QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JULY, 1988

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

0027H/0061Z

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Verna Koselka and Lynne Deelsnyder, telephone number 309-654-2241, extensions 2240 and 2185.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Unit One began the month of July operating at full power. At 1710, the unit was placed in Economic Generation Control (EGC). The unit operated in EGC or was maintained near full power until July 6. At 1820, a load reduction was taken to 555 MWe due to high river temperatures. On July 7, at 1530, another power reduction was taken to 476 MWe, again attributed to high river temperatures. Load was held constant until July 8, when at 0807, a power increase was taken to 650 MWe. At 1215, a power reduction was taken to 300 MWe due to high river temperatures. Power levels were held constant until July 9. At 1100, a load increase was taken to 625 MWe. However, at 1500, a power reduction was taken to 278 MWe due to high river temperatures. Power levels were held constant until July 12 when a load increase to full power was taken with control rods and recirculation pumps. At 1835, 750 MWe was reached. Power levels were held constant until July 14 when a load reduction was taken at 0025 to perform Main Steam Isolation Valve monthly Surveillance. At 0225, the MSIV testing was completed, and load was held constant until July 15, when at 1100 a reduction was taken due to high river temperatures. At 1325, 350 MWe was reached. At 1440, an ascent to full power was begun. At 1530, 772 MWe was reached and another load reduction was taken due to high river temperatures. On July 16, at 1325 power level was at 200 MWe and held constant until July 20. At 0730, a load increase was taken to 400 MWe and power level was maintained until July 21. At 1015, another load increase was taken to 710 MWe. At 2330, a load reduction 525 MWe was taken per the Chicago Load Dispatcher. Power levels were held constant until July 22. At 0530, an ascent to full power was begun. At 0945, full load was achieved. On July 23, at 2130, power levels were adjusted in preparation for EGC operation. At 2158, the unit was placed in EGC. On July 24, at 0030, EGC was tripped to perform weekly turbine testing. At 0050, several alarms were received in the control room due to a feedwater heater trip. Power level was held at 575 MWe while investigations were made into the heater problems. Once the problem was resolved, power levels were adjusted for EGC operation. At 1740, the unit was placed in EGC. The unit operated in EGC or was maintained near full power for the remainder of the month with minor interruptions to perform routine surveillances.

B. Unit Two

Unit Two began the month of July operating at 413 MWe. At 0100 on July 1, a power ascent to full load was begun with control rods and recirculation pumps. 705 MWe was reached at 1020. Due to high river temperatures, a load reduction to 595 MWe was taken at 1130. Power levels were held constant at the Chicago Load Dispatcher's request until July 5. At 1051, an ascent to full power was begun and at 1216, the unit reached 798 MWe. Load was held constant until July 7 for Traversing In-Core Probe set. At 1500, a power reduction was taken to 520 MWe due to high river temperatures. On July 8, at 0700, a load increase to full power was taken and 795 MWe was reached at 0920. At 1130, a power reduction was taken due to high river temperatures. 351 MWe was reached at 1500. Between July 8 and July 23, several fluctuations in power levels occurred due to high river temperatures. Routine surveillances were performed and normal operational activities occurred during this time. On July 22, at 1645, a Generator/ Exciter Field Ground alarm was received in the control room. At 1705, it was discovered that the relay for the Main Generator Field Ground was tripped and would not reset. At 1745, a load reduction was taken to try and clear the alarm. Attempts to clear the alarm were unsuccessful and on July 23 at 0300, another load reduction to 600 MWe was taken. At 2032, a power reduction to hot standby was begun with control rods and recirculation pumps. On July 24, at 0350, the main turbine was tripped and further investigations were made into the generator ground. At 1519, the mode switch was placed in SHUTDOWN, and the reactor was manually scrammed. The unit was shutdown for the remainder of the month and investigations were made into the main generator ground.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

Technical Specification Amendment Nos. 107 and 104 were issued on June 23, 1988 to Facility Operating Licenses DPR-29 and DPR-30.

