



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JUL 17 1985

850719

TSRG 30

MEMORANDUM FOR: Dennis M. Crutchfield, Assistant Director
for Safety Assessment, DL
Thomas M. Novak, Assistant Director
for Licensing, DL
FROM: R. Wayne Houston, Assistant Director
for Reactor Safety, DSI
SUBJECT: RIVER BEND STATION TECHNICAL SPECIFICATION CHANGE
(ONE TIME DEVIATION)

Plant Name: River Bend Station
Docket No.: 50-458
Licensing Stage: OL
Responsible Branch: LB #2, DL
Project Manager: S. Stern
Due Date: N/S
Review Status: Complete

By a letter dated July 11, 1985, Gulf States Utilities Company requested a one time deviation from Section 3.6.2.7 of the Technical Specification for River Bend Station.

Limiting Condition For Operation (LCO) 3.6.2.7 states that the drywell vent and purge system supply and exhaust valves shall be sealed closed during Operational Conditions 1, 2 and 3. This LCO was put in place since the 24" drywell vent and purge valves are not qualified to close under accident conditions in the drywell. The applicant request is to allow the use of the drywell vent and purge system while in Operational Conditions 2 and 3 for up to a cumulative time not to exceed twenty-four hours provided that 5% of rated thermal power is not exceeded.

The proposed change is needed to eliminate the accumulation of ammonia during periods when personnel entry and access to the drywell is necessary to perform various startup test procedures. The applicant anticipates this effort will be completed in approximately a ten day period from the time of initial heatup to rated temperature and pressure for these tests.

The applicant stated that during the hot functional test at River Bend Station, ammonia levels in excess of 50 ppm were released from the fiberglass insulation in the drywell. The applicant further stated that it is anticipated that during the initial heatup testing, offgassing of ammonia will occur since they have replaced some of the drywell insulation and the analysis indicated that

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D. Crutchfield
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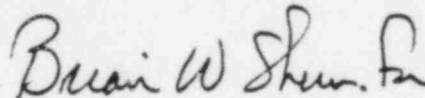
additional offgassing is anticipated from the insulation that has undergone this hot functional test.

Based on our evaluation of the applicant's proposed limited use of the drywell purge for control of ammonia at low power (up to 5% of rated power), we conclude that this limited use does not endanger the safety and health of the public and should be approved. The rationale for accepting this Technical Specification deviation is summarized below.

Since the time of operation at low power, when the need might exist to control the ammonia concentration is short, and since most of the ammonia offgassing will occur at power less than 5% of rated power, the fission product inventory will be low.

The lower fission product inventory results in reduced decay heat following shutdown. This reduced decay heat significantly increases the time available to the operator for any corrective action in the event it is required, and decreases AC power requirements of the minimum necessary decay heat removal system components. Since the proposed use of the drywell purge system is less than 1% of time, the probability of an accident occurring during this time is low. This low probability event along with the lower fission product inventory make the consequences of transients and accidents during this limited time negligible. Therefore, the staff finds the limited use of non qualified purge vent isolation valves to be acceptable.

Enclosed is a marked up copy of Technical Specification 3.6.2.7.



R. Wayne Houston, Assistant Director
for Reactor Safety
Division of Systems Integration, NRR

Enclosure: As stated

cc: R. Bernero
H. Thompson
W. Butler
R. Benedict
M. Haughey
S. Stern

CONTAINMENT SYSTEMS

DRYWELL VENT AND PURGE

LIMITING CONDITION FOR OPERATION

3.6.2.7 The drywell vent and purge system supply and exhaust valves shall be sealed closed*. Sealed closed barriers include blind flanges and sealed closed isolation valves which may be closed manual valves, closed remote-manual valves, and closed automatic valves which remain closed after a loss-of-coolant-accident. Sealed closed isolation valves should be under administrative control to assure that they cannot be inadvertently opened. Administrative control includes mechanical devices to seal or lock the valve closed, or to prevent power from being supplied to the valve operator.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With the drywell vent and purge system supply or exhaust valves open in OPERATIONAL CONDITIONS 1, 2, or 3, and immediately close the drywell vent and purge system valves and be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the next 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.7 At least once per 31 days, verify the drywell vent and purge system supply and exhaust valves to be sealed closed.*

* The drywell vent and purge system may be used when venting ammonia in excess of 12 ppm for a cumulative time not to exceed 24 hours provided that:

- 1 - THERMAL POWER does not exceed 5 percent RATED THERMAL POWER.
- 2 - The 24-inch supply and exhaust isolation valves shall be limited to an opening of 50°.
- 3 - At least once per shift, verify the drywell vent and purge system supply and exhaust valves position and the cumulative time.



FORM EG&G-1481
(Rev. 02-79)

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AUDIT OF RIVER BEND STATION
TECHNICAL SPECIFICATIONS

MAY 1985

REVIEW PERFORMED BY

D. E. BAXTER
S. J. BRUSKE

FIN A6824--Power Reactor Technical Specification Evaluation

NRC TSRG Lead Engineer - Carl Schulten, NRR-DL

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

AUDIT OF RIVER BEND STATION TECHNICAL SPECIFICATIONS

1. INTRODUCTION

The River Bend Station is a General Electric Boiling Water Reactor (BWR) plant. It has been selected for an audit to determine if the River Bend Technical Specifications (T/S) are consistent with the River Bend Final Safety Analysis Report (FSAR) up to and including Amendment 18 and the River Bend Safety Evaluation Report (SER) as supplemented through Supplement 1. The specific sections of the T/S which were audited are listed in Part 2. Differences between these sections of the T/S and the FSAR and SER were identified in Part 4.

2. REVIEW CRITERIA

The following T/S sections were reviewed for this evaluation:

1. Safety Limits
2. Reactor Protection System (RPS) Setpoints
3. Engineered Safety Features Actuation System (ESFAS) Setpoints
4. Pressure Boundary Isolation Valves (PIVs)
5. Containment Isolation Valves (CIVs)
6. Containment Depressurization and Cooling System Limiting Conditions for Operation (LCO's)
7. Combustible Gas Control System LCO's
8. Technical Specification Requirements Contained in the Safety Evaluation Report (SER)
9. Ice Condenser LCO's (Ice Condenser Plants Only)

The sections of the T/S listed in Part 4 were compared to the FSAR and SER to determine if the T/S are consistent, conservative or different than the FSAR and SER. Setpoints and lists of valves and instruments in the T/S were checked against tables in the FSAR and SER.

The SER was reviewed to ensure that T/S requirements identified in the SER were addressed in the T/S.

A description of each difference between the T/S and the FSAR and SER is included in this report.

3. SUMMARY

During the performance of this audit, several differences between the T/S, SER and FSAR were noted. Items which were found to be consistent during this review are indicated as consistent in Part 4 of this report. Items which were not resolved during this review are listed below and have been assigned a status code which indicates the present status of the item. These items are discussed in detail in Part 4 of this report. All other sections were evaluated and found to be consistent or conservative.

<u>Item</u>	<u>Title</u>	<u>Page</u>	<u>Status*</u>
Section I	Thermal Power Safety Limit	3	1
Section V	Containment Isolation Valves	4	1
Section VIII Item 13	Standby Liquid Control System	11	1
Section VIII Item 17	Digital Radiation Monitoring System	13	1
Section VIII Item 18b	Thermal Overload Testing	14	1
Section VIII Item 19	Nonsafety Loads on Emergency Sources	14	1
Section VIII Item 22	Reporting SRV Failures	16	1
Section VIII Item 23	Modification of ADS Logic	17	1

*Status Code

1. Unresolved, awaiting NRC/Utility action.
2. Resolved pending issuance of T/S revision.
3. Resolved pending issuance of SER Supplement.
4. Resolved pending issuance of FSAR Amendment.
5. Resolved, NRC accepted as is.

4. RIVER BEND STATION TECHNICAL SPECIFICATION,
FSAR and SER CONSISTENCY COMPARISON

Section I. - Safety Limits

This section covers the review of the safety limits as defined in Section 2.1 of the Standard Technical Specification. It includes the maximum allowed Thermal Power, RCS pressure, and Reactor Vessel Water Level.

<u>Technical Specification</u>	<u>FSAR Section</u>	<u>SER Section</u>	<u>Evaluation</u>
2.1.1 Thermal Power, Low Pressure or Low Flow	4.4, 15.1 Table 15.0-2	4.4	<u>CONSISTENT</u> . Limit not discussed in referenced FSAR and SER sections.
2.1.2 Thermal Power, High Pressure and High Flow	4.4, 15.1 Table 15.0-2	4.4	See NOTE.
2.1.3 Reactor Coolant System Pressure	5.2.2, 15.1	5.2.5	<u>CONSISTENT</u> .
2.1.4 Reactor Vessel Water Level	6.3, 7.3, 9.1, 15.1	6.3	<u>CONSISTENT</u> .

NOTE: FSAR Table 15.0-2 Item 15 specifies an MCPR of 1.07 after reload. The T/S does not mention any change from 1.06 after reload. This item is DIFFERENT.

Section II. Reactor Protection System Setpoints

This section covers the review of the Reactor Protection System Setpoints to insure the T/S values agree or are conservative to the values assumed in the safety analysis or defined in the SER.

<u>Technical Specification</u>	<u>FSAR Section</u>	<u>SER Section</u>	<u>Evaluation</u>
2.2.1 Reactor Protection System Instrumentation Setpoints 3/4.3.6 Page 3/4 3-62 Table 3.3.5-2	7.2; 15.0 Table 15.0-1	7.2, 15	<u>CONSISTENT</u> . The FSAR and SER referenced setpoints as stated in the T/S.

Section III. Engineered Safety Feature Actuation System Setpoints

This section covers the review of the ESFAS Setpoints to insure the T/S values agree or are conservative to the values identified in the FSAR sections or as defined in the SER as required values.

