR +C N S HL 9EHL4701 01 C= 2 01 CN212 NINE ZLP710 NIHL47C1SA 1JFST0485 TV DR XXXXXX *HLP DPS AC* CD MM NI X1 8K 1 1 SPARE --- DISCONNECT ALL WIRES NINLATCESA 1 US SR +C N S HL 9EHLATOI 01 C= 2 01 CN212 NINLILP189 NINLATCESA 1JFST0485 TV DR XXXXXX *hLP GPS AC* СЭ ММ NI X1 BK 1 SPARE --- DISCONNECT ALL WIRES N1HL47C2SA 2 03 SR +C N S HL 9EHL4701 01 C= 2 01 CN212 N1HLZLP710 N1HL47C2SA 1JFST0485 TV OR XXXXXX *HLP DPS AC * CO MM NI X1 3K 1 SPARE --- DISCONNECT ALL WIRES NINE 47C3SA 1 03 SR *C N S HE , 9ENL4701 01 C= 2 01 CN212 NINE ELP189 NINE 47C3SA 1JFST0485 TV DR XXXXXX *HLP OPS AC* CO NI X1 BK 1 411 1 SPARE --- DISCONNECT ALL WIRES NIML47C3SA 2 03 SR +C N S HL 9EWL4701 01 C= 2 01 CN212 NIMLZEP710 NIML47C3SA 1JFST0485 TV GR XXXXXX *HLP OPS AC*

CO NI K1 BK 1 1 SPARE --- DISCONNECT ALL WIRES NINESOCIST 1 07 SR +C N S HE 9ENESOOI 04 C* 3 01 CN212 NINEZEP189 NINESOCIST 1JESTOARS TY DE XXXXXX *HLP OPS AC* CD HH NI K1 5K K2 WH 1 SPARE --- DISCONNECT ALL SIRES 2 CHANGEU PER FCR SE00953, SU REQ 86-388 MINESOCIST 2 00 SR +C N S NE 92HESOOT 04 C= 3 01 CN212 NIHEZEP710 NIHESOCIST 1JEST0485 TV OR XXXXXX *HLP OPS AC* CO MM 74.1 K1 5K KZ WH 1 SPARE --- DISCONNECT ALL WIRES 2 CHANGED PER FCR SEGUPSS, SU REQ 86-388 NINE 32C1 SA 1 US SR .C N S HE 9EHEB201 00 C= 3 01 CN312 NINEZEP189 NINEB2C1 SA 1JFST0435 TV DR XXXXXX *HLP OPS AC. NI X1 BK 51 WH 61 80 1 SPARE --- DISCONNECT ALL WIRES NIMESECTSA 2 03 SR +C N S HL 9EHL8201 00 C= 3 01 CN312 NIMEZEP710 NIMESECTSA 1JFSTO485 TV DR XXXXXX *HLP OPS AC* CO MM NI X1 dK 51 WH 61 RO 1 SPARE --- DISCONNECT ALL WIRES

END OF THE REPORT

14926-001 PAGE 4 OF 4

*CCP NO. 1-J-FST-0485 REV. 00

SOUTH TEXAS PROJECT ELECTR'C GENERATING STATION 10CFR50.59 DETERMINATION REVIEW FORM

| F PRESSURE | SWITCHE | S |
|--|--|---|
| Other | | |
| | | |
| acility as described in | the FSAR? | |
| 10/29/87 | () Yes | 14N |
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| rocedures as describe | d in the ESAS | 22 |
| 11/17/57 | () Yes | WN |
| Date | | |
| oments not decembed | | |
| 10/22/87 | () Yes | 1 |
| Date | | |
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| Date | - | |
| e affirmative, the cogn s) and delineate the s affected by the propo | nizant Enginee | er shall ne Salani |
| | Dither acility as described in 10/29/87 Date TO 11/197 Date TO 12/87 Date TO 12/87 Date TO 12/87 Date | notedures as described in the FSAR? 10/29/87 Date Tocedures as described in the FSAR 11/17/97 Date Tochnical Specifications? 11/17/97 Date Tothnical Specifications? 11/17/97 Date |

DESIGN CHECKLIST (DCL)

| Title | | | Initiating Cha | | nt No. |
|-----------------------|---|--|------------------|---------------|---------------------------|
| Start-up Sys | | Title: LIQUID WASTE | Rev. 00 | | Of <u>2</u> |
| This check process up | clist must be attached to the init | tiating change document during t ject Document Control. Reference | he entire review | v and approva | 1 |
| AFFECTED YES NO | POTENTIALLY AFFECTED DOCUMENTS | AFFECTED DOCUMENTS WH (Must provide specific | Reference No.) | ION | TURNED OVER TO HL&P |
| | Licensing Documents FSAR FHAR ER Technical Specifications (Plant) Deficiency Evaluation Report (DER) Design Criteria System Descriptions Design Calculations Specifications ASME Design Specifications Stress Report N-5 Package (Piping & Supports) Penetration Seals Design Drawings | DOCUMENT NO. (OR SE | | REV. | Y |
| ORIG. Fal | Date 10/27/67 EGS | THUSE DATE 10/29/47 | e CC till | DATE | 1/16/87 |

Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid

A PORC Review was not required for this design change.

Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid

The Liquid Waste Processing System (LWPS) seal water system supplies seal water to the following pumps in the LWPS: Waste Evaporator Recirculation Pump, Waste Evaporator Auxiliary Feed Pump, Waste Evaporator Concentrates Transfer Pump, Condensate Polishing Regenerative Waste Collection Tank Pump, Laundry and Hot Shower Tank Pump, Spent Resin Sluice Pump, Waste Holdup Tank Pump, Spent Resin Transfer Pump, Floor Drain Tank Pump, Low Activity Spent Resin Transfer Pump, and the Low Activity Spent Resin Sluice Pump. Vendor supplied flow switches provided a low flow alarm and pump cutoff, and vendor supplied pressure switches supplied a low pressure alarm. The low flow pump cutoff was already controlled by field installed flow switches, the low pressure alarms were unnecessary. This arrangement resulted in nuisance alarms each time the system was secured. Adequate protection is provided by these field installed pressure switches making the vendor supplied switches unnecessary. These switches were removed by the associated work document.

Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid

This change deletes nuisance alarms associated with flow and pressure switches in the LWPS seal water system. Adequate protection is still provided by field installed flow switches. This change will not result in an additional exposure to plant personnel.

Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid

This change deletes nuisance alarms associated with the LWPS seal water system and it does not change the amount or chemistry of any waste stream in the Liquid Waste Processing System. Adequate protection is still provided by field installed flow switches in this system. This modification does not change the amount of liquid and gaseous effluents, and solid waste predicted in the License application and amendments.

Deletion Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Scal Water Skid

This modification deletes nuisance alarms associated with the LWPS seal water system and does not affect the amount or radiochemistry of any waste stream. Adequate protection is provided by field installed flow switches in this system. This modification will not result in a change to the expected maximum exposures to a member of the public in the UNRESTRICTED AREA and to the general population as previously estimated in the License application and amendments.

Deletion of Vendor Supplied Flow and Pressure Switches on Liquid Waste Process System Pump Seal Water Skid

This item is not applicable. This is the initial reporting period for the South Texas Project Electric Generating Station.

APPENDIX F

Addition of an Isolok Sampler for Spent Resin Discharge to Mobile Colidification Unit

- a. Attachment 1 A summary of the evaluation that led to the determination that the change could be made in accordance with 10CFR50.59.
- b. Attachment 2 A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems.
- c. Attachment 3 Documentation of the fact that the change was reviewed and approved by the Plant Operations Review Committee (PORC).
- Attachment 4 Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information.
- e. Attachment 5 An estimate of the exposure to plant personnel as a result of the change.
- f. Attachment 6 An evaluation of the change, which shows the predicted release of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments.
- g. Attachment 7 An evaluation of the change, which shows the expected maximum exposure to a member of the public in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments.
- h. Attachment 8 A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and solid waste, to the actual release for the period prior to when the changes are to be made.

Addition of an Isolok Sampler for Spent Resin Discharge to Mobile Solidification Unit

1. 10CFR50.59 Determination Review Form was completed by Support Engineering. It was determined that there were no unreviewed safety questions and no further action was required.

