

September 16, 1998

Mr. Lew W. Myers
Vice President - Nuclear, Perry
Centerior Service Company
P.O. Box 97, A200
Perry, OH 44081

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING PROPOSED
MODIFICATIONS OF THE SAFETY SETPOINT REQUIREMENTS FOR THE
SAFETY RELIEF VALVES - PERRY NUCLEAR POWER PLANT, UNIT NO. 1
(TAC NO. MA2290)

Dear Mr. Myers:

Your letter of July 13, 1998 (PY-CEI/NRR-2298L), proposed modifications to the technical specifications for the Perry Nuclear Power Plant, Unit No. 1, to increase the present $\pm 1\%$ tolerance on the safety mode lift setpoint for the safety/relief valves (SRVs) to $\pm 3\%$. The staff has initiated its review and has determined that the enclosed request for additional information will be necessary to complete its review.

If you should have any questions regarding this request, please contact me at (301) 415-1364.

Sincerely,

Original signed by:

Douglas V. Pickett, Senior Project Manager
Project Directorate III-3
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Enclosure: As stated
Docket No. 50-440
cc: see next page

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NAME	DPickett		EBarnhill	<input checked="" type="checkbox"/>	DTerao*				
DATE	09/16/98		09/15/98		09/02/98		09/ /98		09/ /98

*See Memo from DTerao to R Bellamy dated 9/2/98

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Project Directorate III-3
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 16, 1998

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Centerior Service Company
P.O. Box 97, A200
Perry, OH 44081

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If you should have any questions regarding this request, please contact me at (301) 415-1364.

Sincerely,

A handwritten signature in cursive script, reading "Douglas V. Pickett".

Douglas V. Pickett, Senior Project Manager
Project Directorate III-3
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Enclosure: As stated

Docket No. 50-440

cc: see next page

L. Myers
Centerior Service Company

cc:

Mary E. O'Reilly
FirstEnergy -- A290
10 Center Road
Perry, OH 44081

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Ohio Department of Health
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Columbus, OH 43266-0118

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Agency
DER 3--Compliance Unit
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Public Utilities Commission
East Broad Street
Columbus, OH 43266-0573

William R. Kanda, Jr., Plant Manager
Cleveland Electric Illuminating Co.
Perry Nuclear Power Plant
P.O. Box 97, SB306
Perry, OH 44081

REQUEST FOR ADDITIONAL INFORMATION
SAFETY/RELIEF VALVE SETPOINT TOLERANCE
PERRY NUCLEAR POWER PLANT, UNIT NO. 1
DOCKET NO. 50-440

1. Uncertainties of analysis parameters should be accounted for in the safety analyses used to justify new limits in the plant Technical Specifications (TSs). Provide a discussion of the Safety/Relief Valve (SRV) setpoint testing instrument accuracy and how this source of uncertainty is accounted for in the licensee's safety analysis associated with the proposed TS SRV setpoint tolerance.
2. Provide verification that the functional capability of all safety-related motor-operated valves has been evaluated for the larger differential pressure loads resulting from the increased SRV setpoint tolerance in accordance with the Generic Letters 89-10 and 96-05 programs. In addition, provide verification that the functional capability of all safety-related air-operated and hydraulically-operated valves has not been adversely affected by the increased SRV tolerance.
3. The NRC Safety Evaluation for topical report NEDC-31753P, dated March 8, 1993, stated that in order to increase the SRV setpoint tolerance from $\pm 1\%$ to $\pm 3\%$, half of the SRVs should be tested every 18 months and all SRVs should be tested within 40 months. The licensee's letter dated July 13, 1998, proposes to continue testing the SRVs less frequently in accordance with the plant inservice testing program. Provide the basis for testing the SRVs less frequently than that approved for the above topical report while increasing the SRV setpoint tolerance to $\pm 3\%$, or revise the proposed TS accordingly.