<u>Technical Specification</u>	<u>FSAR Section</u>	<u>SER Section</u>	<u>Evaluation</u>
3/4.3.2 Page 3/4 3-19	6.2,	7.3	<u>CONSISTENT.</u>
Table 3.3.2-2	7.3.1,		
3/4.3.3 Page 3/4 3-36	15.0,		
Table 3.3.3-2	Table 15.0-1		
3/4.3.4 Page 3/4 3-46			
& 51			
Table 3.3.4.1-2			
Table 3.3.4.2-2			
3/4.3.5 Page 3/4 3-57			
Table 3.3.5-2			

Section IV. Pressure Boundary Isolation Valves (PIVs)

This review determines if all of the PIV's identified through the FSAR and SER are included in the T/S.

<u>Technical Specification</u>	<u>FSAR Section</u>	<u>SER Section</u>	<u>Evaluation</u>
3.4.3.2 Page 3/4 4-11	5.2.2	3.9.6	<u>CONSISTENT</u>
Table 3.4.3.2-1		5.2.2	

Neither the FSAR nor SER identify any PIV's for comparison to the T/S Table.

This Item is CONSISTENT.

Section V. Containment Isolation Valves (CIVs)

This review determines if all of the CIVs identified through the FSAR and SER are included in the T/S.

<u>Technical Specification</u>	<u>FSAR Section</u>	<u>SER Section</u>	<u>Evaluation</u>
3/4.6.4 Page 3/4 6-33	6.2.4	6.2.4	<u>DIFFERENT.</u>
Table 3.6.4-1	Table 6.2-40		

All valves listed in FSAR Table 6.2-40 are listed in the T/S Table 3.6.4-1 with the exception of valve 1RHS-V240 which comes through penetration 1KJB-Z20 and is identified on page 5 of 18.

Also in T/S Table 3.6.4-1 Page 3/4 6-43 there appears to be two typographical errors in that valves:

✓ 1B21AOVF)1)A and 1B21AOVF)32B

should be:

✓ 1B21AOVF010A and 1B21AOVF032B respectively.

Section VI. Containment Depressurization and Cooling System
Limiting Conditions for Operation (LCO)

This section reviews the LCOs for the CDCS to insure they adequately cover the operation of the CDCS during all required modes of plant operation.

<u>Technical Specification</u>	<u>FSAR Section</u>	<u>SER Section</u>	<u>Evaluation</u>
3/4.6.3 Page 3/4 6-27	6.2.1	6.2.1	<u>CONSISTENT.</u>
LCO 3.6.3.1	6.2.2	6.2.2	
SR 4.6.3.1			
LCO 3.6.3.2			
SR 4.6.3.2			

The LCOs and Surveillance Requirements (SR) for these systems are effective during modes 1, 2, and 3 and do require all CDCS systems be operable.

This Item is CONSISTENT.

Section VII. Combustible Gas Control System Limiting
Conditions for Operation (LCO)

This section reviews the LCOs for the CGCS to insure they adequately cover the operation of the CGCS during all modes of plant operations.

<u>Technical Specification</u>	<u>FSAR Section</u>	<u>SER Section</u>	<u>Evaluation</u>
3/4.6.6 Page 3/4 6-64	6.2.5	6.2.5	<u>CONSISTENT.</u>
LCO 3.6.6.1			
LCO 3.6.6.2			
LCO 3.6.6.3			

These LCOs cover all of the required operations of the CGCS for modes 1 and 2.

This Item is CONSISTENT.

Section VIII. Technical Specifications Requirements
Contained in the Safety Evaluation Report

This section covers the review of all the items identified in the SER and Supplements as T/S required items and whether they have or have not been adequately addressed in the T/S.

1. SER Section: 2.5.4.3, Foundation Stability Page 2-36 states:

The applicant has committed to monitor structure settlements until there is essentially no movement during a 3-month period or until construction is complete, whichever is later. Subsequently one marker on each seismic Category I building will be monitored biennially for 10 years and, subsequently, following a seismic event equal to or greater than the OBE. The staff concurs in this general plan but also requires the following.

- a. This monitoring requirement is to be made a Technical Specification.
- b. Settlement readings should be made on at least three markers per building to detect differential settlement within buildings.
- c. Settlement readings should be evaluated against criteria to be established by a registered Professional Engineer. If any unexpected conditions develop, a corrective action plan should be proposed and submitted to the staff for review and approval before it is implemented.

T/S Section: 3/4.7.10

Page 3/4 7-35

Table 3.7.10-1 specifies the predicted settlements for the various buildings. T/S 3.7.10 and S/R 4.7.10 specify the structures and the frequency of this monitoring.

This item is CONSISTENT.

2. SER Section: 3.7.4, Seismic Instrumentation System Page 3-25 states:

The applicant has met SRP 3.7.4 except that a seismic instrumentation surveillance scheme has not yet been provided. However, in accordance with stated staff requirements, such a scheme will be incorporated in the Technical Specifications.

T/S Section: 3/4.3.7

Page 3/4 3.70

T/S 3.3.7.2 and S/R 4.3.7.2 specify the required surveillance scheme.

This item is CONSISTENT.

3. SER Section: 3.9.6, Inservice Testing Program Page 3-42 states:

Pressure isolation valves are required to be Category A or AC, according to ASME Code Paragraph IWV-2000, and to meet the appropriate requirements of IWV-3420 of Section XI of the ASME Code, except as discussed below. Limiting conditions for operation (LCO) must be added to the Technical Specifications

that will require corrective action (shutdown or system isolation) when the final approved leakage limits are not met. Also, surveillance requirements that will state the acceptable leak rate testing frequency must be provided in the Technical Specifications.

Periodic leak testing of each pressure isolation valve must be performed at least once each refueling outage, after valve maintenance, before return to service, and for systems rated as less than 50% of RCS design pressure each time the valve has moved from its fully closed position, unless justification is given. The testing interval should average approximately 1 yr. Leak testing should also be performed after all disturbances to the valves are complete, before power operation following a refueling outage or maintenance.

The staff's position on leak rate limiting conditions for operation is that leak rates must be equal to or less than 1 gpm for each valve to ensure the integrity of the valve, demonstrate the adequacy of the redundant pressure isolation function, and give an indication of valve degradation over a finite period.

T/S Section: 3/4.4.3

Page 3/4 4-8

T/S 3.4.3.2 specifies the required leakage limits.

S/R 4.4.3.2.2 specifies the required testing frequencies.

This item is CONSISTENT.

4. SER Section: 4.4.4, Thermal Hydraulic Stability Page 4-30 states:

The stability analysis resulted in a maximum decay ratio of 0.98 for the end-of-life cycle, which is the limiting cycle with respect to stability. Because the calculated maximum stability ratio is equal to that of some of the operating plants (for example, Peach Bottom Units 2 and 3 have a decay ratio of 0.98), the staff concludes that the thermal hydraulic stability result is acceptable for plant operation. However, to provide additional margin for stability, natural circulation under normal operation will be prohibited by Technical Specifications.

Because no analysis has been presented for MCPR limits or stability characteristics for single-loop operation, the staff will require by Technical Specifications that single-loop operation not be permitted until supporting analyses are provided and approved.

T/S Section: 3/4.4.1

Page 3/4 4-1

T/S 3.4.1.1 requires both recirculation loops be in operation for Modes 1 and 2 which also prohibits natural circulation operation.

This item is CONSISTENT.

5. SER Section: 4.4.5, Crud Deposition Page 4-30 states:

Crud deposition causes gradual flow reductions in some light water reactor cores. However, measurement of core flow by jet pump pressure drop and core plate pressure drop will provide adequate indication of such flow reductions, if they should occur. Technical Specifications will require that the core flow be checked at least once every 24 hours to detect flow reduction.

T/S 3/4.4.1

Page 3/4 4-2

S/R 4.4.1.2 requires total core flow be determined once per 24 hours during Modes 1 and 2.

This item is CONSISTENT.

6. SER Section: 4.4.6, Loose Parts Monitoring System Page 4-30 states:

On the basis of an evaluation indicating that the River Bend LPMS is in compliance with RG 1.133, the staff has concluded that the River Bend LPMS is acceptable on the condition that the Technical Specifications include appropriate limiting conditions for operation and surveillance requirements to demonstrate the operability of LPMS channels.

T/S Section: 3/4.3.7

Page 3/4 3-93

T/S 3.3.7.9 specifies the LCO for operation of this system.

S/R 4.3.7.9 specifies the frequencies for system operability verification.

This item is CONSISTENT.

7. SER Section: 5.2.2, Overpressure Protection Page 5-4 states:

A sensitivity study was performed for a BWR/3 to investigate the effects of a higher initial reactor pressure and recirculation pump trip on the results of the overpressure protection analysis. This analysis showed that increasing the initial operating pressure results in an increase in resultant peak system pressure that is less than half the initial pressure increase. For River Bend, the proposed Technical Specification limit on the high reactor pressure scram is 1095 psig. Therefore, because the vessel dome pressure used in the overpressurization analysis was 1045 psig, the maximum increase in the initial pressure would be limited to 50 psig, and the maximum peak system pressure increase during the overpressure design transient would be less than 25 psig. These results indicate that considerable margin is available before the code limit is reached and that GDC 15 will be satisfied even if

increased dome pressure and recirculation pump trip are considered. However, because specific River Bend overpressure analyses (as well as other Chapter 15 transient analyses) were performed assuming an initial dome pressure of 1045 psig, it is the staff's position that the Technical Specifications should include an operating pressure limit of 1045 psig for the power operation and start-up modes.