10CFR50.59 EVALUATION FORM - TYPICAL

(Page 1 of 2)

| DESCRIPTION | | |
|---|---------|---------|
| Unit # | | |
| TT PROCEDURE PLANT MODIFICATION TT CTH | ER | |
| ORIGINATING DOCUMENT NO. CCP 1-M-FST-0249 PEY. | 0 | _ |
| TITLE ADDITION OF ISOLOK SAMPLER FOR SPEN | T BE | SW. |
| DISCHARGE, AND R' LACEMENT OF QUICK DISCHOSE CONNECTION WITH BLIND FLANGES | ONNE | rcT_ |
| DESCRIPTION PROVIDE SAMPLING CAPABILITY FOR | SOL | 112 |
| WASTES BEING DISCHARGED TO THE MOBILE | | |
| SOLIDIFICATION UNIT- | | |
| Does the subject of the review involve a change to the facility as | Yes | No |
| described in the safety analysis report? | | IX. |
| Does the subject of this review involve a change to the procedures as described in the safety analysis report? | <u></u> | 3-28-88 |
| Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? | | X, |
| bes the subject of this review require a change to the Plant echnical Specification? | | 2-28-88 |
| oes the proposed change, although not described in the safety nalysis report, affect items or activities that are described n the safety analysis report? | | 1Z |
| This form, when completed, shall be retained for the life of the pl | ant.) | 7 |

10CFR50.59 EVALUATION FORM - TYPICAL

(Page 2 of 2)

If any answer is affirmative, perform an Unreviewed Safety Question evaluation.

If all answers are negative, no Unreviewed Safety Question evaluation is required.

Documentation of this review must be retained with the review package for the duration of the station license.

Note: "Safety analysis report" includes the FSAR, safety analyses submitted to the NRC in support of their review of the application for an and other license and subsequent amendments to the operating license and other license commitments made to the NRC.

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DOTH TEXAS PROJECT ELECTRIC GENERATING STATION

DESIGN CHECKLIST (DCL)

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| | Licensing Documents FSAR FHAR (Appendix B)* ER Technical Specifications (Plant) | | | | |
| | Deficiency Evaluation Report (DER) | | | | - |
| | Design Criteria | | | | + |
| | System Descriptions | | | | |
| | Design Calculations | | | | |
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SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

| 10CFR50.59 EVALUATION FORM - TYPICAL (PAGE 1 OF 2) UNIT | |
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| 3. DOES THE SUBJECT OF THIS REMEM PROPOSE THE CONDUCT OF TESTS OR EXPERIMENTS NOT DESCRIBED IN THE SAFETY ANALYSIS REPORTS | |
| 4. DOES THE SUBJECT OF THIS REVIEW REQUIRE A CHANCE TO THE PLANT TECHNICAL SPECIFICATIONS? | |
| 5. DOES THE PROPOSED CHANCE, ALTHOUGH NOT DESCRIBED IN THE SAFETY ANALYSIS REPORT, AFFECT ITEMS OR ACTIVITIES THAT ARE DESCRIBED IN THE SAFETY ANALYSIS REPORT? | |
| IF ANY ANSWER IS AFFIRMATIVE, PERFORM AN UNREMEMED SAFETY QUESTION EVALU | ATION: |
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10CFR50.59 EVALUATION FORM - TYPICAL

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Addition of an Isolok Sampler for Spent Res n Discharge to Mobile Solidification Unit

The Solid Waste Processing System collects and processes radioactive spent resins and concentrates from liquid waste streams generated during plant operations, and solidifies these wastes for shipment and burial.

The Solid Waste Processing System consists of the Concentrates Storage Tank which stores evaporator concentrates, Spent Resin Storage Tank which receives spent resins from various plant demineralizers, resin and concentrates transfer pumps which pump spent resins and concentrates streams to vendor solidification equipment.

This change involved addition of an Isolok sampler on the resin discharge piping to sample spent resins for activity prior to dewatering. Sampling was originally done by grab samples which afforded a greater possibility for personnel contamination and exposure. Attached is the work documentation and the 10CFR50.59 Review Determination Form.

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

CONFIGURATION CONTROL PACKAGE COVER SHEET

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SOUTH TEXAS PROJECT CONFIGURATION CONTROL PACKAGE CONTENTS

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| 1. DESIGN CHANGE BASIS | YES X | NO 🗌 |
| 2. FSAR CHANGE REQUEST PACKAGE | YES 🗌 | NO XX |
| 3. MATERIALS SHEET | YES X | NO 🗌 |
| 4. DOCUMENTATION LIST | YES X | NO 🗆 |
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DESIGN CHANGE BASIS

| CCP NO | 1-M-FST-0249 |
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| PAGE 1 | OF 3 |

BASIS OF CHANGE

The installation of an isolok sampler in the MAB truck bay will provide the capability to obtain representative resin samples prior to solidification. This sampling capability is required by the NRC.

DESCRIPTION OF CHANGE

Information provided in the Design Change Basis is provided for information only to enhance the definition of the design change as provided in the attached drawings. Installation and QC verification is to be accomplished in accordance with the drawings only. No XFCR's will be accepted against the Design Change Basis.

- Replace each quick disconnect fitting at the discharge of Valves WS-0036 and WS0042 with 1 1/2" flanges and matching blind flanges per IDCN's to P&ID 9F05048 #1 and Isometric 7M369PWS278, Sht. Alo.
- Modify and extend the existing piping at the discharge of Valve WS-0041 per IDCN's to P&ID 9F05048 #1 and Isometric 7M369PWS278, Sht. A10.
- Modify existing pipe support for Line 1 1/2" WS-1035-WG7 per IDCN to Drawing WS-1035-HF5009. Add new support per IDCN to Drawing WS-1035-HF5010.
- 4. Refer to vendor drawing; Bechtel Log No. BF41002-00006-AXX and weld the flange connections to the adapter as per IDCN #7M369PWS278, Sht. A1C. Assemble sampler and adapter per vendor drawing; Bechtel Log No. BF41002-00001-AXX.
- 5. Mount Isolok Control Station HK-4151 (Ref. vendor drawing; Bechtel Log No. BF41002-00003-AXX) on the truck bay wall per IDCN to drawing 5M15-9Z-46054 and drawing 5Z01-9Z-45080, Sht. 68.

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DESIGN CHANGE BASIS (Cont.)

CCP NO. 1-M-FST-0249

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CCP REV 00

PAGE 2 OF

DESCRIPTION OF CHANGE

5. (Cont'd)

- A. Install vendor supplied yellow and black tubes per vendor drawing; Bechtel Log No. BF41002-00001-AXX from sampler to control station.
- B. No permanent instrument or station air connection shall be installed for the control station.
- 6. Refer to vendor drawings Bechtel Log No. BF41002-00005-AXX and BF41002-00003-AXX to provide the 115 VAC-1PH-60HZ, 1/2 AM electric supply to controller.

NOTE: All existing material-quick disconnecting hose connections and welded pipe stubs - removed by this CCP shall be scrapped



DESIGN CHANGE BASIS (Cont.)

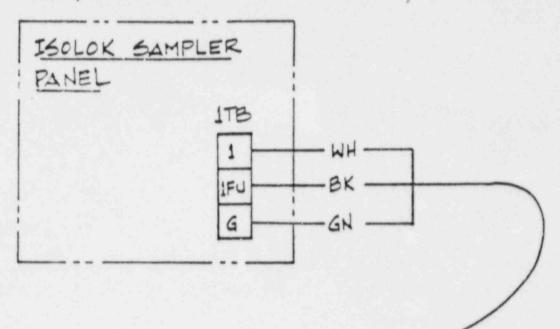
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PAGE 3 OF 3

DESCRIPTION OF CHANGE

7. WIRE FOWER SUPPLY TO CONTROLLER # HK 4151
AS FOLLOWS:



FIELD TO PROVIDE A 6'.0" I 2" LONG CABLE & FABRICATE A MOUNTING BRACKET TO SUPPORT THE COILED CABLE WHEN NOT IN USE.

NOTE: PER SYSTEM ENGINEER NPOD (M. REDCEK) EXTENSION COND TO BE USED FOR POWER IN AREA OF SAMPLER.

