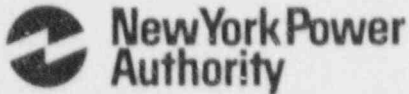


James A. FitzPatrick  
Nuclear Power Plant  
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Radford J. Converse  
Resident Manager

August 8, 1988  
JAFP-88-0744

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Mail Stop Pl-137  
Washington, D.C. 20555

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT  
DOCKET NO. 50-333  
EMERGENCY RESPONSE FACILITIES INSPECTION NO. 88-07  
RESPONSE TO OPEN ITEMS

REFERENCE: 1. NRC letter, R. R. Bellamy to R. J. Converse  
dated July 7, 1988 regarding NRC Inspection  
50-333/88-07, Emergency Response Facilities

Dear Sir:

The NRC transmitted inspection Report No. 50-333/88-07 as an attachment to Reference 1. This inspection report summarized the results of an NRC inspection of the FitzPatrick Emergency Response Facilities (ERF) during the week of May 16-20, 1988.

The report identified three open items. This letter describes the Authority's corrective action along with an implementation schedule for each.

The three open items identified are restated below:

1. Dose calculation procedures do not contain a method for calculating offsite doses resulting from a real time accident in which the source term is leakage through the drywell boundary and whether any other credible, unmonitored accident release pathways exist for which similar methods should be developed (50-333/88-05-01).
2. The validation and verification documentation for the Class A Dose Assessment Model is incomplete (50-333/88-05-02).
3. There is no formalized method for controlling and documenting the maintenance of the dose assessment model (i.e. MMRAS and IDAC) (50-333/88-05-03).

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TO: U.S.N.R.C.  
FROM: R. CONVERSE  
SUBJECT: INSPECTION NO. 88-07

August 8, 1988  
JAFP-88-0744  
Page 2

Response to Open Items (Corrective Action and Schedule)

1. A plant procedure will be developed for determining radiological source terms for use in dose calculations based on drywell radiation monitor readings and the maximum allowable containment leak rate as specified in the FitzPatrick Technical Specifications. This procedure will also include steps to identify credible, unmonitored accident release pathways.

This procedure will be implemented by January, 1989.

2. A Validation and Verification program will be developed for the computer programs that are part of the FitzPatrick Meteorological Monitoring and Radiological Dose Assessment System (MMRAS). A methodology similar to that used to validate the FitzPatrick Offsite Dose Calculation Manual (ODCM) will be used. The validation and verification methodology will be documented in a program report. The program will also develop and document a detailed test plan.

This program will be complete by May, 1989.

3. A plant procedure for the control and maintenance of computer software is currently under development. This procedure will be implemented to control revisions to both MMRAS and IDAC (Interim Dose Assessment System) computer programs.

This procedure will be implemented for MMRAS and IDAC by May, 1989.

*W. Converse for*  
RADFORD J. CONVERSE

RJC/AHZ/mam

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