T/S Section: 3/4.4.6

Page 3/4 4-24

T/S 3.4.6.2 specifies reactor steam dome pressure be less than 1045 psig during Modes 1 and 2.

This item is CONSISTENT.

8. SER Section: 5.4.6, Reactor Core Isolation Cooling System
Page 5-19 states:

A high point vent is provided, and the system will be checked at least once every 31 days to ensure that the pump discharge lines are filled. The RCIC system includes a full flow test line with water return to the condensate storage tank for periodic testing. The Technical Specifications will include a flow test at least every 92 days and a system functional test at least every 18 months, with simulated automatic actuation and verification of proper automatic valve position. Both tests verify that the RCIC pump will develop a minimum flow of 600 gpm.

T/S Section: 3/4.7.3

Page 3/4 7-8

S/R 4.7.3 specifies the required testing be performed at the specified frequencies.

This item is CONSISTENT.

9. SER Section: 6.2.3, Secondary Containment Design Page 6-23
states:

The applicant has committed to include in the Technical Specifications periodic functional testing of the secondary containment structures and systems, including the SGTS and the fuel building charcoal filtration system drawdown times to establish a negative pressure. The staff will also require that the secondary containment inleakage rate be periodically checked. The inclusion of these periodic testing requirements will be verified during the staff review of the Technical Specifications.

T/S Section: 3/4.6.5

Page 3/4 6-49

S/R 4.6.5.1c specifies the system required drawdown limits and times.

SR 4.6.5.4b.1 specifies the SGTS inleakage verification requirement and frequency.

SR 4.6.5.6c.1 specifies the Fuel Building inleakage verification requirements and frequency.

This item is CONSISTENT.

10. SER Section: 6.2.6.3, Containment Leakage Type C Test Page 6-36 states:

For the above systems, the applicant indicated that a liquid inventory will produce a water seal during the post-accident period, and only liquid leakage from the containment will occur. The combined leakage from all these valves will satisfy the acceptance criteria of 10 CFR 100 regarding the site radiological safety analysis and will be included in the plant Technical Specifications. This leakage will therefore be excluded when the combined leakage rate for all penetrations and valves is determined, as provided in Appendix J, Paragraph III.C.3.

T/S Section: 3/4.6.1

Page 3/4 6-3

T/S 3.6.1.3b specifies the leakage limit for all of these systems as required.

This item is CONSISTENT.

11. SER Section: 6.3.3.3, ECCS Functional Design Page 6-41 states:

The applicant has addressed long-term leakage from the first isolation valve outside the suppression pool following a LOCA (LRG-II issue 3-RSB). In a letter dated December 12, 1983 [from J. E. Booker (GSU) to H. R. Denton (NRC)], the applicant postulated a conservative leak rate of 50 gpm from the valve. It was indicated that the leakage will drain to one of the reactor building sumps, each of which is equipped with a 50-gpm pump to transfer water to the radwaste system. Redundant capability for water processing is provided in the radwaste system, and an operability requirement for the system will be included in the plant Technical Specifications. Water processed in the radwaste system is returned to the condensate storage tank from where it can be injected into the vessel and suppression pool by the high pressure core spray system, the control rod drive hydraulic system, or the reactor core isolation cooling system pumps. A closed loop is therefore provided to maintain suppression pool inventory.

T/S Section: 3/4.11.1

Page 3/4 11-5

T/S 3.11.1.3 specifies the Liquid Radwaste Treatment System be in operation at all times.

This item is CONSISTENT.

12. SER Section: 7.2.2.10, Protection System Surveillance Testing
Page 7-12 states:

The staff will verify that the Technical Specifications include appropriate surveillance requirements to require periodic (online) demonstration of the operability of the RPS and ESF instrument channels logic and actuation devices.

T/S Section: 3/4.3.1,
3/4.3.2 and 3/4.3.3

Page 3/4 3-1, 3/4 3-10 and
Page 3/4 3-30 respectively

S/R 4.3.1.1 requires the RPS periodic demonstrations be performed as specified.

S/R 4.3.2.1 requires part of the ESF periodic demonstrations be performed as specified.

S/R 4.3.3.1 requires part of the ESF periodic demonstrations be performed as required.

This item is CONSISTENT.

13. SER Section: 7.4.2.3, Standby Liquid Control System Page 7-34
states:

The SLCS design includes an interlock that prevents the storage tank suction valves from opening (thus preventing system level manual initiation) if test valve C41-F031 is open. This valve (test tank suction valve) is opened to allow testing of the SLCS pumps by circulating water through the pumps and back to the test tank. The interlock is provided to prevent dilution of the sodium pentaborate from water in the test tank. SLCS inoperable status indication (except for valve position indicator lights) is not provided in the control room when valve C41-F031 is open. Bypassed and inoperable status indication is also addressed in Section 7.5.2.2 of this report.

The staff will confirm that sufficient SLCS inoperable status indication is provided in the control room.

The River Bend Technical Specifications will include provisions for periodic testing of the SLCS interlock with valve C41-F031.

T/S Section: 3/4.1.5

Page 3/4 1-19

T/S 3.1.5 and S/R 4.1.5 give all of the Limiting Conditions for operation and surveillance requirements for the SLCS but make no provisions for periodic testing of the SLCS interlock with Valve C41-F031.

This item is DIFFERENT.

*Also in
SSER #2*

*Kendall
P. [unclear]
or
Kew*

14. SER Section: 7.6.2.2, High Pressure/Low Pressure System Interlocks Page 7-47 states:

The low pressure ECCS injection valves for LPCS and LPCI A, B, and C are interlocked to prevent them from opening in response to either manual (control room hand switches) or automatic (ECCS initiation logic) actuation signals if reactor pressure is greater than the permissive setpoint value. Manual actuation of each injection valve is prevented if reactor high pressure is sensed by a single pressure sensor downstream of the valve (between the injection valve and series testable check valve). Automatic actuation of each injection valve on a LOCA signal (high drywell pressure and/or low reactor vessel water level) is prevented if reactor high pressure is sensed by four divisionally associated pressure sensors arranged in a one-out-of-two-taken-twice logic configuration. The staff concludes that this design complies with RG 1.62 and Section 4.17 of IEEE 279-1971, which state that the amount of equipment (interlocks) common to both manual and automatic initiation should be kept at a minimum so that failures within the manual or automatic portions of the system will not prevent initiation of a protective action by both manual and automatic means. All 12 pressure channels providing the reactor low pressure permissive interlock function are safety-related Class 1E equipment and will be required to be periodically tested in accordance with the River Bend Technical Specifications.

T/S Section: 3/4.3.3

Page 3/4 3-41

S/R 4.3.3.1 and Table 4.3.3.1-1 item A.1.d and A.1.e specify the required surveillance of the interlocks.

This item is CONSISTENT.

15. SER Section: 7.6.2.3, SRV Low-Low Set Logic Page 7-49 states:

The River Bend Technical Specification surveillance requirements will include provisions for periodically testing both trains of LLS instrument channels and logic.

T/S Section: 3/4.4.2

Page 3/4 4-7

S/R 4.4.2.2.1 specifies the required periodic testing of the LLS channels.

This item is CONSISTENT.

16. SER Section: 7.6.2.6, Rod Pattern Control System Microprocessor Page 7-52 states:

The River Bend Technical Specifications will include the appropriate provisions for RPCS periodic testing and operability.

S/R 4.1.4.2 provides for periodic testing for system operability as required.

This item is CONSISTENT.

17. SER Section: 7.6.2.7, Digital Radiation Monitoring System
Page 7-52 states:

The staff will confirm that the River Bend DRMS design is acceptable with regard to the following:

- a. the adequacy of the isolation devices used in the DRMS (see Section 7.2.2.6)
- b. the software design methodology (development and qualification) used, and the implementation of the methodology in the final design
- c. DRMS test capability and Technical Specification surveillance requirements.

T/S Section: There was no T/S identified that specifically addresses the DRMS.

This item is DIFFERENT.

18. SER Section: 8.4.2, Containment Electrical Penetrations
Page 8-14 states:

- a. Responding to the staff concern on the fault current interrupting capability of the motor contactors, the applicant has stated that the contactors are specified to be in complete conformance to National Electrical Manufacturers Association (NEMA) ICS standards, which requires that a design test be performed to demonstrate the ability of the contactor to make and break current of 10 times the contactor-rated current for 10 operations. The applicant has also stated that any currents in excess of 10 times the contactor-rated current will be interrupted by the penetration circuit breaker. The staff finds this arrangement for overcurrent protection of the penetration acceptable. The motor contactors in this case are an integral part of the penetration overcurrent protection system; therefore, they must be tested periodically along with the penetration fuses and circuit breakers. The staff will ensure that the River Bend Technical Specifications contain a requirement to test the interrupting capability of the motor contactors at a value equal to approximately the locked rotor current of the associated motors.

- b. In a letter dated March 7, 1984, the applicant provided a penetration overcurrent protection curve that showed the use of a motor contactor and its thermal overloads as an overcurrent device for the protection of the penetration. Thermal overloads used in the application in motor-operated valve circuits will not have their thermal overloads bypassed. The thermal overload, however, is set at approximately 160% of the motor full load current. This is conservative with respect to maintaining operation of the motor and complies with Position C.2 of RG 1.106. The staff will ensure that the River Bend Technical Specifications contain a requirement to periodically calibrate these thermal overloads.

Lazevnick

*set time coil - Lazevnick
E. Duxley
7/2/84*

T/S Section: 3/4.8.4

Page 3/4 8-25

- a. S/R 4.8.4.2a.3 specifies that once per 18 months that at least 10% of all motor starters be subjected to locked rotor current as specified.

This item is CONSISTENT.

- b. T/S Section: No T/S was identified that specifically discusses periodic testing of the thermal overloads as specified.

This item is DIFFERENT.