CS-AD-2286a (9-16-87)

3 POLE, 3 WIRE GROUNDING PLUG, STRAIGHT BLADE 15 A., 120 V. AC., HUBBELL CAT. Nº 5266-C CR APPROVED EQUAL. BOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

CONFIGURATION CONTROL PACKAGE MATERIAL SHEET

00 No. 1-M-F57-0249

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OF 3 PAGE CHG DESCRIPTION UNIT OF PROCUMEMENT ADD / MATERIAL REQUISTION* REQUESTED CMA MEASURE SIZE OTY MESPONSSILLTY "SPEC Na. No. REV MATERIAL TYPE DELIVERY TYBINGOLDHBI MENT NUMBER ITEM No. REV DATE SAMPLER ASSET FMK H 2" P.O# BF-41002 LOCATION ENG. 0 FM-01953 0 CIOAI5 PIPE 5.6. ASTM A-312 GRADE TP316 OR SIGL SMLS 4'-0" SCH. 40 CONST. 7MPE 9NO4-02 S.S. ASTM A.312 GRADE TP. 316L OR 316 SCH BO 6" 314" 7MPE 9PO4-00-3/4 CONST FLANGES SO . S. FORGED W.N. R.F. ASTM A-182 P-316 OR 2" F. 3166L SCH. 40 CONST 7MF88NDZ-07 2'x3/4 W.O.L SCH 2"x3/4 40 S.S. B.W. ASTM CONST 7MW19ND4-02x06-3 A-403 WIF 316 05 FILL SMIS 150465 FORGED W.N. F.F. ASTM 4-182 F-316 OR 11/2 F-3161 SCH. 40 CONST. FF 7MF CBND4-01-K SOM S S. FORGED BLD FLG F.F. RF 7MFBBNO4-01-12 F316L A-182 F-316 OR 11/2 CONST. BLIND 7MFAB 9D4-01-1/2 GASKETS GPIRAL WOUND CONST. TYPE 304 WIND-5GAA BA BA - 02 x . 175 ING WITH ASESTOS 12" FILLER C.S. OUTER CONST. 5GAABABA- 01-12x-175 RING FLEXITALLIC STYLE . CG" API 601

BOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

CONFIGURATION CONTROL PACKAGE MATERIAL SHEET

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SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

CONFIGURATION CONTROL PACKAGE MATERIAL SHEET

000 No. 1-M-F5T-0249

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| 3 | AND | WEASURE | SIZE | OTY. | PROCUMENTAL | *SPEC No. | 400/ | MATERIAL | - REGLISTION" | REQUESTED |
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SOUTH TEXAS PROJECT CONFIGURATION CONTROL PACKAGE DOCUMENTATION LIST

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000 NO. 1-M-FST-0249 REV. 60

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| IDON HUMBER ISSUE STARTUP SYSTEMS AFFECTED: W50/ | 0ATE 008 NO. 14926 PAGE / OF | 2 |
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| FSAR CHANCE REQUIRED? YES NO | A CONTRACTOR OF THE PARTY OF TH | _ |
| TRUCK COADING : AD | De specific) PROVIDE SAMPLING VALVE TO LIST. | Ar |

DESCRIPTION OF CHANGE

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REV HL&P DATE 14926-001

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| ORIGINATOR | CHECKER | ECS | | PE, | |
| A.J. MUNGUIA | KSnor4 | one? | fin | * ATTIMENT | |

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Date

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INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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JOB NO. 14926 PAGE 2 OF 2

DESCRIPTION OF CHANGE:

| TAG NUMBER | GENERIC 1.D. | FNG | P\$10 | LINE NO. | CLASS | SPEC. | vss. | VENDOR DATA | P.O.1 | P.O. ITEM | VENDOR |
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000 NO. 1-M.FST -249

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INTERIM DRAWING CHANGE NOTICE (IDCN)

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JOB NO. 14926 PAGE | OF 2

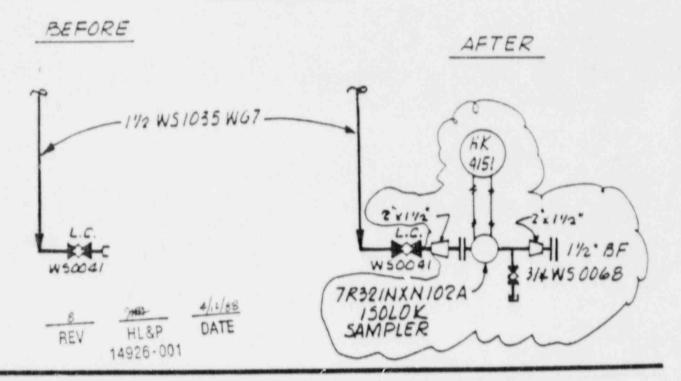
STARTUP SYSTEMS AFFECTED: 1W501

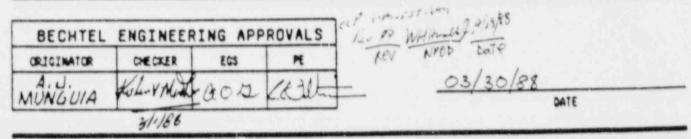
FSAR CHANCE REQUIRED? YES

NO X

REASON FOR CHANGE (Please be specific) ADD 150LOK SAMPLER TO LINE 11/2 WS1035 WG7 FOR SPENT RESIN DISCHARGE REPLACE QUICK DISCONNECT HOSE CONNECTIONS AT THE DISCHARGE SIDE OF VALVES WSOO36 AND WSOO 42 WITH 142" FLAT FACE FLANGES FOR COMPATIBILITY WITH VENDOR CAREA 1524 AND #2176 DESCRIPTION OF CHANGE

COORD. E 1







000 NO. 1-M- FST- 249 REV. \$\$

INTERIM DRAWING CHANGE NOTICE (IDCN) (Cont'd)

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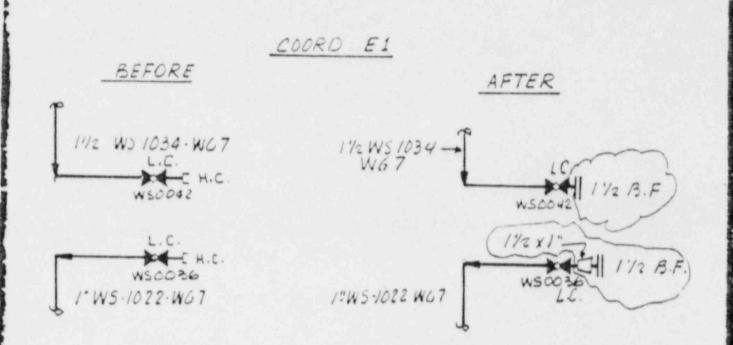
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JOB NO. 14926 PAGE 2 OF 2

DESCRIPTION OF CHANGE:

REPLACE QUICK DISCONNECT HOSE CONNECTIONS WITH FLAT FACE FLANGES





000 NO. 1-M-FST-0459 REV. 00

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308 NO. 14926 PAGE _ OF 3

| FSAR | CHANGE | REQUIRED? | YES | NO A | |
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| - | - | NAME OF TAXABLE PARTY. | PRINCIPAL PRINCI | - | MARKET . |

| | CHANGE (Please be specific) |
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| PROYIDE | SAMPLING AT TRUCK LOADING. |
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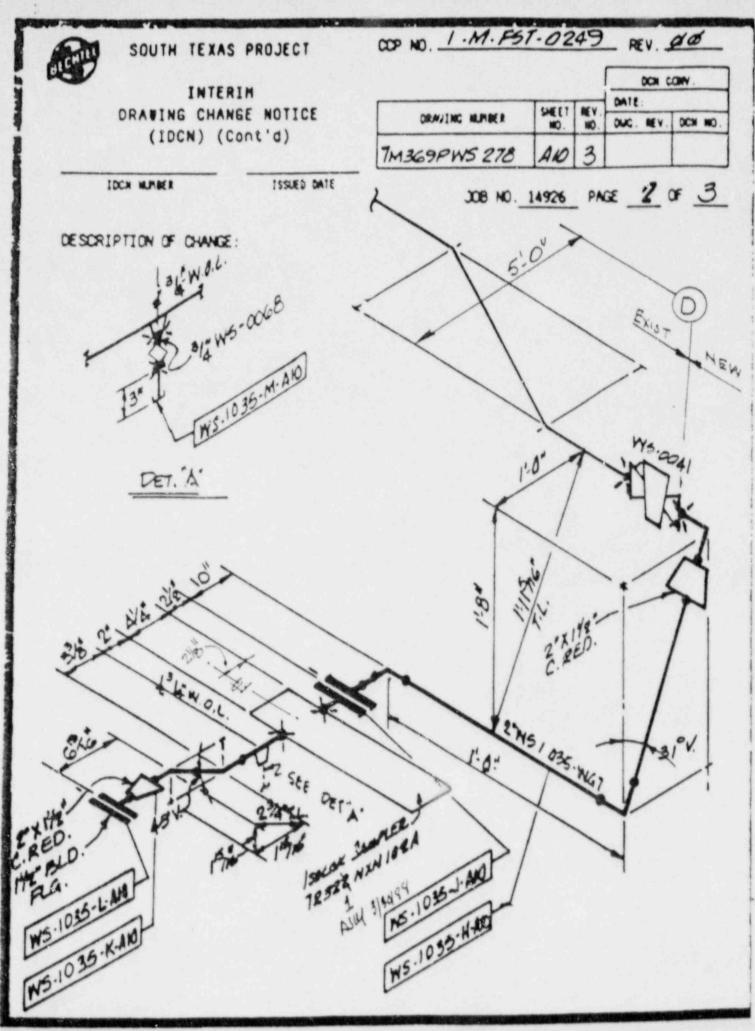
DESCRIPTION OF CHANGE

