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LICENSEE	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION					
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER	16)	PAGE (3)		
Peach Bottom Atomic Powe	er	YEAR SEQUENTI	AL ALVIEION NUMBER			
Station - Unit 3	0 15 10 10 10 1 21 7 18	815-011	3-010	012 OF 0		

Unit Conditions Prior to Event:

Reactor Power 0% Refuel Mode with Reactor Vessel Head Removed

Description of the Event:

On November 17, 1985 at 1830 hours when the control room operator proceeded to place the Residual Heat Removal system in the shutdown cooling mode, the suction valves would not open. Investigation revealed that on November 15, 1985 at approximately 1315 hours when a plant operator applied a maintenance permit to the Unit 3 Primary Containment Isolation System (PCIS), he removed the wrong fuse. The permit required removal of fuse F2 on terminal strip BB in panel 30C42. However, the operator removed fuse F3 instead of F2 due to a misleading label.

Fuse F3 is in the electrical feed to a relay in the PCIS logic circuitry for the isolation of the shutdown cooling suction valves, MO-3-10-17 and MO-3-10-18, and head spray isolation valves, MO-3-10-32 and MO-3-10-33, when reactor vessel pressure is greater than 75 psig. By removing fuse F3, the relay in the PCIS logic was de-engergized and an isolation signal for the shutdown cooling suction valves was initiated. Because the MO-3-10-17, MO-3-10-18, MO-3-10-32, and MO-3-10-33 valves were in the closed position at the time, no valve movement occurred. With this isolation initiated and unable to be reset because the fuse was removed, the control room operator could not open the suction valves when he attempted to establish shutdown cooling.

This isolation is part of a Group II isolation; however, in this case only the MO-3-10-17, MO-3-10-18, MO-3-10-32, and MO-3-10-33 valves were affected due to the location of this fuse in the circuitry.

The EIIS code for the affected system is JM, PCIS.

LICENSEE EVENT	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION							U.S. MUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES 8/31/86					
FACILITY MAME ())	DOCKET NUMBER (3)						PAGE 131						
Peach Bottom Atomic Powe	-	VEA8 .		Maulatias Alvelos									
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Consequences of the Ev	ent:												

Fuse F2 had been removed previously by another permit. Thus, the equipment that should have been de-energized by this permit was de-energized.

Refueling of the Unit 3 vessel was completed on November 14, 1985, only one day before the event (three days before discovery). At the time of this occurrence, decay heat removal was provided by the fuel pool cooling system. Circulation between the reactor vessel and the fuel pool was provided by temporary pumps and the reactor cavity to fuel pool slot until the pumps were removed on November 15, 1985. Also, the decay heat load in the reactor vessel was low because the unit had been shutdown since July 15, 1985 and approximately 35% of the fuel bundles had been replaced with new bundles during the fuel reloading. Shutdown cooling operation was not needed at the time of the discovery, but was being placed in service to prepare for installation of the fuel pool gates, which prevent flow between the fuel pool and the reactor cavity. Therefore, decay heat removal was not in jeopardy because of this event.

Cause of Event:

The non-licensed plant operator removed the wrong fuse because the label strip identified the wrong fuse as F2. The strip of fuse labels, which is located behind the row of fuses, had slipped down. Because fuse F3 is located directly below fuse F2, fuse F3 appeared to be fuse F2. Consequently, the operator unknowingly committed a personnel error.

Corrective Actions:

Fuse F3 was installed at approximately 2000 hours on November 17, 1985 and the isolation was reset. Subsequently, the labels were moved up to their correct position.

The Independent Safety Engineering Group is investigating this incident to assess its impact on the plant and determine the extent to which this type of problem exists elsewhere in the

LICENSEE EVENT REPO	RT (LER) TEXT CONTINU	UATION APPROVED	GULATORY COMMISTIC DWS NO. 3150-0104 DIALS		
Peach Bottom Atomic Power	DOCKET NUMBER []	LEA NUMBER (8) VEAR SEQUENTIAL ALVILON NUMBER NUMBER	PAGE (3)		
Station - Unit 3	0 15 10 10 10 12 17 18	815 - 01213-010	014 05 0 1		
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and Primary Containment I evaluated to determine th Recommendations to preclu the investigation finding revealed that the use of relabeling may be appropri- possible solutions will b	Isolation System, Emerg Isolation System ne potential for ide such an even gs. Preliminary adhesive on the riate long-term be evaluated.	fuse labeling and a similar occurr t will be made ba investigation has existing labels solutions. These	e being cence. ased on as or		
A revision to this LER windeveloped and accepted. February 17, 1986.	ill be submitted We expect to su	when recommendat bmit the revised	ions are LER by		
Previous Similar Occurre	nces:	Sec. Sec.			
None.					
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PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

December 17, 1985

Docket No. 50-278

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555

> SUBJECT: Licensee Event Report Peach Bottom Atomic Power Station - Unit 3

This LER concerns an inadvertent actuation of a portion of the Primary Containment Isolation System with Unit 3 shutdown for refueling. While in the closed position the shutdown cooling suction valves and the bead spray isolation valves received an isolation signal.

Docket 50-278
3-85-23
00
November 15, 1985
November 17, 1985
December 17, 1985
Peach Bottom Atomic Power Station RD 1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,

20 Helinil

W. T. Ullrich Superintendent Nuclear Generation Division

> IE22 11,

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC T. P. Johnson, NRC Resident Inspector