19. SER Section: 8.4.6, Nonsafety Loads on Emergency Sources
Page 8-17 states:

The emergency lighting to the control room is the only non-Class 1E system that is connected to the Class 1E system and is not tripped on a LOCA signal. This is necessary because a portion of the lighting must be available during a loss-of-offsite power to achieve an orderly shutdown. The FSAR states that, from the Class 1E bus up to and including the power receptacle, the circuit is designed as Class 1E with two independent overcurrent protection devices installed in the circuit to ensure protection of the Class 1E portion of the circuits. These provisions are acceptable if both overcurrent devices are coordinated with the Class 1E bus feeder breaker and are tested periodically. In a supplement to this report, the staff will confirm that both overcurrent devices are coordinated with the Class 1E bus feeder breaker. The staff also will ensure that provisions are included in the River Bend Technical Specifications to test the overcurrent devices on a periodic basis.

T/S Section: No T/S section was identified that specifically addresses testing the overcurrent devices of these particular 120 V systems.

This item is DIFFERENT.

20. SER Section: 9.1.3, Spent Fuel Pool Cooling and Cleanup System
Page 9-6 states:

Under abnormal heat load conditions, the reactor plant component cooling water (RPCCW) system provides cooling water to the fuel pool heat exchangers. The maximum abnormal heat load is based on a full-core offload 10 days after the last normal refueling outage and a storage load of 3104 spent fuel bundles. Under these conditions, the cooling system will maintain the temperature of the water at or below 156°F. If the reactor plant component cooling water system is not available, the safety-related standby service water system would be available to cool the heat exchangers through the portion of the RPCCW piping to the standby service water system, which is designed to seismic Category I and Quality Group C standards for this purpose. The FSAR states that BTP ASB 9-2, "Residual Decay Energy for Light Water Reactors for Long Term Cooling," was used to calculate the heat loads. The applicant was committed to providing a Technical Specification that will prohibit storage of spent fuel in the upper containment fuel storage pool during normal operation.

T/S Section: Design Feature 5.6 Page 5-6

D/F 5.6.1.1 specifies that storage of spent fuel in the upper containment fuel storage pool is prohibited during normal operation.

This item is CONSISTENT.

21. SER Section: 15.2, Increase in Reactor Pressure Page 15-6 states:

- a. In analyzing anticipated operational transients, the applicant has taken credit for plant operating equipment that is not normally reviewed by the staff because it is not considered essential for safety. The staff has discussed the application of this equipment generically with GE. On the basis of these discussions, it is the understanding of the staff that the most limiting transient that takes credit for this equipment is the feedwater control failure event. Further, the only plant operating equipment that plays a significant role in mitigating this event (excess feedwater) is the turbine bypass system and the level 8 high water level trip (closes turbine stop valves). To insure an acceptable level of performance for River Bend, the staff has determined that this equipment must be identified in the plant Technical Specifications with regard to availability, setpoints, and surveillance testing.

- b. Whenever equipment or systems are included in the Technical Specifications, it is necessary to define operating limits for the case when the equipment is inoperable. One option is to require a power reduction to less than 25% of rated power if the equipment is inoperable. The basis for this reduction is that MCPR is unimportant below this level. The staff requires additional analyses to cover the feedwater controller failure event.

Until such analyses are provided, the staff will require power reduction to 25% if the turbine bypass system becomes inoperable. This limiting condition of operation will be put in the plant Technical Specifications.

T/S Section: 3/4 7.9, 3/4.3.1 Page 3/4 7-33, 3/4 3-1

- a. T/S 3.3.1 and 3.7.9 provide the required operability statements.
- S/R 4.3.1.1 through 4.3.1.3 and 4.7.9 provide the required surveillance requirements.

This item is CONSISTENT.

- b. T/S 3.7.9 specifies that if the TBS is inoperable then the reactor must be in STARTUP within 6 hours.

This item is CONSERVATIVE.

22. SER Section: 15.9.4 II.K.3.3, Reporting Safety and Relief Valve Failures Page 15-20 states:

Because River Bend has not yet operated, no valve failures have yet been reported. The applicant will promptly report safety/relief valve failures via the Licensee Event Report system and will summarize failures in the annual report. The plant Technical Specifications will require these failures to be reported within 30 days.

T/S Section: Administrative Controls 6.9 Page 6-16

A/C 6.9.1.5 specifies summarizing all challenges to SRV's but does not specify summarizing SRV failures as required.

This item is DIFFERENT.

A/C 6.9.1.6 specifies monthly reporting of challenges to SRV's but does not specify SRV failure reporting as required.

This item is DIFFERENT.

23. SER Section: 15.9.4 II.K.3.18, Modification of ADS Logic
Page 15-22 states:

By letter dated December 12, 1983, the applicant adopted the results of the BWR owners group report on TMI Action Plan Item II.K.3.18 "Modification of Automatic Depressurization Systems (ADS) Logic--Feasibility for Increased Diversity for Some Events." The applicant has committed to modify the ADS logic to bypass the high drywell pressure trip after a sustained low water level signal and to add a manual switch that may be used to inhibit ADS actuation if necessary. This is consistent with option 4 of the owners group study and is acceptable to the staff with the following conditions: (1) installation must be completed prior to initial criticality; (2) Technical Specifications must be provided for the bypass timer and manual inhibit switch; (3) the use of the inhibit switch must be addressed in the plant emergency procedures; and (4) a plant-specific analysis must be provided to justify the bypass timer setting.

T/S Section: 3/4.3.3 No T/S or S/R was identified that specifically discusses the bypass timer or manual inhibit switch.

This item is DIFFERENT.

24. SSER 1 Section: 6.2.3, Secondary Containment Functional Design
Page 6-1 states:

In the SER, the staff required that before plant operation begins and at each refueling outage, the shield building, auxiliary building, and the fuel building must be tested to verify that the inleakage will not exceed 2000 cfm, 5000 cfm, and 5000 cfm at a pressure of -0.50 inch wg, -0.25 inch wg, and -0.25 inch wg, respectively. The staff will require that these periodic testing requirements be included in the Technical Specifications.

T/S Section: 3/4.6.5

Page 3/4 6-50

S/R 4.6.5.1c.1,2,3 and 4 specify that once per 18 months the above stated conditions be met.

This item is CONSISTENT.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION IV
611 RYAN PLAZA DRIVE, SUITE 1000
ARLINGTON, TEXAS 76011

MAY 13 1985

J. Reeb
2 Dean

MEMORANDUM FOR: Dennis M. Crutchfield, Assistant Director for Safety
Assessment, Division of Licensing
Thomas M. Novak, Assistant Director for Licensing,
Division of Licensing

FROM: Richard P. Denise, Director, Division of Reactor Safety and
Projects

SUBJECT: FINAL DRAFT OF THE RIVER BEND UNIT 1 TECHNICAL SPECIFICATIONS

This provides information on the Region IV review of the final draft technical specifications for River Bend. It is forwarded as requested by your letter of April 19, 1985.

A Region IV team inspection was conducted from April 29 through May 3, 1985. The results of this inspection were that some 25 items were identified, which appear to be under NRR cognizance. These are documented in the attached inspection report forms. Except for the attached, no additional items were identified which we feel require action on your part. There were, however, over 300 items related to the licensee's implementing procedures. These will be documented in Inspection Report 50-458/85-35.

Although our inspection of the draft technical specifications was primarily directed toward the licensee's readiness to implement them, we did review the technical specifications for obvious discrepancies between them, the Safety Evaluation Report (NUREG-0989, including Supplement 1), and the Final Safety Analysis Report (through Amendment 17). There were no problems found in this area, which are not already noted in the attached report forms.

Richard P. Denise
FOR Richard P. Denise, Director
Division of Reactor Safety and
Projects

Attachments:
As stated

~~8509060.236~~

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.6.6.3.91

PAGE NO.: 3/4 6-66

TS REQUIREMENT: Verify visible glow from glow plug
Tip of each normally accessible igniter

LIC. PROCEDURE NO.: _____ ISSUE DATE: _____ REV.: _____

TITLE: _____

	<u>YES</u>	<u>NO</u>
Is there any difference between FSAR/SER and TS?	_____	_____ <input checked="" type="checkbox"/>
Is installed system consistent with TS?	_____ <input checked="" type="checkbox"/>	_____
Are there any problems with the TS (factual or editorial)?	_____ <input checked="" type="checkbox"/>	_____
Does procedure carry out TS requirement?	_____	_____
Does the procedure walkdown indicate that it should work as written?	_____	_____
Has licensee completed procedure walkdown?	_____	_____

REMARKS: All igniters are inside containment, which would appear to mean that they are not normally accessible during normal operation. Embroid for the sole purpose of looking at glow plugs would violate ALARA. Para 4.6.6.3.91² covered by other means and para 4.6.3.6 crew S/C map.

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?
Review TS wording ← Why? -X

INSPECTOR(S): _____

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	<u>YES</u>	<u>NO</u>
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____
STATUS OF ITEM	OPEN	CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ _____

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4.8.1.1

PAGE NO.: 3/4 8 7 1

TS REQUIREMENT: 4.8.1.1.2.f.9 Each of the above required diesel generators shall be demonstrated OPERABLE: at least once per 8 months, during shutdown, by: (see attached page)

LIC. PROCEDURE NO.: _____ ISSUE DATE: _____ REV.: _____

TITLE: (see attached page)

	YES	NO
Is there any difference between FSAR/SER and TS?	_____	<input checked="" type="checkbox"/>
Is installed system consistent with TS?	<input checked="" type="checkbox"/>	_____
Are there any problems with the TS (factual or editorial)?	<input checked="" type="checkbox"/>	_____
Does procedure carry out TS requirement?	_____	<u>N/A</u>
Does the procedure walkdown indicate that it should work as written?	_____	<u>N/A</u>
Has licensee completed procedure walkdown?	_____	<u>N/A</u>

REMARKS: (1) T.S. REQUIREMENT STATES 330 KW FOR MAX LOAD FOR DGs A and B INSTEAD OF THE CORRECT (APPARENTLY) 3130 KW

(2) Procedure SIP-309-0603 specifies incorrect load limit for Div 3 diesel generator. (it is based on superseded TS revision)

(3) Procedures for Div 1 and Div 2 not issued yet. draft

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

(1) Licensee must issue revised procedures which reflect final TS draft. (1) NRC must correct technical specification

(2) Licensee must correct procedure.