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DRAWING CHANGE NOTICE
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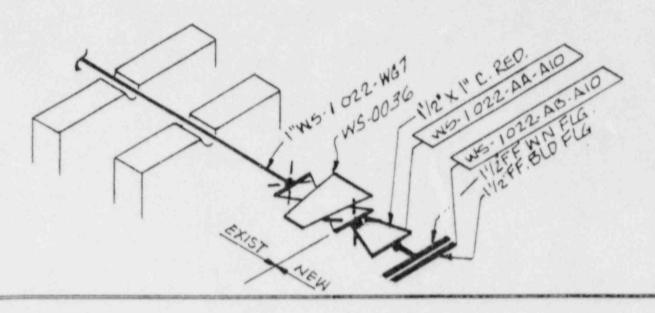
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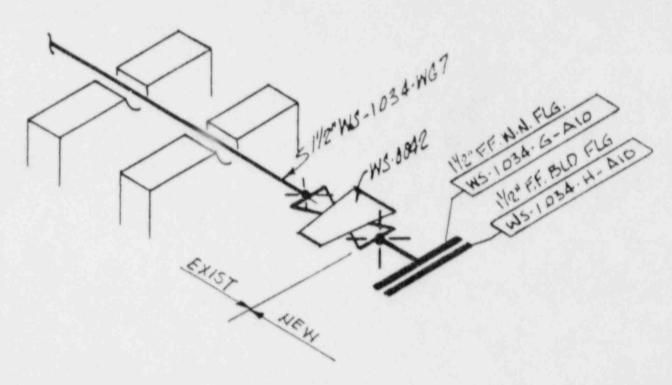
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JOB NO. 14926 PAGE 3 OF 3

DESCRIPTION OF CHANGE:







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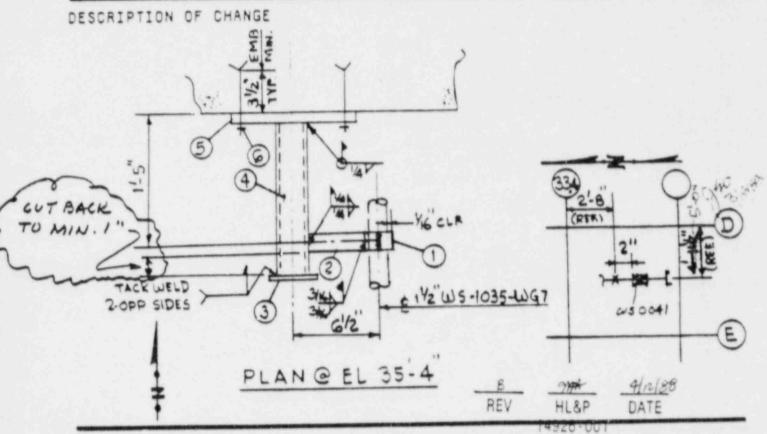
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JOB NO. 14926 PAGE / OF /

FSAR CHANGE REQUIRED? YES ___ NO _

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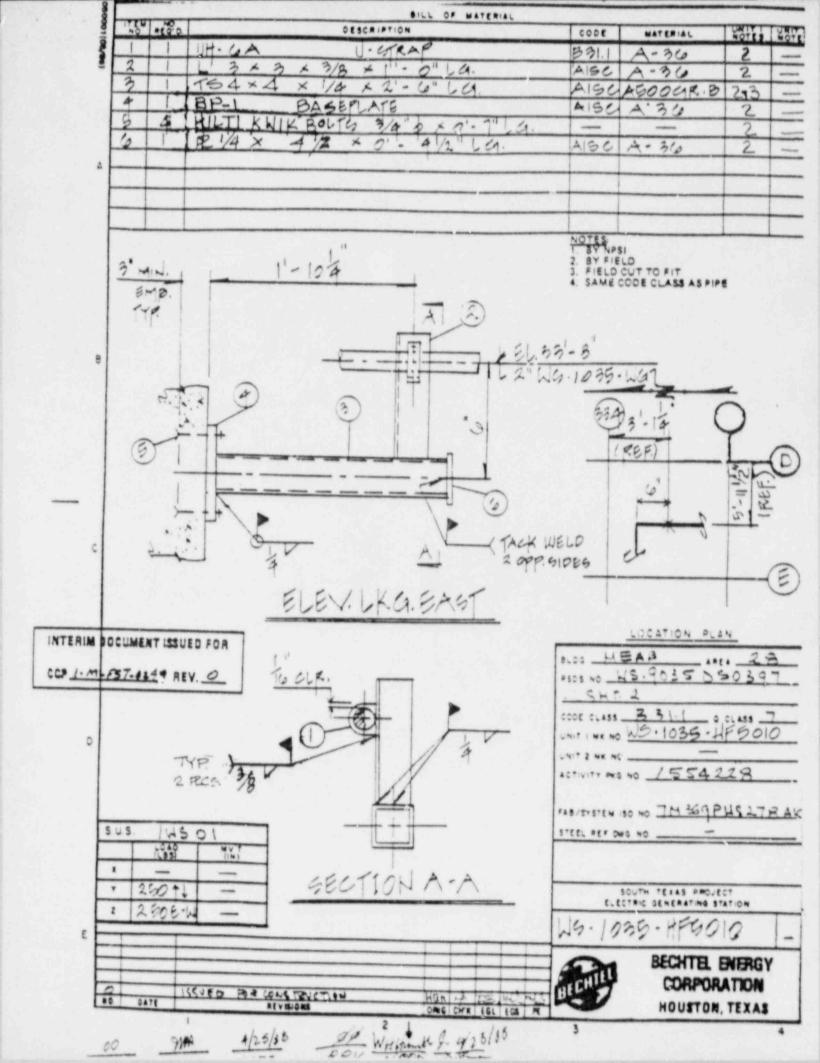


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INTERIM DRAWING CHANGE NOTICE (IDCN)

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| IDCN MURBER STARTUP SYSTEMS AFFECTED: | | JOB NO. 14926 PAGE 1 OF Z. |
|---------------------------------------|--|--|
| FSAR CHANGE REQUIRED? YES | Control of the State of the Sta | |
| TO LINE | (Please be specif | 10) ADD ISOLOK SAMPLER 57 CONTROL STATION NIWS-HK-415 |
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| DESCRIPTION OF CH | ANGE | |

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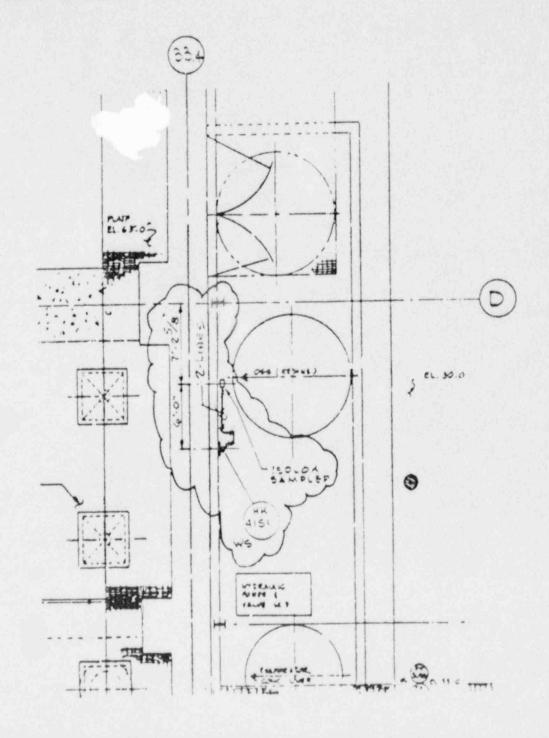
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JOB NO. 14926 PAGE 2 OF 2

DESCRIPTION OF CHANGE:



10CFR50.59 EVALUATION FORM - TYPICAL

(Page 1 of 2)

| DESCRIPTION | | |
|---|-------|----------------|
| Unit # | | |
| PROCEDURE PLANT MODIFICATION TO OTHE | ER | |
| ORIGINATING DOCUMENT NO. CCP 1-M-FST-0249 REV. | 0 | _ |
| TITLE ADDITION OF ISOLOK SAMPLER FOR SPEN | T RE | SW. |
| DISCHARGE, AND REPLACEMENT OF QUICK DISCHOSE CONNECTIONS WITH BLIND FLANGES | ONNE | ECT_ |
| DESCRIPTION PROVIDE SAMPLING CAPABILITY FOR | SOL | 112 |
| WASTES BEING DISCHARGED TO THE MOBILE | | |
| SOCIDIFICATION UNIT | | |
| Does the subject of the review involve a change to the facility as described in the safety analysis report? | | NO DE |
| Does the subject of this review involve a change to the procedures as described in the safety analysis report? | | 200 3.28-88 |
| Does the subject of this review propose the conduct of tests or experiments not described in the safety analysis report? | | IX. |
| Does the subject of this review require a change to the Plant echnical Specification? | | 7-28-88 |
| oes the proposed change, although not described in the safety nalysis report, affect items or activities that are described n the safety analysis report? | | M. |
| This form, when completed, shall be retained for the life of the pl | ant.) | |

10CFR50.59 EVALUATION FORM - TYPICAL

(Page 2 of 2)

If any answer is affirmative, perform an Unreviewed Safety Question evaluation.

