340-20 E. Butchen

¥

B/ 22



#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

MEMORANDUM FOR: Cecil Thomas, Chief Standardization and Special Projects Branch Division of Licensing

FROM:

Frank C. Cherny, Acting Chief Mechanical Engineering Branch Division of Engineering

SUBJECT:

REVIEW OF RIVER BEND, UNIT 1 TECHNICAL SPECIFICATIONS

The Mechanical Engineering Branch has reviewed the following sections of the River Bend, Unit 1 Technical Specifications Draft dated April 26, 1985:

3/4.4.3 Reactor Coolant System Leakage 3/4.7.4 Snubbers

Attached are copies of pages 3/4.4-9 of Section 3/4.4.3 and Surveillance Requirement 4.4.3.2.2 (page 3/4.4-10 of Section 3/4.4.3) with the MEB comments. The change on page 3/4.4-9 is required in order to conform to standard technical specification criteria. We are recommending that the sub-paragraphs c and d on pages 3/4.4-10 be added unless the applicant can state that the following conditions exist at the Perry, Unit 1 plant for all Pressure Isolation Valves (PIVs):

- full closure of the PIVs is verified in the control room by direct monitoring position indicators,
- (2) Inadvertent opening of the PIVs is prevented by interlocks which require the primary system pressure to be below subsystem design pressure prior to opening, and
- (3) gross intersystem leakages into the core spray, low-pressure coolant injection, and shutdown cooling, return and suction lines would be detected by high-pressure alarms and increases in the suppression pool level.

In Table 3.4.3.2-1, page 3/4.4-11, valve 1RHS\*V240 is a 1° check valve which is placed downstream of an orifice (size unknown) from the RCS. It does not appear to connect to low pressure piping (Ref: drawing 12210-FSK-27-7E). The applicant may want to confirm the above and, if correct, remove this valve from the RCS Pressure Isolation Valve list in the table.

Section 3/4.7.4 is acceptable as written.

Frank C. Cherny, Acting Chief

8506140403XA 466

Frank C. Cherny, Acting Chief Mechanical Engineering Branch Division of Engineering

Attachment: As stated

cc: See Page 2.

Cecil Thomas

cc: R. Bosnak, DE S. Brown, DL H. Brammer, DE O. Rothberg, DE H. Shaw, DE

CONTACT: H. Shaw,,DE:MEB 49-24420 0. Rothberg, DE:MEB 49-27864

## REACTOR COOLANT SYSTEM

## OPERATIONAL LEAKAGE

# LIMITING CONDITION FOR OPERATION

- 3.4.3.2 Reactor coolant system leakage shall be limited to:
  - No PRESSURE BOUNDARY LEAKAGE. a.
  - 5 gpm UNIDENTIFIED LEAKAGE. b.
  - 25 gpm total leakage (averaged over any 24-hour period). c.
  - 1 gpm leakage at a reactor coolant system pressure of 1025 ± 15 psig d. from any reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

#### ACTION:

- With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within a. 12 hours and in COLD SHUTDOWN within the next 24 hours.
- With any reactor coolant system leakage greater than the limits in b b. and/or c, above, reduce the leakage rate to within the limits within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

With any reactor coolant system pressure isolation valve leakage greater c. than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two other closed manual \$\$\$\$ deactivated automatic or check\* valves, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

which have been verified not to exceed the allowable leakage limit at the last refueiing outage or after the last time the valve was disturbed, whichever is more recent.

RIVER BEND - UNIT 1

FINAL DRAFT

# REACTOR COOLANT SYSTEM

# FINAL DRAFT

APR 2 6 1985 25/5/55

#### SURVEILLANCE REQUIREMENTS

4.4.3.2.1 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

- a. Monitoring the drywell atmospheric particulate radioactivity at least once per 12 hours.
- Monitoring the drywell and pedestal floor drain sump flow rates at least once per 12 hours,
- c. Monitoring the drywell air coolers condensate flow rate at least once per 12 hours, and
- d. Monitoring the reactor vessel head flange leak detection system at least once per 24 hours.

4.4.3.2.2 Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE by leak testing pursuant to Specification 4.0.5 and verifying the leakage of each valve to be within the specified limit:

- a. At least once per 18 months, and
- b. Prior to returning the valve to service following maintenance, repair or replacement work on the valve which could affect its leakage rate fand

The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 3.

\* a Prior to entance Hor Surrower wheneve the plant has been in Coro Surrows for 72 have or more and if leakage testing has not been performed in the presions nime months and the device following value actuation due to automatic or manual action or flow through the value.

RIVER BEND - UNIT 1