UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 124
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated November 12, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 124, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective on issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 124

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
3/4 4-14a	3/4 4-14a
3/4 4-14b	3/4 4-14b
3/4 4-14c	3/4 4-14c

REACTOR COOLANT SYSTEM

PRESSURE ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.4.6.3 Reactor Coolant System pressure isolation valves as shown in Table 4.4-3 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4

ACTION:

1. With any Reactor Coolant System Pressure Isolation Valve leakage greater than the limit stated in Table 4.4-3, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of a closed manual or deactivated automatic valve, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.*

2. The provision of Specification 4.0.4 is not applicable for entry into MODE 3 or 4.

SURVEILLANCE REQUIREMENTS

4.4.6.3.1 Leakage testing** of each valve listed in Table 4.4-3 shall be accomplished prior to entering MODE 2 after every time the plant is placed in the COLD SHUTDOWN condition for refueling and prior to returning the valve to service after each maintenance, repair or replacement work is performed; and

4.4.6.3.2 Additional leakage testing of each valve identified by note (d) listed in Table 4.4-3 shall be accomplished prior to entering MODE 2 after each time the plant is placed in COLD SHUTDOWN for 72 hours if testing has not been accomplished in the preceding 9 months.

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* Motor operated valves shall be placed in the closed position and power supplies de-energized.

** To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating compliance within the valve leakage criteria.

TABLE 4.4-3

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

System	Valve No.	Leakage Rates (a) Allowable/Maximum
Loop 1, Cold leg, LHSI	SI-23	≤ 3.0/≤ 5.0 gpm(b)(d)
	SI-12	≤ 3.0/≤ 5.0 gpm(b)(d)
Loop 2, Cold leg, LHSI	SI-24	≤ 3.0/≤ 5.0 gpm(b)(d)
	SI-11	≤ 3.0/≤ 5.0 gpm(b)(d)
Loop 3, Cold leg, LHSI	SI-25	≤ 3.0/≤ 5.0 gpm(b)(d)
	SI-10	≤ 3.0/≤ 5.0 gpm(b)(d)
Loop 1, Hot leg, LHSI	SI-15	≤ 3.0/≤ 5.0 gpm(b)
Loop 2, Hot leg, LHSI	SI-16	≤ 3.0/≤ 5.0 gpm(b)
Loop 3, Hot leg, LHSI	SI-17	≤ 3.0/≤ 5.0 gpm(b)
Common, Hot leg, LHSI	SI-13	≤ 3.0/≤ 5.0 gpm
	SI-14	≤ 3.0/≤ 5.0 gpm
Loop 1, Cold leg, SIACC	SI-48	≤ 5.0/≤ 5.0 gpm(b)
	SI-51	≤ 5.0/≤ 5.0 gpm(b)
Loop 2, Cold leg, SIACC	SI-49	≤ 5.0/≤ 5.0 gpm(b)
	SI-52	≤ 5.0/≤ 5.0 gpm(b)(d)
Loop 3, Cold leg, SIACC	SI-50	≤ 5.0/≤ 5.0 gpm(b)
	SI-53	≤ 5.0/≤ 5.0 gpm(b)(d)
Loop 1, Hot leg, RHS	MOV-RH-700	≤ 5.0/≤ 5.0 gpm(b)
	MOV-RH-701	≤ 5.0/≤ 5.0 gpm(b)
Loop 2, Cold leg, RHS	MOV-RH-720A	≤ 5.0/≤ 5.0 gpm(b)(c)
Loop 3, Cold leg, RHS	MOV-RH-720B	≤ 5.0/≤ 5.0 gpm(b)(c)

(a) At function pressure:

1. Leakage rates less than or equal to 0.5 gpm/inch diameter are acceptable.

2. Leakage rates greater than 0.5 gpm/inch diameter but less than or equal to 5.0 gpm are considered acceptable if the latest measured rate has not exceeded the rate determined by the previous test by an amount that reduces the margin between measured leakage rate and the maximum permissible rate of 5.0 gpm by 50 percent or greater.

~~Order dated April 20, 1981~~
Amendment No. 124

3. Leakage rates greater than 0.5 gpm/inch diameter but less than or equal to 5.0 gpm are considered unacceptable if the latest measured rate exceeded the rate determined by the previous test by an amount that reduces the margin between measured leakage rate and the maximum permissible rate of 5.0 gpm by 50 percent or greater.

4. Leakage rates greater than 5.0 gpm are considered unacceptable.

5. Observed leakage rates shall be adjusted to the function maximum pressure in accordance with ASME XI IWV 3423.

(b) Minimum test differential pressure shall not be less than 150 psid.

(c) Leakage rate continuously monitored during plant operation, no other leakage rate testing required. Leakage rate acceptance criteria shall be as stated in (a) and (b) above and shall be recorded at intervals as noted in paragraph 4.4.6.3.1 as a minimum.

(d) Both surveillances 4.4.6.3.1 and 4.4.6.3.2 are required.