INSPECTOR(S): C.C. Harbuck

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	YES	NO
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ PS-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4.8.1.1

PAGE NO.: 3/4 8.7

TS REQUIREMENT: 4.8.1.1.2.f.12 Each of the above DGs shall be demonstrated OPERABLE: At least once per 8 mos., during shut down, by: (see attached)

LIC. PROCEDURE NO.: STP-305-0601 0602

ISSUE DATE: _____

REV.: _____

TITLE: DIV 1 (DIV 2) Loss of offsite Power / ECCS activation Test

	<u>YES</u>	<u>NO</u>
Is there any difference between FSAR/SER and TS?	_____	<input checked="" type="checkbox"/>
Is installed system consistent with TS?	<input checked="" type="checkbox"/>	_____
Are there any problems with the TS (factual or editorial)?	<u>Possibly</u> _____	_____
Does procedure carry out TS requirement?	_____	<u>N/A</u>
Does the procedure walkdown indicate that it should work as written?	_____	<u>N/A</u>
Has licensee completed procedure walkdown?	_____	<u>N/A</u>

REMARKS: (1) The procedures have not yet been issued.
(2) TS limit of ±10% appears to be arbitrary.

STATUS OF ITEM

OPEN

CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

① Licensee issue procedures

② NRC determine correctness of ±10%

NRC follow up

INSPECTOR(S): C. C. Harbuck

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	<u>YES</u>	<u>NO</u>
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM

OPEN

CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-35

PAGE NO.: _____

TECHNICAL SPECIFICATION DATA CONTINUATION SHEET

TS PARA: 3/4.8.1.1

PAGE NO.: 3/4 8-7, 8-8

4.8.1.1.2.f. 13

REQUIREMENT:

13. Verifying that the following diesel generator lockout features prevent diesel generator starting only when required:

- a) For Diesel Generators 1A and 1B:
- 1) Diesel control panel loss or control power.
 - 2) Starting air pressure below 50 psi.
 - 3) Stop solenoid energized.
 - 4) Diesel in the maintenance mode (includes barring device engaged).
 - 5) Overspeed trip device actuated.
 - 6) Generator backup protection lockout relay tripped.
- b) For Diesel Generator 1C:
- 1) Diesel generator lockout relays not reset.
 - 2) Diesel engine mode switch not in "AUTO" position.
 - 3) Diesel generator output breaker closed before start of diesel.
 - 4) *Diesel generator output breaker in racked-out position.
 - 5) *Diesel generator regulator mode switch not in "AUTO" position.
 - 6) Insufficient starting air pressure.
 - 7) Loss of dc power to diesel generator controls.

*Take this to
- PSB*

** Items 4) and 5) do not electrically block diesel generator from emergency starting; however these will affect the loading and operation of the diesel. Missing*

Page 7

(2) (continued) is to observe ^{***} an indication light in the control room. This in fact may not work for all the lockouts. For example, Div 3 diesel, the lockouts nos. 3), 4), and 5) apparently don't prevent a manual start, even though an indicating light would energize.

(3) The TS is vague concerning manual starts. Are they also included? Why is an ECCS actuation signal not simulated to see if the diesels start even with the lockouts?

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4.8.3

PAGE NO.: 3/4 8-17 thru 8-22

TS REQUIREMENT: _____

LIC. PROCEDURE NO.: _____

ISSUE DATE: _____

REV.: _____

TITLE: _____

	YES	NO
Is there any difference between FSAR/SER and TS?	_____	_____
Is installed system consistent with TS?	_____	_____
Are there any problems with the TS (factual or editorial)?	<u>X</u>	_____
Does procedure carry out TS requirement?	_____	_____
Does the procedure walkdown indicate that it should work as written?	_____	_____
Has licensee completed procedure walkdown?	_____	_____

REMARKS: 1. Reviewer can't see the need for taking the UPS off line during equalization charge. UPS systems are designed to operate without degradation during this higher voltage period. 2. On page 8-21, is the statement in paragraphs a.1 & b.1 less than correct? or should it be either

STATUS OF ITEM OPEN CLOSURE

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

- ① NRC must resolve
- ② NRC must correct statements

INSPECTOR(S): K.A. Ward

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	YES	NO
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM OPEN CLOSURE

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4.8.4.1

PAGE NO.: 3/4 8-24 & 25

TS REQUIREMENT: 4.8.4.1. a and b: Each of the primary containment penetrations conductor over-current protective devices shown in Table 3.8.4.1-1 shall be demonstrated operable.

LIC. PROCEDURE NO.: ISSUE DATE: REV.:

TITLE: see continuation sheet

	YES	NO
Is there any difference between FSAR/SER and TS?	<u> </u>	<u> </u> *
Is installed system consistent with TS?	<u> </u>	<u> </u> *
Are there any problems with the TS (factual or editorial)?	<u>✓</u>	<u> </u>
Does procedure carry out TS requirement?	<u> </u>	<u> </u> *
Does the procedure walkdown indicate that it should work as written?	<u> </u>	<u>N/A</u>
Has licensee completed procedure walkdown?	<u>✓</u>	<u> </u>

REMARKS: * Procedure review incomplete and FSAR/SER - TS review not completed.

(1) TS contains typographical errors on numbering of paragraphs 4.8.4.2 should be 4.8.4.1 on page 8-24.

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

Licensee to: clear "laters" (2), issue Rev. 1 of procedures (2) (3), issue procedure (4).

NRC to correct TS.

INSPECTOR(S): C.C. Harbuck

FOLLOWUP ACTION: INSPECTION REPORT 50-458/

	YES	NO
Were problems corrected?	<u> </u>	<u> </u>
Were any other problems identified?	<u> </u>	<u> </u>

STATUS OF ITEM OPEN CLOSED

REMARKS:

INSPECTOR(S):

INSPECTION REPORT 50-458/ 85-35

PAGE NO.:

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4.8.4.3

PAGE NO.: 3/4 8-33

TS REQUIREMENT: 4.8.4.3.6

see continuation sheet

LIC. PROCEDURE NO.: 5TP-508-1600 ISSUE DATE: 4-24-85 REV.: 1

TITLE: EPA Channel Calibration

	<u>YES</u>	<u>NO</u>
Is there any difference between FSAR/SER and TS?	_____	<u>N/A</u>
Is installed system consistent with TS?	_____	<u>N/A</u>
Are there any problems with the TS (factual or editorial)?	<input checked="" type="checkbox"/>	_____ <i>see remarks</i>
Does procedure carry out TS requirement?	_____	<u>N/A</u>
Does the procedure walkdown indicate that it should work as written?	_____	<u>N/A</u>
Has licensee completed procedure walkdown?	_____	<u>N/A</u>

REMARKS (1) TS paragraphs 4.8.4.3.6.1 and .2 are incomplete
(Bus B Vac limits)
(Procedure was not reviewed)

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

NRC must correct TS

INSPECTOR(S): C.C. Harbuck

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	<u>YES</u>	<u>NO</u>
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.9.2.C.3.A

PAGE NO.: 3/4 9-4

TS REQUIREMENT: Verify minimum SAM count rate during Fuel Unloading

LIC. PROCEDURE NO.: STP-000-0001 ISSUE DATE: 8/24/74 REV.: 0

TITLE: Daily Operating Logs

	<u>YES</u>	<u>NO</u>
Is there any difference between FSAR/SER and TS?	<u>NOT VERIFIED</u>	<u>NOT VERIFIED</u>
Is installed system consistent with TS?	<u>NOT VERIFIED</u>	<u>NOT VERIFIED</u>
Are there any problems with the TS (factual or editorial)?	<u>X</u>	<u> </u>
Does procedure carry out TS requirement?	<u> </u>	<u>X</u>
Does the procedure walkdown indicate that it should work as written?	<u>NOT VERIFIED</u>	<u>NOT VERIFIED</u>
Has licensee completed procedure walkdown?	<u> </u>	<u>X</u>

REMARKS: ① TS allows count rates less than 0.7 cps, This is contrary to NRR (core performance) practice
② "LATENS" are in the procedure

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

- ① NRR must correct TS
- ② Applicant must incorporate letters

INSPECTOR(S): Dale A. Powers

FOLLOWUP ACTION: INSPECTION REPORT 50-458/

	<u>YES</u>	<u>NO</u>
Were problems corrected?	<u> </u>	<u> </u>
Were any other problems identified?	<u> </u>	<u> </u>
STATUS OF ITEM	<u>OPEN</u>	<u>CLOSED</u>

REMARKS:

INSPECTOR(S):

INSPECTION REPORT 50-458/ 85-35

PAGE NO.:

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.10.1

PAGE NO.: 3/4 10-1

TS REQUIREMENT: THE THERMAL POWER AND REACTOR COOLANT TEMPERATURE SHALL BE VERIFIED TO BE WITHIN THE LIMITS AT LEAST ONCE PER HOUR DURING LOW POWER PHYSICS TESTS.