These amendments delete the upper tolerance of the Reactor Low-Low Water Level setpoint and correct typographical errors in the bases.

Technical Specification Amendment Nos. 110 and 106 were issued on June 30, 1988 to Facility Operating Licenses DPR-29 and DPR-30.

These amendments revise the trip setpoint for refueling floor radiation monitors used at the Quad Cities Nuclear Power Station, and elevate the level of management authorization required for approving changes to plant operating procedures.

B. Facility of Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

There were no Deviation Reports or Licensee Events Report associated with the Safety Related Work Request Log this month.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST NO.: Q62003

LER NUMBER: 87-019

COMPONENT: System 1000 - Cold load setting reset on spring can, 1009A-M-207.

CAUSE OF MALFUNCTION: The cause of the piping supports to be in noncompliance with the Final Safety Analysis Report is due to a design error. During a 1980 modification as-built configurations were not accurately documented on drawings used for the original piping stress analysis.

RESULTS & EFFECTS ON SAFE OPERATION: The safety of the plant and personnel were not affected during this event. The systems were analyzed and determined to be operable.

ACTION TAKEN TO PREVENT REPETITION: Spring can load settings were revised, a new support was installed and another support was reinforced as corrective action for this event to prevent recurrence. The stations engineering department now requires a dimensional verification be performed on all Safety Related Modifications as part of the new modification program, implemented in April, 1987.

WORK REQUEST NO.: Q62004

LER NUMBER: 87-019

COMPONENT: System 1000 - Cold load setting reset on spring can, 1009A-W-208.

CAUSE OF MALFUNCTION: The cause of the piping supports to be in noncompliance with the Final Safety Analysis Report is due to a design error. During a 1980 modification as-built configurations were not accurately documented on drawings used for the original piping stress analysis.

RESULTS & EFFECTS ON SAFE OPERATION: The safety of the plant and personnel were not affected during this event. The systems were analyzed and determined to be operable.

ACTION TAKEN TO PREVENT REPETITION: Spring can load settings were revised, a new support was installed and another support was reinforced as corrective action for this event to prevent recurrence. The stations engineering department now requires a dimensional verification be performed on all Safety Related Modifications as part of the new modification program, implemented in April, 1987.

WORK REQUEST NO.: Q62005

LER NUMBER: 87-019

COMPONENT: System 2300 - New support installed, 2325-M-201.

CAUSE OF MALFUNCTION: The cause of the piping supports to be in noncompliance with the Final Safety Analysis Report is due to a design error. During a 1980 modification as-built configurations were not accurately documented on drawings used for the original piping stress analysis.

RESULTS & EFFECTS ON SAFE OPERATION: The safety of the plant and personnel were not affected during this event. The systems were analyzed and determined to be operable.

ACTION TAKEN TO PREVENT REPETITION: Spring can load settings were revised, a new support was installed and another support was reinforced as corrective action for this event to prevent recurrence. The stations engineering department now requires a dimensional verification be performed on all Safety Related Modifications as part of the new modification program, implemented in April, 1987.

WORK REQUEST NO.: Q62441

LER NUMBER: 87-019

COMPONENT: System 2300 - Reinforced support on line 2-2310-4".

CAUSE OF MALFUNCTION: The cause of the piping supports to be in noncompliance with the Final Safety Analysis Report is due to a design error. During a 1980 modification as-built configurations were not accurately documented on drawings used for the original piping stress analysis.

RESULTS & EFFECTS ON SAFE OPERATION: The safety of the plant and personnel were not affected during this event. The systems were analyzed and determined to be operable.