If all answers are negative, no Unreviewed Safety Question evaluation is required.

Documentation of this review must be retained with the review package for the duration of the station license.

Note:

"Safety analysis report" includes the FSAR, safety analyses submitted to the NRC in support of their review of the application for an operating license and subsequent amendments to the operating license and other license commitments made to the NRC.

| Documents | Reviewed: | | | | |
|-------------|-------------------------|----------------|-------|-------------------|----------------|
| FSAR | SECT. | 10.9.8,11. | 3 NKC | QUESTIONS | 4 ANSWOKS |
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| Prepared by | QOJIER WELLER Ori | Midal & Sh | etife | / 4/12/88 Date | |
| | 60 | 100 | | Date | |
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DESIGN CHECKLIST (DCL)

| Title Doc.: AD | DITION OF BOLOK | SAMPLER | CCP | Change Document 1-M-FST-024 Page 1 of | 19_ |
|--------------------|--|---|------------------|---|---------------------------|
| Start-up Syste | em <u> W 50 </u> | Title: SOLID WASTE | | | |
| | | tiating change document during to bject Document Control. Referen | | | |
| AFFECTED YES NO | POTENTIALLY AFFECTED DOCUMENTS | AFFECTED DOCUMENTS WHICH REQU (Must provide specific reference of | | OTHER AFFECTED DISC. | TURNED OVER TO HL&P |
| | Licensing Documents FSAR FHAR (Appendix B)* ER Technical Specifications (Plant) Deficiency Evaluation Report (DER) Design Criteria System Descriptions Design Calculations | OCCUMENT NO IOR SECTION) | Acv . | | |
| | ASME Design Specifications Stress Report V 5 Package P ping & Supports) Renetration Seals | | | | |
| **** | Design Drawings Lic Review Required | 68329F05048#1 2M 369 WS278 5H1 A10 WS-1035-HF 5009 WS-1035-HF 5010 4-2-5M15-9-2-46054 | 8 3 B A | TIDEN INCLUDE | |
| Orig QDZ | Jaly Date 3. 29-88 EG | Santualy Date 3 29.8 | PE C.C | Oate | 150/88 |

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION Initiating Change Document No. DESIGN CHECKLIST (DCL) Continued Rev. Page 2 of 2 TURNED AFFECTED DOCUMENTS WHICH REQUIRE REV AFFECTED POT_ TIALLY OTHER SVER (Must provide specific reference no) TO HLAP YES NO AFFECTED LOCUMENTS AFFECTED DISC . DOCUMENT NO. IOR SECTION! REV Y Instrument Index Equipment Index IDEN INCLUOUS Valve Master File 56549760002 2.3 Setpoint List Supplier Dwgs., Inst. Manuals, etc. Configuration Control Packages Design Verification Reports HELBA . ALARA Torr 'Missile Flooding Analyses * Internal Missile *

Equipment Qualification .

Pre Operational Texting Procedures Results

Start up Testing Procedures Results

EE580 Data Base

Other (Specify)

* Programmatic Review Required

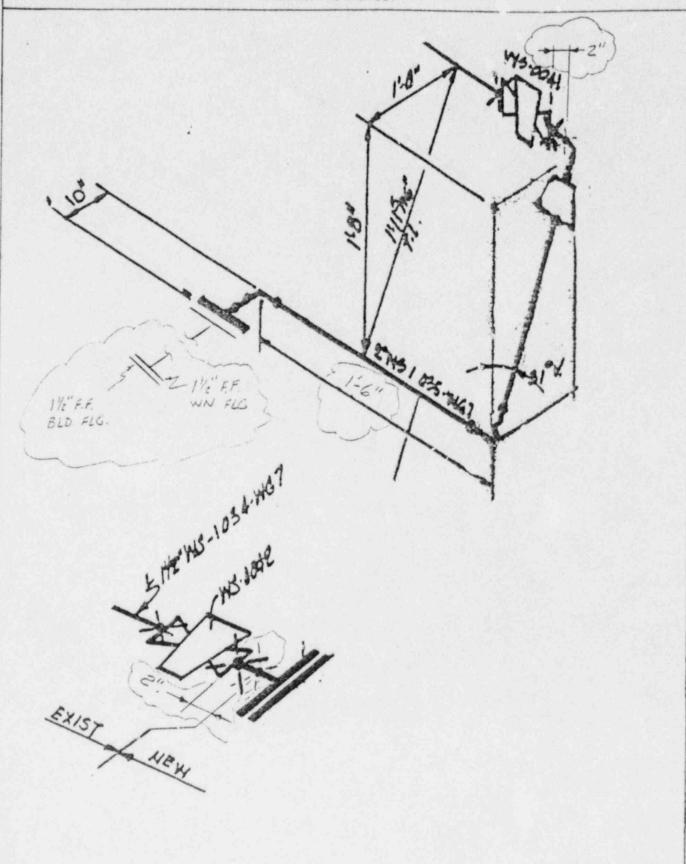
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SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION 1. PAGE 2 OF 2 JOB NO. 14926

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PAGE 1 OF 1

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION DESIGN VERIFICATION FORM

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10CFR50.59 EVALUATION FORM - TYPICAL (PAGE 2 OF 2) UNITA 1 ORIGINATING DOCUMENT NUMBER _ COY-1-M-FST-0749 __ REV. _O . DOCUMENTATION OF THIS REVIEW MUST BE RETAINED WITH THE REVIEW PACKAGE FOR THE DURATION OF THE STATION LICENSE. "SWETY ANALYSIS REPORT" INCLUDES THE FSAR, SAFETY ANALYSES SUBMITTED TO THE NRC IN SUPPORT OF THIER REVIEW OF THE APPLICATION FOR AN OPERATING LICENSE AND SUBSEQUENT AMENOMENTS TO THE OPERATING LICENSE AND OTHER LICENSE COMMITMENTS MADE TO THE HRC. DOCUMENTS REVIEWED: FER HXUP-07365 FS42 (NO APPLICABLE PREMINENT) INTERDISCIPLINE COORDINATION REQUIRED? TYES IN NO F YES, CROLE APPROPRIATE DISCIPLINE, THEN COTAIN THER CONCURRENCE (INITIAL) _____ GYFL ______ SLEC _____ EQ _____ OTHER IMPACT TO OTHER DEPARTMENT? TYES TO NO IF YES, THEN IDENTIFY:____ 4/3/1/23 PREPARED BY: CEDAMU BEC Thaket - Litelen 1 6130188 RESPONSIBLE ENGINEER DATE CONCURRENCE: 18th 12 18 Lan. On 12 July To have 1 6/22/28 MANAGER, SUPPORT ENGINEERING DATE

Addition of an Isolok Sampler for Spent Resin Discharge to Mobile Solidification Unit

A PORC leview was not required for this design change.

Addition of an Isolok Sampler for Spent Resin Discharge to Mobile Solidification Unit

The Solid Waste Processing System Isolok Sampler samples spent resin activity prior to dewatering. This sampler was installed at the request of Chemical Operations to decrease chances of personnel contamination and exposure. Previously, grab samples were taken to determine resin activity. This sampler is located in the new truck bay convenient to transfer operations.

Addition of an Isolok Sampler for Spent Resin Discharge to Mobile Solidification Unit

This change added an Isolok sampler in the new truck bay to sample spent resin prior to dewatering. Previously, sampling was accomplished via grab samples, a method which afforded a greater possibility for personnel contamination and exposure. This change will decrease the chances of contamination and reduce exposure in the sampling process.