LIC. PROCEDURE NO.: STP-050-0704 ISSUE DATE: 1-11-85 REV.: 0 11

TITLE: THERMAL POWER AND REACTOR COOLANT SYSTEM TEMPERATURE VERIFICATION DURING LOW POWER PHYSICS-TEST

- | | <u>YES</u> | <u>NO</u> |
|---|------------|-----------|
| 1. Is there any difference between FSAR/SER and TS? | _____ | <u>✓</u> |
| 2. Is installed system consistent with TS? | <u>NA</u> | <u>NA</u> |
| 3. Are there any problems with the TS (factual or editorial)? | <u>✓</u> | _____ |
| 4. Does procedure carry out TS requirement? | <u>✓</u> | _____ |
| 5. Does the procedure walkdown indicate that it should work as written? | _____ | <u>✓</u> |
| 6. Has licensee completed procedure walkdown? | _____ | <u>✓</u> |

REMARKS: ② SURVEILLANCE REQUIREMENTS SHOULD INCLUDE REQUIREMENT FOR "A SECOND LICENSED OPERATOR OR OTHER TECHNICALLY QUALIFIED MEMBER OF THE UNIT TECHNICAL STAFF IS PRESENT AND VERIFIES COMPLIANCE WITH THE PHYSICS TEST."

⑤ STEP 7.4 - NO SOURCE RANGE MONITOR INSTRUMENT NUMBER C51-R600
STEP 7.5 - NEED TO DEFINE HOW TO OBTAIN THERMAL POWER

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

3. NRC RECONCILE BY FUEL LOAD
5 LICENSEE RECONCILE BY FUEL LOAD

INSPECTOR(S): DWIGHT D. CHAMBERLAIN

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

- | | <u>YES</u> | <u>NO</u> |
|--|------------|-----------|
| Were problems corrected? | _____ | _____ |
| Were any other problems identified? | _____ | _____ |
| STATUS OF ITEM OPEN CLOSED | | |

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.10.2

PAGE NO.: 3/4 10-2

TS REQUIREMENT: WHEN THE SEQUENCE CONSTRAINTS IMPOSED ON CONTROL ROD GROUPS BY THE RPCS ARE BYPASSED, VERIFY: (SEE CONTINUATION SHEET)

LIC. PROCEDURE NO.: STP-500-0705 ISSUE DATE: 1-02-85 REV.: 0 (11)

TITLE: ROD SEQUENCE VERIFIED IN WHEN RPCS IS BYPASSED FOR TESTING

	YES	NO
1. Is there any difference between FSAR/SER and TS?	NA	NA
2. Is installed system consistent with TS?	NA	NA
3. Are there any problems with the TS (factual or editorial)?	✓	
4. Does procedure carry out TS requirement?		✓
5. Does the procedure walkdown indicate that it should work as written?	✓	
6. Has licensee completed procedure walkdown?	✓ 0705	

REMARKS: (4) ACTUAL ROD POSITIONS ARE RECORDED AND VERIFIED BUT REQUIRED POSITION LIMITS ARE NOT INDICATED

(3) 4.10.2.a MOVEMENT OF CONTROL RODS FROM 100% ROD DENSITY INSTEAD OF 75%

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

(3) NRC RECONCILE BY FUEL LOAD

(4) LICENSEE RECONCILE BY FUEL LOAD

INSPECTOR(S): DWIGHT D. CHAMBERLAIN

FOLLOWUP ACTION: INSPECTION REPORT 50-458/_____

	YES	NO
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ PS-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.10.3

PAGE NO.: 3/4 10-3

TS REQUIREMENT: WITHIN 30 MINUTES PRIOR TO AND AT LEAST ONCE PER 12 HOURS DURING THE PERFORMANCE OF A SHUTDOWN MARGIN DEMONSTRATION, VERIFY THAT; (SEE CONTINUATION SHEET)

LIC. PROCEDURE NO.: STP-050-3601 ISSUE DATE: 2-15-85 REV.: 0 (1)

TITLE: SHUTDOWN MARGIN DEMONSTRATION

- | | <u>YES</u> | <u>NO</u> |
|---|-------------|-------------|
| 1. Is there any difference between FSAR/SER and TS? | <u>NA</u> | <u>NA</u> |
| 2. Is installed system consistent with TS? | <u>NA</u> | <u>NA</u> |
| 3. Are there any problems with the TS (factual or editorial)? | <u>✓</u> | <u> </u> |
| 4. Does procedure carry out TS requirement? | <u>✓*</u> | <u> </u> |
| 5. Does the procedure walkdown indicate that it should work as written? | <u> </u> | <u>✓</u> |
| 6. Has licensee completed procedure walkdown? | <u> </u> | <u>✓</u> |

REMARKS: 4* DATA SHEET 2 OF STP-050-3601 REVIEWED WHICH INVOKES REQUIREMENTS OF TECHNICAL SPECIFICATION 4.10.3 - EDITORIAL COMMENTS DATA SHEET 2 SECTION (1)(b) SHOULD STATE "THE ROD PATTERN CONTROL SYSTEM OPERABLE PER SPECIFICATION 3.1.4.2" SECTION (2) SHOULD STATE "A SECOND LICENSED OPERATOR OR OTHER TECHNICALLY QUALIFIED MEMBER OF THE UNIT TECHNICAL STAFF ..."

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

3. NRC RECONCILE BY FUEL LOAD

4. LICENSEE RECONCILE BY FUEL LOAD

5. LICENSEE RECONCILE BY FUEL LOAD

INSPECTOR(S): DWIGHT D. CHAMBERLAIN

FOLLOWUP ACTION: INSPECTION REPORT 50-458/

- | | <u>YES</u> | <u>NO</u> |
|--|-------------|-------------|
| Were problems corrected? | <u> </u> | <u> </u> |
| Were any other problems identified? | <u> </u> | <u> </u> |
| STATUS OF ITEM OPEN CLOSED | | |

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 75-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4 4.3.2.1

PAGE NO.: 3/4 3-11

TS REQUIREMENT: Channel check, channel functional test, and channel calibration operations

LIC. PROCEDURE NO.: SA-09-061 ISSUE DATE: 2/1/85 REV.: 0

TITLE: _____

	YES	NO
Is there any difference between FSAR/SER and TS?	_____	<u>N/A</u> _____
Is installed system consistent with TS?	_____	<u>N/A</u> _____
Are there any problems with the TS (factual or editorial)?	<u>✓</u> _____	_____
Does procedure carry out TS requirement?	_____	_____ <u>✓</u>
Does the procedure walkdown indicate that it should work as written?	_____	<u>N/A</u> _____
Has licensee completed procedure walkdown?	_____	<u>✓</u> _____

REMARKS: 1) The note (a) in Table 4.3.2.1-1 for manual initiation channel functional test is confusing and open for interpretation
2) The channel functional test for manual initiation of main steam line isolation can be performed without actual initiation. No procedures exist to perform this test.

check somebody

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

1) NRC must determine requirement for function testing for manual initiation and clarify note a as required
2) Licensee must develop procedures for main steam line isolation testing and

Contract

INSPECTOR(S): Will Bennett

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	YES	NO
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-25

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4, 43.2.1

PAGE NO.: 3/4 3-11

TS REQUIREMENT: Channel check, channel functional test, and channel calibration operations

LIC. PROCEDURE NO.: 877-051-4201 ISSUE DATE: 12/19/84 REV.: 0

TITLE: NSSS - Reactor Vessel Water Level - Low Low, Level 2; Low, Low, Low, Level 1, Channel Calibration (B21-N081A; B21-N691A; B21-N692A)

	YES	NO	
Is there any difference between FSAR/SER and TS?	<input type="checkbox"/>	<input type="checkbox"/>	N/A
Is installed system consistent with TS?	<input type="checkbox"/>	<input type="checkbox"/>	N/A
Are there any problems with the TS (factual or editorial)?	<input checked="" type="checkbox"/>	<input type="checkbox"/>	
Does procedure carry out TS requirement?	<input type="checkbox"/>	<input checked="" type="checkbox"/>	
Does the procedure walkdown indicate that it should work as written?	<input type="checkbox"/>	<input type="checkbox"/>	N/A
Has licensee completed procedure walkdown?	<input type="checkbox"/>	<input type="checkbox"/>	

REMARKS: 1) TS Table 3.3.2-2, Trip Function 1, a. specifies "5-45.5 inches" should be "2-45.5 inches"; 2) Procedure, as written, does not specify 31 day channel functional test as required by technical specifications. Procedure not walked down.

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

TS correction required. Licensee must implement periodicity requirements of technical specifications

INSPECTOR(S): W.R. Bennett

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	YES	NO
Were problems corrected?	<input type="checkbox"/>	<input type="checkbox"/>
Were any other problems identified?	<input type="checkbox"/>	<input type="checkbox"/>
STATUS OF ITEM	OPEN	CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-35

PAGE NO.: _____

MAY 13 1985

MEMORANDUM FOR: Dennis M. Crutchfield, Assistant Director for Safety
Assessment, Division of Licensing

Thomas M. Novak, Assistant Director for Licensing,
Division of Licensing

FROM: Richard P. Denise, Director, Division of Reactor Safety and
Projects

SUBJECT: FINAL DRAFT OF THE RIVER BEND UNIT 1 TECHNICAL SPECIFICATIONS

This provides information on the Region IV review of the final draft technical specifications for River Bend. It is forwarded as requested by your letter of April 19, 1985.

A Region IV team inspection was conducted from April 29 through May 3, 1985. The results of this inspection were that some 25 items were identified, which appear to be under NRR cognizance. These are documented in the attached inspection report forms. Except for the attached, no additional items were identified which we feel require action on your part. There were, however, over 300 items related to the licensee's implementing procedures. These will be documented in Inspection Report 50-458/85-35.