ACTION TAKEN TO PREVENT REPETITION: Spring can load settings were revised, a new support was installed and another support was reinforced as corrective action for this event to prevent recurrence. The stations engineering department now requires a dimensional verification be performed on all Safety Related Modifications as part of the new modification program, implemented in April, 1987.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

There were no Deviation Reports or Licensee Event Reports associated with the Safety Related Work Request Log for the month of July for Unit 1 or Unit 2.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

UNIT 1

Licensee Event Report Number	Date	Title of Occurrence
88-009	6-10-88	HPCI Inoperable due to hot aux. oil pump motor.
88-010*Now listed as 88-024 under Unit 2	6-21-88	1/2 D.G. Autostart
88-011	6-25-88	RCIC Inoperable

UNIT 2

88-014	4-20-88	Deviation from Process Control Program
88-015	5-30-88	ESF Actuation due to Blown Fuse
88-016	6-1-88	Partial Group 2 Isolation - Blown Fuse
88-017	6-7-88	MSIV Air Line Hanger not meeting FSAR rqeuirements
88-018	6-9-88	Spurious Group III Isolation/ ESF two times
88-019	6-11-88	ESF Actuation - when fuse in 902-40 panel removed
88-020* Reclassified as nonreportable.	6-19-88	TIP #3 Failed to Withdraw on a Group II Isolation
88-021*Now listed as 88-020	6-20-88	ECCS Initial Signal due to Valving error
88-022*Now listed as 88-023	6-22-88	Failure of Bus 28/29-5
88-023*Now listed as 88-021	6-26-88	HPCI Isolation
88-024*Was listed as 88-010 under Unit 1	6-21-88	1/2 D.G. Autostart

*These have been renumbered since the report in June due to reclassifying of LER 88-020 under Unit 2.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT One

DATE August 9, 1988

COMPLETED BY Lynne Deelsnyder

TELEPHONE 309-654-2241

OPERATING STATUS 0000 070188

1. REPORTING PERIOD: 2400 073188 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY (MWe-Net): 769
   DESIGN ELECTRICAL RATING (MWe-Net): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	744.0	4862.2	113926.5
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3421.9
7. HOURS GENERATOR ON LINE	744.0	4608.2	110066.4
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	909.2
9. GROSS THERMAL ENERGY GENERATED (MWH)	1435960	10464382	233805139
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	450157	3389521	75813023
11. NET ELECTRICAL ENERGY GENERATED (MWH)	427044	3231321	71134610
12. REACTOR SERVICE FACTOR	100.0	95.1	80.1
13. REACTOR AVAILABILITY FACTOR	100.0	95.1	82.5
14. UNIT SERVICE FACTOR	100.0	90.2	77.4
15. UNIT AVAILABILITY FACTOR	100.0	90.2	78.0
16. UNIT CAPACITY FACTOR (Using MDC)	74.6	82.2	65.1
17. UNIT CAPACITY FACTOR (Using Design MWe)	72.7	80.1	63.4
18. UNIT FORCED OUTAGE RATE	0.0	6.8	5.5

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT Two

DATE August 9, 1988

COMPLETED BY Lynne Deelsnyder

TELEPHONE 309-654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 070188 / 2400 073188 GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY (MWe-Net): 769
   DESIGN ELECTRICAL RATING (MWe-Net): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	567.3	3008.3	107665.4
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	2985.8
7. HOURS GENERATOR ON LINE	555.8	2941.7	104477.0
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	702.9
9. GROSS THERMAL ENERGY GENERATED (MWH)	968550	6224580	223595147
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	301041	2001600	71559384
11. NET ELECTRICAL ENERGY GENERATED (MWH)	284596	1907322	67467347
12. REACTOR SERVICE FACTOR	76.3	58.9	76.2
13. REACTOR AVAILABILITY FACTOR	76.3	58.9	78.3
14. UNIT SERVICE FACTOR	74.7	57.6	73.9
15. UNIT AVAILABILITY FACTOR	74.7	57.6	74.4
16. UNIT CAPACITY FACTOR (Using MDC)	49.7	48.5	62.1
17. UNIT CAPACITY FACTOR (Using Design MWe)	48.5	47.3	60.5
18. UNIT FORCED OUTAGE RATE	25.3	10.3	8.4

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254
UNIT ONE
DATE August 3, 1988
COMPLETED BY Lynne Deelsnyder
TELEPHONE 309-654-2241

