



Northern States Power Company

Prairie Island Nuclear Generating Plant

1717 Wakonade Dr. East  
Welch, Minnesota 55089

September 4, 1998

10 CFR 50.59

U S Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

**PRAIRIE ISLAND NUCLEAR GENERATING PLANT**

Docket Nos. 50-282 License Nos. DPR-42  
50-306 DPR-60

**License Amendment Request Dated September 4, 1998  
Reactor Coolant Vent System**

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Attached is a request for a change to the Technical Specifications, Appendix A of the Operating Licenses, for the Prairie Island Nuclear Generating Plant. Northern States Power Company submits this request in accordance with the provisions of 10CFR50.90. This amendment request proposes to clarify the surveillance requirements and limiting conditions for operability for the Reactor Coolant Vent System. This submittal also satisfies a recent corrective action commitment made in LER 1-98-09.

The current Technical Specifications require that the reactor coolant vent valve testing should be performed at the end of each refueling outage to demonstrate vent system operability. Because the solenoid operated vent valves have on occasion failed their surveillance testing and required repairs, if the valve surveillance testing is performed at the end of the outage this situation has the potential to delay a unit's return to service.

Unit 2 will begin a refueling outage on November 7, 1998 that is scheduled to continue until December 22, 1998. To reduce the probability that reactor coolant vent valve surveillance testing will delay the return to service of this unit after its refueling outage,

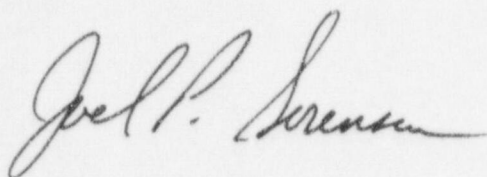
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NSP requests that this license amendment is granted prior to the completion of this Unit 2 outage.

Exhibit A contains a description of the proposed change, the reasons for requesting the change, the supporting safety evaluation, and the significant hazards determination. Exhibit B contains current Prairie Island Technical Specification pages marked up to show the proposed change. Exhibit C contains the revised Prairie Island Technical Specification pages incorporating the proposed change.

If you have any questions related to this license amendment request, please contact John Stanton at 612-388-1121 x4083.



Joel P. Sorensen  
Plant Manager  
Prairie Island Nuclear Generating Plant

Attachments:

Affidavit

Exhibit A, Evaluation of Proposed Changes to the Technical Specification  
Appendix A of Operation License DPR-42 and DPR-60.

Exhibit B, Marked Up Technical Specification Pages

Exhibit C, Revised Technical Specification Pages

c: Regional Administrator -- III, NRC  
NRR Project Manager, NRC  
Senior Resident Inspector, NRC  
Kris Sanda, State of Minnesota  
J E Silberg