Although our inspection of the draft technical specifications was primarily directed toward the licensee's readiness to implement them, we did review the technical specifications for obvious discrepancies between them, the Safety Evaluation Report (NUREG-0989, including Supplement 1), and the Final Safety Analysis Report (through Amendment 17). There were no problems found in this area, which are not already noted in the attached report forms.

Original Signed By

JH E. H. Johnson

Richard P. Denise, Director
Division of Reactor Safety and
Projects

Attachments:
As stated

bcc:

R. D. Martin
R. P. Denise
E. H. Johnson
J. P. Jaudon
W. Seidle
D. Chamberlain
R. Farrell
RPB1
AI File
RIV File

RIV:RPB1/A
JPJaudon/1k
5/10/85

RPB1 *EH*
EHJohnson
5/10/85

DRSP *RPD*
RPDenise
5/10/85
For

AI 85-266

~~8509060236~~
13pp.

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.10.4.1

PAGE NO.: 3/4 10-4

TS REQUIREMENT: THE TIME DURING WHICH THE ABOVE SPECIFIED REQUIREMENT HAS BEEN SUSPENDED SHALL BE VERIFIED TO BE LESS THAN 24 HOURS AT LEAST ONCE PER HOUR DURING PHYSICS TESTS (SEE CONTINUATION SHEET)

LIC. PROCEDURE NO.: STP-053-0701 ISSUE DATE: 1-07-85 REV.: 0 4

TITLE: REACTOR RECIRCULATION LOOPS OPERATION DURING TESTING

- | | <u>YES</u> | <u>NO</u> |
|---|-------------|-------------|
| 1. Is there any difference between FSAR/SER and TS? | <u>NA</u> | <u>NA</u> |
| 2. Is installed system consistent with TS? | <u>NA</u> | <u>NA</u> |
| 3. Are there any problems with the TS (factual or editorial)? | <u>✓</u> | <u> </u> |
| 4. Does procedure carry out TS requirement? | <u> </u> | <u>✓</u> |
| 5. Does the procedure walkdown indicate that it should work as written? | <u>NA</u> | <u>NA</u> |
| 6. Has licensee completed procedure walkdown? | <u> </u> | <u>✓</u> |

REMARKS: ③ SURVEILLANCE REQUIREMENTS SHOULD INCLUDE REQUIREMENT FOR "A SECOND LICENSEE OPERATOR OR OTHER TECHNICALLY QUALIFIED MEMBER OF THE UNIT TECHNICAL STAFF IS PRESENT AND VERIFIES COMPLIANCE WITH THE PHYSICS TEST OR STARTUP TEST PROCEDURE.

④ STP STEPS 7.4, 7.5 AND 7.6 NOT CLEAR AND WOULD NEVER RESULT IN STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

- ③ NRC Reconcile by fuel load
- 4. Licensee reconcile by fuel load

INSPECTOR(S): Dwight D. Chamberlain

FOLLOWUP ACTION: INSPECTION REPORT 50-458/

- | | <u>YES</u> | <u>NO</u> |
|--|-------------|-------------|
| Were problems corrected? | <u> </u> | <u> </u> |
| Were any other problems identified? | <u> </u> | <u> </u> |
| STATUS OF ITEM OPEN CLOSED | | |

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 75-75 PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.10.5

PAGE NO.: 3/4 10-5

TS REQUIREMENT: THE REACTOR VESSEL SHALL BE VERIFIED TO BE UNPRESSURIZED AND THE THERMAL POWER AND REACTOR COOLANT TEMPERATURE SHALL BE VERIFIED (SEE CONTINUATION SHEET)

LIC. PROCEDURE NO.: STP-507-3702 ISSUE DATE: 2-27-85 REV.: 0

TITLE: THERMAL POWER, REACTOR COOLANT TEMPERATURE AND REACTOR VESSEL PRESSURE VERIFICATION DURING TRAINING STARTUPS

	YES	NO
1. Is there any difference between FSAR/SER and TS?	NA	NA
2. Is installed system consistent with TS?	NA	NA
3. Are there any problems with the TS (factual or editorial)?	✓	
4. Does procedure carry out TS requirement?	✓	
5. Does the procedure walkdown indicate that it should work as written?		✓
6. Has licensee completed procedure walkdown?		✓

REMARKS: (3) SURVEILLANCE REQUIREMENTS SHOULD INCLUDE REQUIREMENT FOR "A SECOND LICENSED OPERATOR OR OTHER TECHNICALLY QUALIFIED MEMBER OF THE UNIT TECHNICAL STAFF IS PRESENT AND VERIFIES COMPLIANCE WITH TRAINING STARTUP PROCEDURES."

(5) STEP 7.5 - NEED TO DEFINE HOW TO OBTAIN THERMAL POWER

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

(3) NRC RECONCILE BY FUEL LOAD

(5) LICENSEE RECONCILE BY FUEL LOAD

INSPECTOR(S): DWIGHT D. CHAMBERLAIN

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	YES	NO
Were problems corrected?		
Were any other problems identified?		
STATUS OF ITEM OPEN CLOSED		

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 75-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.2.1

PAGE NO.: 3/4 2-1

TS REQUIREMENT: VERIFY APLHGRS ARE WITHIN LIMITS AT THE SPECIFIED TIMES.

LIC. PROCEDURE NO.: STP-050-300 ISSUE DATE: 3/21/85 REV.: 0

TITLE: POWER DISTRIBUTION LIMITS VERIFICATION

	<u>YES</u>	<u>NO</u>
Is there any difference between FSAR/SER and TS?	<u>NOT</u>	<u>VERIFIED</u>
Is installed system consistent with TS?	<u>NOT</u>	<u>VERIFIED</u>
Are there any problems with the TS (factual or editorial)?	<u>X</u>	_____
Does procedure ^{APPEAR TO} carry out TS requirement?	<u>X</u>	_____
Does the procedure walkdown indicate that it should work as written?	<u>NOT</u>	<u>VERIFIED</u>
Has licensee completed procedure walkdown?	_____	<u>X</u>

REMARKS: ON PAGE 3/4 2-2, THE FUEL NUMBER SHOULD BE P8S1B071 NOT P8S1B094. THE APPLICANT HAS PROVIDED 5 MAPLHGR CURVES PRIOR TO AMENDING THE FSAR WHICH IDENTIFIES 3 FUEL TYPES

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

NRR MUST CORRECT THE TYPOGRAPHICAL ERROR AND REVIEW THE MAPLHGR CURVES. THE LICENSEE MUST REQUEST AN AMENDMENT TO THE FSAR. ALL EFFORTS DUE PRIOR TO OL.

INSPECTOR(S): DALE A. POWERS

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	<u>YES</u>	<u>NO</u>
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.2.1

PAGE NO.: 3/4 2-1

TS REQUIREMENT: VERIFY APLHGRS ARE WITHIN LIMITS AT THE SPECIFIED TIMES.

LIC. PROCEDURE NO.: STP-050-3001 ISSUE DATE: 3/21/85 REV.: 0

TITLE: POWER DISTRIBUTION LIMITS VERIFICATION

	<u>YES</u>	<u>NO</u>
Is there any difference between FSAR/SER and TS?	<u>NOT</u>	<u>VERIFIED</u>
Is installed system consistent with TS?	<u>NOT</u>	<u>VERIFIED</u>
Are there any problems with the TS (factual or editorial)?	<u>X</u>	<u> </u>
Does procedure ^{APPEAR TO} carry out TS requirement?	<u>X</u>	<u> </u>
Does the procedure walkdown indicate that it should work as written?	<u>NOT</u>	<u>VERIFIED</u>
Has licensee completed procedure walkdown?	<u> </u>	<u>X</u>

REMARKS: ON PAGE 3/4 2-2, THE FUEL NUMBER SHOULD BE P851B071 NOT P851B094. THE APPLICANT HAS PROVIDED 5 MAPLHGR CURVES PRIOR TO AMENDING THE FSAR WHICH IDENTIFIES 3 FUEL TYPES

STATUS OF ITEM (OPEN) CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

NRR MUST CORRECT THE TYPOGRAPHICAL ERROR AND REVIEW THE MAPLHGR CURVES. THE LICENSEE MUST REQUEST AN AMENDMENT TO THE FSAR. ALL EFFORTS DUE PRIOR TO OL.

INSPECTOR(S): DALE A. POWERS

FOLLOWUP ACTION: INSPECTION REPORT 50-458/

	<u>YES</u>	<u>NO</u>
Were problems corrected?	<u> </u>	<u> </u>
Were any other problems identified?	<u> </u>	<u> </u>

STATUS OF ITEM OPEN CLOSED

REMARKS:

INSPECTOR(S):

INSPECTION REPORT 50-458/ 85-35

PAGE NO.:

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4, 5.2

PAGE NO.: 3/4 5-6 Through 3/4 5-7

TS REQUIREMENT: Numerous - "ECCS Shutdown"

LIC. PROCEDURE NO.: N/A ISSUE DATE: N/A REV.: N/A

TITLE: N/A

	YES	NO	NOT REVIEWED
Is there any difference between FSAR/SER and TS?	_____	_____	✓
Is installed system consistent with TS?	_____	_____	✓
Are there any problems with the TS (factual or editorial)?	✓	_____	_____
Does procedure carry out TS requirement?	_____	_____	✓
Does the procedure walkdown indicate that it should work as written?	_____	_____	✓
Has licensee completed procedure walkdown?	_____	_____	✓

REMARKS: 1) reviewed the above sections for completeness, facts, and editorial. one typing mistake identified e.g. Section 3.5.2.e "... Specification 3.5.3.6 ..." is incorrect. 2) Verified that section 4.5.2 appears to identify appropriate surveillance requirements necessary to prove OPERABILITY of systems identified in section 3.5.2

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

Specification 3.5.3.6 given above should be 3.5.3.6 (N/A correct)

INSPECTOR(S): D.L. DuBois

FOLLOWUP ACTION: INSPECTION REPORT 50-458/_____

	YES	NO
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4.5.3

PAGE NO.: 3/4 5-8 Through 3/4 5-9

TS REQUIREMENT: Numerous - "Suppression Pool"

LIC. PROCEDURE NO.: N/A ISSUE DATE: N/A REV.: N/A

TITLE: N/A

	YES	NO	NOT Reviewed
Is there any difference between FSAR/SER and TS?	<u> </u>	<u> </u>	<u>✓</u>
Is installed system consistent with TS?	<u> </u>	<u> </u>	<u>✓</u>
Are there any problems with the TS (factual or editorial)?	<u>✓</u>	<u> </u>	<u> </u>
Does procedure carry out TS requirement?	<u> </u>	<u> </u>	<u>✓</u>
Does the procedure walkdown indicate that it should work as written?	<u> </u>	<u> </u>	<u>✓</u>
Has licensee completed procedure walkdown?	<u> </u>	<u> </u>	<u>✓</u>

REMARKS: 1) reviewed above sections for completeness, facts, and editorial. Section 4.5.3.2 is missing part b. 2) verified that section 4.5.3 appears to identify appropriate surveillance requirements necessary to prove OPERABILITY of systems identified in section 3.5.3

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

Section 4.5.3.2. b is missing. It follows the "or" statement of 4.5.3.2.a.