This change will result in increased maintenance activities in the Mechanical Auxiliary Building in order to properly maintain the Isolok sampler, this added exposure will be offset by the ability to sample resin in a safer and more time efficient manner. The number of resin transfers and operation of plant equipment will not be affected by this change. No additional exposure will occur as a result of this modification.

Addition of an Isolok Sampler for Spent Resin Discharge to Mobile Solidification Unit

This change added an Isolok sampler which provides a safer method of sampling spent resin prior to solidification. This modification does not change the amount of liquid and gaseous effluents, and solid waste predicted in the License application and amendments.

Addition of an Isolok Sampler for Spent Resin Discharge to Mobile Solidification Unit

This modification added an Isolok sampler on the resin discharge piping to a cilitate sampling of spent resin prior to solidification for shipment and burial. This modification will not result in a change to the expected maximum exposures to a member of the public in the UNRESTRICTED AREA and to the general population as previously estimated in the License application and amendments.

Addition of an Isolok Sampler for Spent Resin Discharge to Mobile Solidification Unit

This item is not applicable. This is the initial reporting period for the South Texas Project Electric Generating Station.

APPENDIX G

STATION PROBLEM REPORT REGARDING PROBLEM REPORT NO. 880197

UNPLANNED RELEASE OF WMT 1E on May 30, 1988

June 13, 1988

Prepared by:
William F. Scott, II
and
Peggy Lofton Travis

Table of Contents

| I. | Description of Event | 1 |
|------|-----------------------------------|---|
| II. | Causes of Event | 2 |
| III. | Analysis of Event | 2 |
| IV. | Corrective Actions | 2 |
| ٧. | Additional Information | 3 |
| | A. Previous Similar Events | |
| | B Failed Component Identification | |

I. Description of Event

At the time of the event on May 30, 1988, described in the following report, the South Texas Project Electric Generating Station was in Mode 5 at 150°F and approximately 400 psig. On this date at the time of the occurrence, Waste Monitor Tanks (WMT) 1D and 1E were in recirculation in accordance with 1PCP13-WL-0005 (Waste Monitor Tank (WMT) Operations). WMT 1F was being filled from the Floor Drain Tank. The surveillance data package for WMT 1D was received at 2255 hours by the Radwaste Control Room for the release of WMT 1D as required by 1PSP07-WL-0001 (Liquid Waste Effluent Rerlease).

At approximately 2300 hours, the Chemical Operations Radwaste Control Room Operator (RWO) accidently placed WMT 1E Pump Discharge Valve Hand Switch (1-WL-FV-5064A) in the "Discharge" position instead of the WMT 1D Pump Discharge Valve Hand Switch (1-WL-FV-5061A) as called for in the procedure. This then allowed for the inadvertent discharge of WMT 1E instead of WMT 1D when the RWO placed the LWPS Discharge Header 3-Way Valve (1-WL-FV-4077) in "Discharge" in accordance with 1PCP13-WL-0005 (Waste Monitor Tank (WMT) Operations). At 2308 hours, after the Radwaste Control Room Operator trainee had observed the incorrect Hand Switch in "Discharge", the RWO secured the discharge of WMT 1E. WMT 1E had been sampled in accordance with 1PSP07-WL-0001 (Liquid Waste Effluent Releases) prior to the discharge; bowever, the surveillance package had not been completed at that time.

Approximately seven hundred ninety-nine (799) gallons of water from WMT 1E were discharged to the Main Cooling Reservoir (MCR). The surveillance data package was subsequently completed in accordance with 1PSP07-WL-0001 (Liquid Waste Effluent Releases) after the discharge had occurred. Samples met the acceptance criteria established in Section 6.0 of 1PSP07-WL-0001 (Liquid Waste Effluent Releases). Analysis results were as follows:

Integrated Release by Nuclide

| | uC! | | uCi |
|-------|----------|-------|----------|
| H-3 | 4.87E+04 | Cr-51 | 3.66E+01 |
| Mn-54 | 5.81E+00 | Fe-59 | 2.30E+00 |
| Co-58 | 2.74E+02 | Co-60 | 5.26E+00 |
| Zr-95 | 3.02E+00 | Nb-95 | 2.19E+00 |

The post-release off-site dose calculations for the inadvertent release of WMT 1E were within Technical Specification limits and were as follows:

Liquid Effluent Doses (mrem)

Whole Body: 0.0000 Highest Organ: 0.0000

Organ: Adult's GI-Track
Receptor: Little Robbins Slough

No adverse impact to the environment occurred. The health and safety of the general public were not adversely affected.

* ..

II. Cause of Event

Approximately 799 gallons of water from WMT 1E were inadvertently discharged to the MCR without the required sampling being completed prior to discharge when the RWO accidentally placed WMT 1E Pump Discharge Valve Hand Switch (1-WL-FV-5064A) into "Discharge" instead of the WMT 1D Pump Discharge Valve Hand Switch (1-WL-FV-5061A) as per 1PCP13-WL-0005 (Waste Monitor Tank (WMT) Operations). A representative sample had been obtained for the 799 gallons that were inadvertently discharged to the MCR although the analysis had not been completed at that time.

The major causative factor for this occurrence was cognitive personnel error. Although the RWO had an approved procedure at hand, it was inadequately followed. The RWO is a utility-nonlicensed operator. Contributing to the major cause was the fact that there were two (2) WMT's in recirculation simultaneously; thereby, allowing less visual distinction between the operating and non-operating trains and facilitating the possibility of an error on the part of the operator.

III. Analysis of Event

The discharge of 799 gallons from WMT 1E without the completed surveillance data package constituted an unplanned release as defined in Regulatory Guide 1.21 and, as such, will be included in the next Semi-Annual Radioactive Effluent Release Report as per OPGP03-ZX-0007 (Preparation of the Semi-Annual Radioactive Effluent Release Report). Although analysis is required "a priori" in Table A3-1 of the Off-Site Dose Calculation Manual as referenced in Technical Specification 4.11.1.1, quantification of the WMT 1E release was completed shortly after discharge.

This occurrence did not involve any plant safety systems; therefore, no safety consequences resulted from the event. Subsequent monitoring of the situation by plant personnel determined that no adverse impacts to the environment occurred nor was the health and safety of the general public adversely affected.

IV. Corrective Actions

Upon discovery, the RWO secured the discharge of WMT IE and notified the Chemical Operations Foreman. The required surveillance data package for WMT IE was completed after the discharge had occurred. No further remedial action is required.

The responsible RWO on watch received individual counseling concerning the unmonitored release. This action was completed June 3, 1988. Crew briefings were initiated and subsequent corrective action items were assigned following a second occurrence on June 7, 1988. They are as follows:

All Chemical Operations personnel shall receive an incident briefing in subsequent shift turnover meetings. This is the responsibility of the Chemical Operations & Analysis Manager, T. E. Underwood, and shall be completed no later than June 20, 1988. Procedures IPCP13-WL-0005 (Waste Monitor Tank (WMT) Operations) and IPSP07-WL-0001 (Liquid Waste Effluent Releases) have been revised to clarify requirements for independent valve lineup verification for all Waste Monitor Tank discharges and to provide additional control of WMT's in recirculation or discharging to minimize the consequences of line-up errors. These revisions were approved by PORC on June 11, 1988, and pending comment resolution will become effective no later than June 17, 1988. This action shall be the responsibility of the CO&A Manager, T. E. Underwood

The aforementioned Division Manager assigned the above corrective action items has agreed to the action and the due date committed.

V. Additional Information

No similar previous events have occurred.

The NPRDS history data base reveals no previous items. The item was not applicable to NPRDS and was not reported in NPRDS.

Supportive documentation is attached.

APPENDIX H

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At approximately 1828 hours on June 7, 1988, with the plant in cold shutdown (Mode 5), an unmonitored release of 1504 gallons of liquid effluent occurred due to an operator (utility - nonlicensed) inadvertently discharging the wrong Waste Monitor Tank (WMT). A subsequent analysis showed the release to be within procedural and Technical Specification requirements. The root cause of the event was personnel error. The responsible individual was counseled, and the incident was reviewed with others involved in radioactive effluent processing. Procedures have been revised to require independent verification of the valve lineup. An investigation into the need for human factors engineering of the related control panels will be performed.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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DESCRIPTION OF EVENT:

On June 7, 1988, with the unit in cold shutdown (Mode 5), Waste Monitor Tanks (WMT) 1D, 1E and 1F were in recirculation in accordance with the WMT Operations procedure. Two independent surveillance data packages were received at 1815 hours by the Radwaste Control Room for the release of WMT 1E as required by the Liquid Waste Effluent Release procedure. At this time, the Liquid Waste Effluent Radiation Monitor (LWERM) was not operable due to internal contamination of the sample chamber. The Liquid Waste Effluent Release procedure and Technical Specification 3.3.3.10 permit effluent releases to continue with the LWERM inoperable, provided that the following actions are taken prior to the release:

- At least two independent samples are analyzed in accordance with Technical Specification 4.11.1.1, and
- At least two technically qualified members of the facility staff independently verify the release rate calculations and discharge line valving.