MONTH JULY, 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	733	17	203
2	716	18	229
3	704	19	227
4	749	20	327
5	727	21	540
6	672	22	647
7	525	23	745
8	413	24	724
9	314	25	698
10	277	26	680
11	357	27	758
12	482	28	766
13	705	29	759
14	760	30	703
15	638	31	738
16	294		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265
UNIT TWO
DATE August 3, 1988
COMPLETED BY Lynne Deelsnyder
TELEPHONE 309-654-2241

MONTH JULY, 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	537	17	214
2	520	18	236
3	491	19	230
4	521	20	325
5	667	21	542
6	755	22	639
7	655	23	676
8	485	24	17
9	392	25	-10
10	549	26	- 9
11	466	27	- 8
12	578	28	- 7
13	716	29	- 8
14	754	30	- 9
15	635	31	-10
16	296		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

ID/5A

DOCKET NO. 50-254

UNIT NAME QUAD-CITIES UNIT ONE

DATE August 9, 1988

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

COMPLETED BY L. DEELSNYDER

REPORT MONTH JULY, 1988

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
88-13	880707	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures
88-14	880708	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures
88-15	880709	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures
88-16	880716	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures
88-17	880717	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures

APPROVED
AUG 16 1982
QCOSR

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 50-265

UNIT NAME QUAD-CITIES UNIT TWO

COMPLETED BY L. DEELSNYDER

DATE August 9, 1988

REPORT MONTH JULY, 1988

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
88-7	880708	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures
88-8	880709	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures
88-9	880716	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures
88-10	880717	F	0.0	H	5	-------	ZZ	ZZZZZZ	Power Reduction Taken Due to High River Temperatures
88-11	880724	F	188.2	A	2	-------	HA	GENERA	Reactor Manually Scrammed Due to Main Generator Ground Fault

APPROVED
AUG 16 1982
QCOSR

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. MAIN STEAM RELIEF VALVE OPERATIONS

There were no Main Steam Relief Valve Operations for the reporting period.

B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 psig.

RESULTS OF SCRAM TIMING MEASUREMENTS

PERFORMED ON UNIT 1 & 2 CONTROL

ROD DRIVES, FROM 1-1-88 TO 12-31-88

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
		0.375	0.900	2.00	3.5	7 sec.	
5-16-88	89	0.29	0.67	1.45	2.56	3.07 (G-11)	Unit 1, Hot Scram Timing A Sequence
6-17-88	2	0.30	0.68	1.49	2.55	2.60 (P-7)	Unit 1, G-7 and P-7 Hot Scram Timing (P-7 had probe replaced.)
6-27-88	177	0.31	0.68	1.44	2.52	2.96 (H-5)	Unit 2 Hot Scram Timing
7-2-88	1	0.29	0.63	1.35	2.36	2.36 (P-11)	Unit 2, P-11 Hot Scram Timed after accumulator work.

0027H/0061Z

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 9 Cycle: 10

2. Scheduled date for next refueling shutdown: 6-10-89

3. Scheduled date for restart following refueling: 9-2-89

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   MARCH 10, 1989

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1773

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2008

APPROVED

APR 20 1978

Q.C.O.S.R.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

1. Unit: Q2 Reload: 9 Cycle: 10

2. Scheduled date for next refueling shutdown: 12-2-89

3. Scheduled date for restart following refueling: 3-3-90

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:
   SEPTEMBER 2, 1989

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
   NONE AT PRESENT TIME.

7. The number of fuel assemblies.
   a. Number of assemblies in core: 724
   b. Number of assemblies in spent fuel pool: 1475

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
   a. Licensed storage capacity for spent fuel: 3897
   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2008

APPROVED
APR 20 1978
Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center

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RAR-88-36

July 27, 1988

Director of Nuclear Reactor Regulations
U. S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, D. C. 20555

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of July, 1988.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

R. A. Robey
Services Superintendent

RAR/vmk

Enclosure

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