INSPECTOR(S): D. L. DuBois

FOLLOWUP ACTION: INSPECTION REPORT 50-458/

	YES	NO
Were problems corrected?	<u> </u>	<u> </u>
Were any other problems identified?	<u> </u>	<u> </u>

STATUS OF ITEM OPEN CLOSED

REMARKS:

INSPECTOR(S):

INSPECTION REPORT 50-458/ 85-35

PAGE NO.:

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.6.1.1.B PAGE NO.: 3/4 6-1
 TS REQUIREMENT: Primary Containment Integrity - closed
value verification

LIC. PROCEDURE NO.: 000-0201 ISSUE DATE: 1/1/85 REV.: 0
 TITLE: Monthly Operating Logs

	YES	NO
Is there any difference between FSAR/SER and TS?	---	<u>NA</u>
Is installed system consistent with TS?	---	<u>NA</u>
Are there any problems with the TS (factual or editorial)?	<u>✓</u>	---
Does procedure carry out TS requirement?	---	---
Does the procedure walkdown indicate that it should work as written?	---	---
Has licensee completed procedure walkdown?	---	---

REMARKS: Double asterisk: note at bottom of tech spec page
3/4 6-1 talks of valves which are, "...inside the
primary containment, steam tunnel or drywell..."
This appears to be redundant since drywell is
totally inside primary containment

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

Verify wording

INSPECTOR(S): Farrell

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	YES	NO
Were problems corrected?	---	---
Were any other problems identified?	---	---

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ _____ PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.6.5.1A

PAGE NO.: 3/4 6-49

TS REQUIREMENT: _____

LIC. PROCEDURE NO.: 000-0001 ISSUE DATE: 8/24/84 REV.: Ø

TITLE: Daily ops log

	YES	NO
Is there any difference between FSAR/SER and TS?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Is installed system consistent with TS?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Are there any problems with the TS (factual or editorial)?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Does procedure carry out TS requirement?	<input type="checkbox"/>	<input type="checkbox"/>
Does the procedure walkdown indicate that it should work as written?	<input type="checkbox"/>	<input type="checkbox"/>
Has licensee completed procedure walkdown?	<input type="checkbox"/>	<input type="checkbox"/>

REMARKS: Procedure says leak rate is a "LATER" - but can be taken from Tech spec. Tech spec says to pressure curlock to 19.2 (psig) [i.e., 33.9 psia] and measure leak rate at 3 psid.

STATUS OF ITEM

OPEN

CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

The numbers do not make sense. Verify '19.2 as pounds per square inch G.A.G.B'

INSPECTOR(S): _____

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	YES	NO
Were problems corrected?	<input type="checkbox"/>	<input type="checkbox"/>
Were any other problems identified?	<input type="checkbox"/>	<input type="checkbox"/>

STATUS OF ITEM

OPEN

CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ _____

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.6.5.1 A

PAGE NO.: 3/4 6-49

TS REQUIREMENT: Secondary Containment - verify daily that the pressure in Shield B

LIC. PROCEDURE NO.: 000-000/ ISSUE DATE: 8/24/95 REV.: 0

TITLE: Daily Ops log

	YES	NO
Is there any difference between FSAR/SER and TS?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Is installed system consistent with TS?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Are there any problems with the TS (factual or editorial)?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Does procedure carry out TS requirement?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Does the procedure walkdown indicate that it should work as written?	<input type="checkbox"/>	<input type="checkbox"/>
Has licensee completed procedure walkdown?	<input type="checkbox"/>	<input type="checkbox"/>

REMARKS: Building and annulus pressures in TS do not appear to agree with SER, para 6.2.3, page 6-20

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

Verify TS numbers are correct.

INSPECTOR(S): Farrell

FOLLOWUP ACTION: INSPECTION REPORT 50-458/_____

	YES	NO
Were problems corrected?	<input type="checkbox"/>	<input type="checkbox"/>
Were any other problems identified?	<input type="checkbox"/>	<input type="checkbox"/>

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/_____ PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 4.6.5-1.C 1 & 2

PAGE NO.: 3/4 6-50

TS REQUIREMENT: Standby Gas Treatment System Operability

LIC. PROCEDURE NO.: 257-0607 ISSUE DATE: 1/30/85 REV.: Ø

TITLE: SG-TS operability Test

	<u>YES</u>	<u>NO</u>
Is there any difference between FSAR/SER and TS?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Is installed system consistent with TS?	<input type="checkbox"/>	<input type="checkbox"/>
Are there any problems with the TS (factual or editorial)?	<input type="checkbox"/>	<input type="checkbox"/>
Does procedure carry out TS requirement?	<input type="checkbox"/>	<input type="checkbox"/>
Does the procedure walkdown indicate that it should work as written?	<input type="checkbox"/>	<input type="checkbox"/>
Has licensee completed procedure walkdown?	<input type="checkbox"/>	<input type="checkbox"/>

REMARKS: Flow rates/Times in Tech Spec appear to be at variance with SER, para 6.2.3, pg 6-21

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

Verify correctness of TS numbers.

INSPECTOR(S): Farrell

FOLLOWUP ACTION: INSPECTION REPORT 50-458/_____

	<u>YES</u>	<u>NO</u>
Were problems corrected?	<input type="checkbox"/>	<input type="checkbox"/>
Were any other problems identified?	<input type="checkbox"/>	<input type="checkbox"/>

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/_____ PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4 8.1.1

PAGE NO.: 3/4 8-4

TS REQUIREMENT: 4.8.1.1.2.f

General Comment

LIC. PROCEDURE NO.: N/A

ISSUE DATE: _____

REV.: _____

TITLE: _____

	YES	NO
Is there any difference between FSAR/SER and TS?	_____	<u>N/A</u>
Is installed system consistent with TS?	_____	<u>N/A</u>
Are there any problems with the TS (factual or editorial)?	<u>✓</u>	_____
Does procedure carry out TS requirement?	_____	<u>N/A</u>
Does the procedure walkdown indicate that it should work as written?	_____	<u>N/A</u>
Has licensee completed procedure walkdown?	_____	<u>N/A</u>

REMARKS: Frequency for performance of all items listed under this section f is in error. T.S reads 8 months instead of correct 18 months.

STATUS OF ITEM OPEN CLOSED

FOR "OPENS", WHO MUST DO WHAT BY WHEN?

NRC must correct T.S.

INSPECTOR(S): C.C. Harbuck

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

	YES	NO
Were problems corrected?	_____	_____
Were any other problems identified?	_____	_____

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

INSPECTION REPORT 50-458/ 85-35

PAGE NO.: _____

TECHNICAL SPECIFICATION REVIEW DATA SHEET

TS PARA: 3/4.8.1.1

PAGE NO.: 3/4 8-7

TS REQUIREMENT: 4.8.1.1.2.f. 13 Each of the above OGA shall be demonstrated OPERABLE: at least once per 8 mos. during shutdown, by:
(see attached page)

LIC. PROCEDURE NO.: STP-305-0607 ISSUE DATE: 2-11-85 REV.: 0

TITLE: DIV I Diesel Generator Lockout Operability Test

0608 DIV II Rev 0 2-11-85
 0609 DIV III Rev 0 2-27-85

	YES	NO
Is there any difference between FSAR/SER and TS?	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Is installed system consistent with TS?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
Are there any problems with the TS (factual or editorial)?	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Does procedure ^{appear to} carry out TS requirement?	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Does the procedure walkdown indicate that it should work as written?	<input type="checkbox"/>	<u>N/A</u>

Has licensee completed procedure walkdown?

REMARKS: ① Formal of procedures inconsistent with ADM-0015

- ① TS referenced wrong
- ② Procedures do not appear to meet intent of the TS. Once a particular lockout is inserted no attempt is made to start the diesel; proof that it won't start

STATUS OF ITEM OPEN CLOSED (continued)

FOR "OPENS", WHO MUST DO WHAT BY WHEN:

- ① licensee must correct procedure
- ② Licensee must revise procedure to carry out TS requirement
- ③ REC must clarify TS requirement

INSPECTOR(S): E.C. Harbuck

FOLLOWUP ACTION: INSPECTION REPORT 50-458/ _____

YES NO

Were problems corrected?

Were any other problems identified?

STATUS OF ITEM OPEN CLOSED

REMARKS: _____

INSPECTOR(S): _____

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