At 1824 hours, the Mechanical Auxiliary Building Roving Operator (MAB Rover) opened the manual isolation valves on the common discharge line in accordance with the procedure. The Radwaste Control Room Operator (RWO) then inadvertently placed the WMT 1D Pump Discharge Valve handswitch in the "Discharge" position instead of the WMT 1E Pump Discharge Valve handswitch, as required by the procedure. At 1828 hours the RWO placed the discharge header three-way valve handswitch in "Discharge", which resulted in a discharge from WMT 1D. After the MAB Rover returned to the Radwaste Control Room, it was discovered that the wrong tank was lined up for discharge. The discharge was terminated at 1835 hours.

A total of 1504 gallons of water from WMT ID was discharged to the Main Cooling Reservoir. Samples were collected from WMT ID for analysis at 1856 and 1857 hours. Although these samples were obtained after the discharge had occurred, since the tank had been in continuous recirculation since 0838 hours that day, they were representative of the water that had just been discharged from WMT ID. Subsequent analysis of the samples met the acceptance criteria of the Liquid Waste Effluent Release procedure.

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LICENSEE EVEN , REPORT (LER) TEXT CONTINUATION.

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DESCRIPTION OF EVENT (Cont.):

The results of the analysis were as follows:

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| H-3 | 2.09E+03 | microCi | 1:93E+03 | microCi |
| Mn-54 | 3.02E+03 | microCi | 1.73E+03 | microCi |
| Co-58 | 8.60E+01 | microCi | 5.48E+01 | microC1 |
| Zr-95 | 6.83E+00 | microCi | 5.69E+00 | microCi |
| Cr-51 | 2.93E+01 | microCi | 2.32E+01 | microCi |
| Fe-59 | 1.98E+00 | microCi | not detect | |
| Co-60 | 2.32E+00 | microCi | 1.80E+00 | microCi |
| NE-95 | 6.03E+00 | microCi | 5.11E+00 | microCi |
| WC-42 | 0.03E+00 | microC1 | 5.11E+00 | micro |

The post-release off-site dose calculations for the inadvertent release of WMT 1D were within Technical Specification limits and were as follows:

Liquid Effluent Doses (mrem)

Whole Body: 0.0000 Highest Organ: 0.0000

Organ: Adult's GI-Track Receptor: Little Robbins Slough

The NRC was notified of the event at 0010 hours on June 8, 1988.

CAUSE OF EVENT:

The primary root cause of this occurrence was cognitive personnel error. Although the RWO had an approved procedure at hand, it was inadequately followed. The RWO is a utility-nonlicensed operator.

A contributing factor was the fact that there were three WMT's in recirculation simultaneously, allowing less visual distinction between the operating and non-operating trains. This increased the potential for an error on the part of the operator.

The inoperability of the LWERM created delays in the discharge of the WMT's and was partly responsible for the three tanks being in recirculation at the same time. The inoperability of the LWERM also created the condition, as a result of Technical Specification requirements, that required two independent sample analyses and independent verifications of the release rate calculations and flowpath lineup prior to discharge. These requirements had not been met for WMT 1D.

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CAUSE OF FVENT(Cont.):

While the liquid waste effluent release procedure included the sampling and verification requirements for a release with the LWERM out of service, the method for verification of the valve lineup was not clear. The WMT operations procedure did not address these requirements.

ANALYSIS OF EVENT:

1504 gallons of liquid waste was discharged from WMT 1D while the Liquid Waste Effluent Radiation Monitor was inoperable, without first completing two sample analyses and independently verifying the flow path lineup. This constituted a violation of Technical Specifications and, as such, is reportable under 10CFR50.73(a)(2)(i)(B).

Subsequent analysis of the contents of WMT 1D by plant personnel determined that the release was within procedural and Technical Specification limits. Therefore, there was no adverse impact to the environment nor to the health and safety of the public.

CORRECTIVE ACTION:

- The discharge from WMT 1D was terminated immediately upon discovery. Representative samples of the tank contents were obtained, and the required analyses were performed.
- 2. The RWO received individual counseling concerning the unmonitored release.
- Personnel involved in the release of radioactive effluents received a briefing on the incident.
- 4. In order to minimize the potential for valve lineup errors, the WMT operations and liquid waste effluent release procedures have been revised to require independent valve lineup verification for WMT discharges, and to provide additional control of WMT's while in recirculation or discharge. The procedure revision also incorporates an improved method for backflushing the LWERM. This should enhance and increase the availability of the instrument for effluent discharges.
- An investigation into the need for human factors engineering of the related Radwaste Control Room panels will be completed by September 30, 1988.

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ADDITIONAL INFORMATION:

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A similar event occurred on May 30, 1988, when an RWO inadvertently released effluent from the wrong WMT. On that occasion, the LWERM was in service so there was no violation of Technical Specifications, and the incident was not reportable to the NRC. Corrective actions for that event were in progress but had not been completed when the event described in this LER occurred.

HL&P is conducting a separate investigation into the problem of inoperability of the LWERM.

APPENDIX I

NUCLEAR PLANT OPERATIONS Document Control Center

Plant Procedures

OPGP03-ZA-0002 Rev. 10 Page 34 of 34

FIELD CHANGE REQUEST FORM OPGP03-ZA-0002-4 (Page 1 of 1)

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| The Car | | Dat | · 2/25/88 |
| A.C | tho zed Ind/vidual (See | Addendum 3) | |
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| 1000 | SECTION C - FINAL REVI | EW AND APPROVAL | |
| Ante: CoEutz | ant DM review shall be o | ompleted within | 14 calendar day |
| of approval i | n Section B. | | |
| D | (1) 110 | | |
| tarmanent C | hange [] NO [] YES, Auto | matic Incorporat | ion [] NO [] Y |
| patisiactor | y [] NO [] YES | | |
| * Reviewed 'v | | | |
| HELTEMED .A | Carrie | Date | |
| | Cognizant DM | | |
| .75 | | | |

- * This signature is not required for field changes to NON Q Procedures
- ** This signature not required for Division Procedures.

 Plant Manager signature required for Station Procedures, Department Manager signature required for Division Procedures.

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5.0 Procedure

- 5.1 Process Description
 - 5.1.1 Dry Active Waste
 - 5.1.1.1 Dry Active Waste (DAW) is segregated upon collection into trasa receptacles equipped with poly bags, marked as potentially contaminated, potentially noncontaminated or noncompression. When full, the waste is transported to the sorting area and handled in accordance with OPCP14-WS-0010 (Dry Active Waste Collection, Sorting and Segregation).

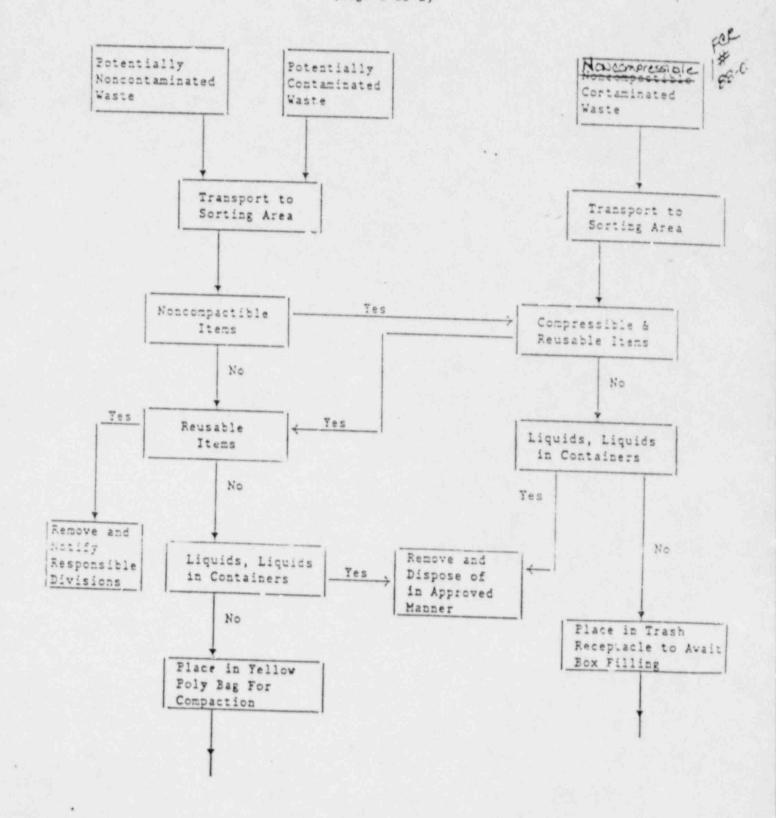
5.1.1.2 In the sorting area, noncompactible items, liquids and containers of liquids shall be removed from the potentially contaminated and the potentially uncontaminated waste. Liquids shall be handled in accordance with OPCP10-ZO-0002 (Disposal of Radioactive Waste Liquid). Reusable items shall be placed in containers and the responsible divisions notified. Noncompactible items shall be placed in noncompactible trash receptacles. The compactible waste is then bagged for compaction.

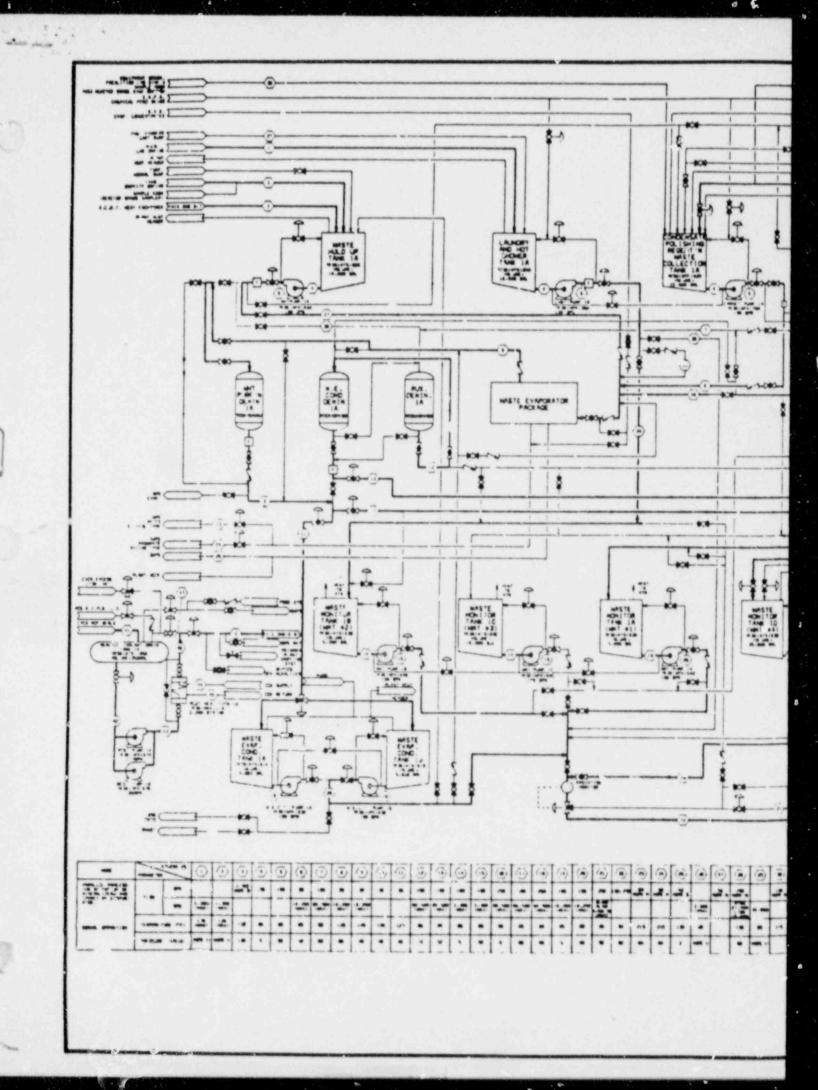
a. Compactible Waste

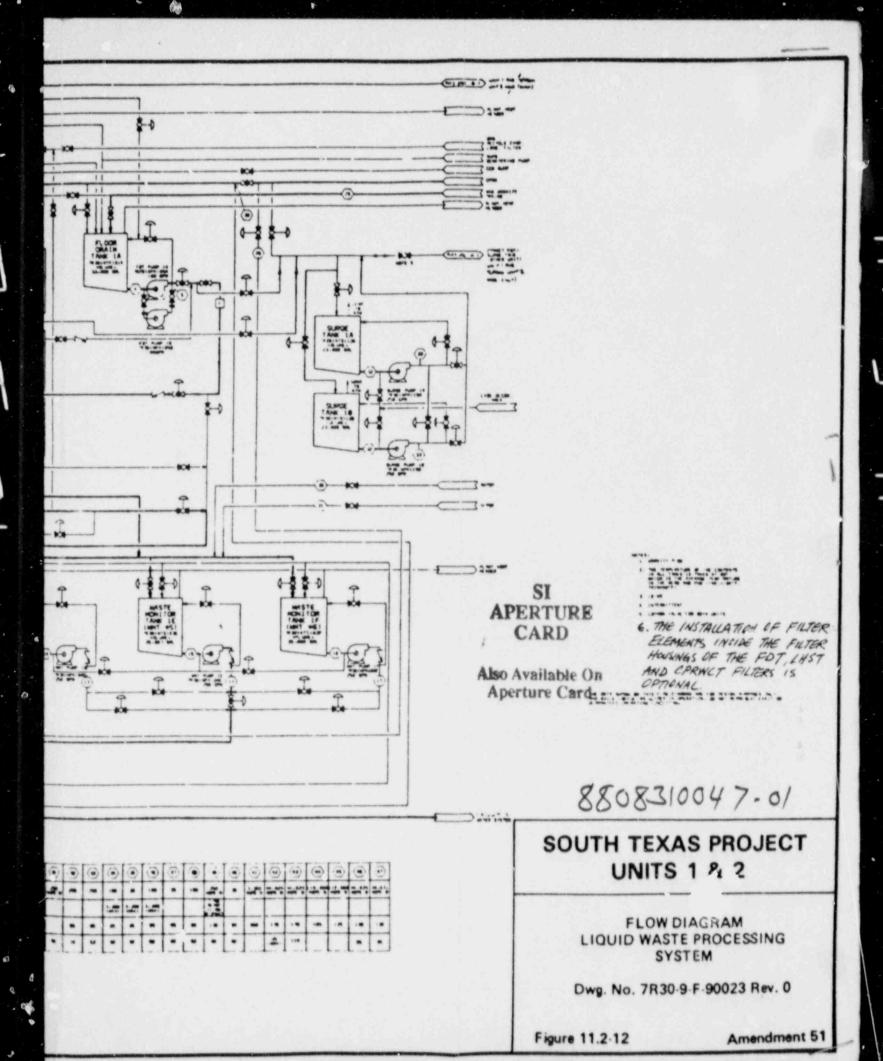
All contaminated or potentially contaminated compactible waste shall be taken to a designated area, compressed into approved strong tight containers, sealed, labeled, weighed and surveyed in accordance with OPCP14-WS-0004 (Dr. Radioactive Waste Operations) and them placed storage in accordance with OPCP, 4-WS-0006 (Radioactive Waste Package Transfer to Storage).

5.1.1.3 In the sorting area, co-pactible and reusable items, liquids and liquids in containers shall be removed from the concempression receptacle. Liquids shall be handled in accordance with OPCP10-ZO-0002 (Disposal of Radioactive Waste Liquids). Reusable items shall be placed in containers and the responsible divisions notified. Compactible waste is then bagged for compaction. Noncompactible waste is placed in trash receptacles.

ADDENDUM 1 DAW FLOW CHART (Page 1 of 2)







August 23, 1988 ST-HL-AE-2769 File No.: GO2 10CFR50.36a

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555

South Texas Project Electric Generating Station
Unit 1
Docket No. STN 50-498
Semiannual Radioactive Effluent
Release Report for the First Half of 1988

Pursuant to the South Texas Project Electric Generating Station (STPEGS) Unit 1 Operating License NPF-76 Appendix A Technical Specification 6.9.1.4 and 10CFR50.36a, attached is the Semiannual Radioactive Effluent Release Report for the first half of 1988. The report covers the period from March 8, 1988 (date of initial criticality) to June 30, 1988.

If you should have any questions on this matter, please contact Mr. C.A. Ayala at (512) 972-8628.

G. E. Vaughn Vice President

18 Vanyl

Nuclear Plant Operations

GEV/CAA/nl

Attachment: Semiannual Radioactive Effluent

Release Report for the First Half

of 1988.

1/1 TE48

ST-HL-AE-2769 File No.: G02 Page 2

cc:

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NOTE: The above copies distributed without the attachment, except as noted by asterisk (*).